

August 1, 1996

Mr. Gregory M. Rueger  
Pacific Gas and Electric Company  
NPG - Mail Code A10D  
P. O. Box 770000  
San Francisco, California 94177

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER PLANT,  
UNIT NO. 1 (TAC NO. M95434) AND UNIT NO. 2 (TAC NO. M95435)

Dear Mr. Rueger:

The Commission has issued the enclosed Amendment No. 114 to Facility Operating License No. DPR-80 and Amendment No. 112 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant (DCPP), Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated May 9, 1996.

These amendments revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2 to revise Technical Specifications (TS) 3/4.3.2, "Engineered Safety Features Actuation System Instrumentation," and 3/4.6.2, "Containment Spray System." The changes clarify the description of the initiation signal required for operation of the containment spray system at DCPP and correctly incorporate changes made in previous license amendments. All of the changes are administrative in nature.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed By  
Steven D. Bloom, Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-275  
and 50-323

- Enclosures: 1. Amendment No. 114 to DPR-80
- 2. Amendment No. 112 to DPR-82
- 3. Safety Evaluation

cc w/encls: See next page

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PDIV-2 Reading	GHill (4), T5C3
EPeyton	OGC, 015B18
LHurley, RIV	ACRS, T2E26
WBateman	DChamberlain, RIV
KPerkins, WCFO	CVM (SE)
SBloom	JDyer, RIV
JKilcrease, WCFO	TMarsh

DOCUMENT NAME: DC95434.AMD

OFC	LA:PDIV-2	PDIV-2	HIGB	OGC	PDIV-2
NAME	EPeyton	SBloom	JWermiel	C Marco	WBateman
DATE	7/8/96	7/8/96	7/9/96	7/10/96	7/13/96

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Pacific Gas and Electric Company  
NPG - Mail Code A10D  
P. O. Box 770000  
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These amendments revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2 to revise Technical Specifications (TS) 3/4.3.2, "Engineered Safety Features Actuation System Instrumentation," and 3/4.6.2, "Containment Spray System." The changes clarify the description of the initiation signal required for operation of the containment spray system at DCPP and correctly incorporate changes made in previous license amendments. All of the changes are administrative in nature.

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2. Amendment No. 112 to DPR-82  
3. Safety Evaluation

cc w/encls: See next page

cc:

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Diablo Canyon Nuclear Power Plant  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114  
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated May 9, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

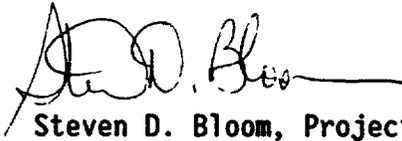
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(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 114, are hereby incorporated in the license. Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven D. Bloom, Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: August 1, 1996



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY  
DOCKET NO. 50-323  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112  
License No. DPR-82

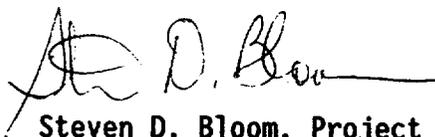
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated May 9, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 112, are hereby incorporated in the license. Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven D. Bloom, Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: August 1, 1996

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

3/4 3-16  
3/4 3-24  
3/4 3-32  
3/4 3-34  
3/4 3-35  
3/4 6-11

INSERT

3/4 3-16  
3/4 3-24  
3/4 3-32  
3/4 3-34  
3/4 3-35  
3/4 6-11

TABLE 3.3-3

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. Safety Injection (Reactor Trip, Feedwater Isolation, Start Diesel Generators, Containment Fan Cooler Units, and Component Cooling Water)					
a. Manual Initiation	2	1	2	1, 2, 3, 4	19
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4	14
c. Containment Pressure-High	3	2	2	1, 2, 3, 4	20
d. Pressurizer Pressure-Low	4	2	3	1, 2, 3 <del>4</del>	20
e. DELETED					
f. Steam Line Pressure-Low	3/steam line	2/steam line in any steam line	2/steam line	1, 2, 3 <del>4</del>	20

DIABLO CANYON - UNITS 1 & 2

3/4 3-15

Amendment Nos. 84 and 83  
84 and 83

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
2. Containment Spray (coincident with SI signal)					
a. Manual	2	2 with 2 coincident switches	2	1, 2, 3, 4	19
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4	14
c. Containment Pressure-High-High	4	2	3	1, 2, 3, 4	17
3. Containment Isolation					
a. Phase "A" Isolation					
1) Manual	2	1	2	1, 2, 3, 4	19
2) Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3, 4	14
3) Safety Injection	See Item 1. above for all Safety Injection initiating functions and requirements.				
b. Phase "B" Isolation					
1) Manual	2	2 with 2 coincident switches	2	1, 2, 3, 4	19

DIABLO CANYON - UNITS 1 & 2

3/4 3-16

Unit 1 - Amendment No. 89, 114  
Unit 2 - Amendment No. 88, 112

TABLE 3.3-4

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Safety Injection (Reactor Trip, Feedwater Isolation, Start Diesel Generators, Containment Fan Cooler Units, and Component Cooling Water)		
a. Manual Initiation	N.A.	N.A
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A
c. Containment Pressure-High	≤ 3 psig	≤ 3.3 psig
d. Pressurizer Pressure-Low	≥ 1850 psig	≥ 1844.4 psig
e. DELETED		
f. Steam Line Pressure-Low	≥ 600 psig (Note 1)	≥ 594.6 psig (Note 1)

DIABLO CANYON - UNITS 1 & 2

3/4 3-23

Amendment Nos. 37 & 36, 71 & 71,  
84 & 83

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
2. Containment Spray (coincident with SI signal)		
a. Manual Initiation	N.A.	N.A
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A
c. Containment Pressure-High-High	≤ 22 psig	≤ 22.3 psig
3. Containment Isolation		
a. Phase "A" Isolation		
1) Manual	N.A.	N.A
2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
3) Safety Injection	See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.	
b. Phase "B" Isolation		
1) Manual	N.A.	N.A
2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A
3) Containment Pressure-High-High	≤ 22 psig	≤ 22.3 psig

DIABLO CANYON - UNITS 1 & 2

3/4 3-24

Unit 1 - Amendment No. 84, 114  
Unit 2 - Amendment No. 83, 112

TABLE 3.3-5 (Continued)

TABLE NOTATIONS

- (1) Diesel generator starting delay not included because offsite power available.
- (2) Notation deleted.
- (3) Diesel generator starting and loading delays included.
- (4) Diesel generator starting delay not included because offsite power is available. Response time limit includes opening of valves to establish SI path and attainment of discharge pressure for centrifugal charging pumps (where applicable). Sequential transfer of charging pump suction from the VCT to the RWST (RWST valves open, then VCT valves close) is included.
- (5) Diesel generator starting and sequence loading delays included. Offsite power is not available. Response time limit includes opening of valves to establish SI path and attainment of discharge pressure for centrifugal charging pumps. Sequential transfer of charging pump suction from the VCT to the RWST (RWST valves open, then VCT valves close) is included.
- (6) The maximum response time of 48.5 seconds is the time from when the containment pressure exceeds the High-High Setpoint until the spray pump is started and the discharge valve travels to the fully open position assuming off-site power is not available. The time of 48.5 seconds includes the 28-second maximum delay related to ESF loading sequence. Spray riser piping fill time is not included. The 80-second maximum spray delay time does not include the time from LOCA start to "P" signal.
- (7) Diesel generator starting and sequence loading delays included. Sequential transfer of charging pump suction from the VCT to the RWST (RWST valves open, then VCT valves close) is not included. Response time limit includes opening of valves to establish SI flow path and attainment of discharge pressure for centrifugal charging pumps, SI, and RHR pumps (where applicable).
- (8) Does not include Trip Time Delays. Response times include the transmitters, Eagle-21 Process Protection cabinets, Solid State Protection System cabinets and actuation devices only. This reflects the response times necessary for THERMAL POWER in excess of 50% RTP.

TABLE 4.3-2

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

DIABLO CANYON - UNITS 1 & 2  3/4 3-32  Unit 1 - Amendment No. 61, 84, 89, 114 Unit 2 - Amendment No. 60, 83, 88, 112	FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALI- BRATION	CHANNEL OPERA- TIONAL TEST	TRIP	ACTUATION LOGIC TEST	MASTER RELAY TEST	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
					ACTUATING DEVICE OPERA- TIONAL TEST				
	1. Safety Injection, (Reactor Trip Feedwater Isolation, Start Diesel Generators, Containment Fan Cooler Units, and Component Cooling Water)								
	a. Manual Initiation	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
	b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	Q(4)	1, 2, 3, 4
	c. Containment Pressure-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3, 4
	d. Pressurizer Pressure-Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
	e. DELETED								
	f. Steam Line Pressure-Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
	2. Containment Spray (coincident with SI signal)								
	a. Manual Initiation	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
	b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	Q	1, 2, 3, 4
	c. Containment Pressure-High-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3, 4

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

DIABLO CANYON - UNITS 1 & 2 3/4 3-33 Unit 1 - Amendment No. 84, 87, 89, 102, 103 Unit 2 - Amendment No. 83, 86, 88, 101, 102		<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALI- BRATION</u>	<u>CHANNEL OPERA- TIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERA- TIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MASTER RELAY TEST</u>	<u>SLAVE RELAY TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
		3. Containment Isolation								
		a. Phase "A" Isolation								
		1) Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
		2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	Q(4)	1, 2, 3, 4
		3) Safety Injection	See Item 1. above for all Safety Injection Surveillance Requirements.							
		b. Phase "B" Isolation								
		1) Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
		2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	Q	1, 2, 3, 4
		3) Containment Pressure-High-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3, 4
		c. Containment Ventilation Isolation								
		1) Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M(1)	M(1)	Q	1, 2, 3, 4
		2) Deleted								
		3) Safety Injection	See Item 1. above for all Safety Injection Surveillance Requirements.							
		4) Containment Ventilation Exhaust Radiation-High (RM-44A and 44B)	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3, 4

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

DIABLO CANYON - UNITS 1 & 2  3/4 3-34  Unit 1 - Unit 2 - Amendment No. 61, 94, 103, 114 69, 93, 102, 112	FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALI- BRATION	CHANNEL OPERA- TIONAL TEST	TRIP ACTUATING DEVICE OPERA- TIONAL TEST	ACTUATION LOGIC TEST	MASTER RELAY TEST	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
	4. Steam Line Isolation								
	a. Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
	b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M <sup>(1)</sup>	M <sup>(1)</sup>	Q	1, 2, 3
	c. Containment Pressure-High-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
	d. Steam Line Pressure-Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
	e. Negative Steam Line Pressure Rate-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	3 <sup>(3)</sup>
	5. Turbine Trip and Feedwater Isolation								
	a. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M <sup>(1)</sup>	M <sup>(1)</sup>	Q	1, 2
	b. Steam Generator Water Level-High-High	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2
	6. Auxiliary Feedwater								
	a. Manual	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
	b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.	N.A.	N.A.	M <sup>(1)</sup>	M <sup>(1)</sup>	Q	1, 2, 3
	c. Steam Generator Water Level-Low-Low								
	1) Steam Generator Water Level-Low-Low	S	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3 <sup>(5)</sup>
	2) RCS Loop ΔT	N.A.	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2

TABLE 4.3-2 (Continued)

**ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS**

DIABLO CANYON - UNITS 1 & 2	FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALI- BRATION	CHANNEL OPERA- TIONAL TEST	TRIP ACTUATING DEVICE OPERA- TIONAL TEST	ACTUATION LOGIC TEST	MASTER RELAY TEST	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
	6. Auxiliary Feedwater (Continued)								
	d. Undervoltage - RCP	N.A.	R	N.A.	R	N.A.	N.A.	N.A.	1
	e. Safety Injection	See Item 1. above for all Safety Injection Surveillance Requirements.							
	7. Loss of Power								
	a. 4.16 kV Emergency Bus Level 1	N.A.	R	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
	b. 4.16 kV Emergency Bus Level 2	N.A.	R	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
3/4 3-35	8. Engineered Safety Feature Actuation System Interlocks								
Unit 1 -	a. Pressurizer Pressure, P-11	N.A.	R	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
Amendment No. 60, 83, 85, 102, 112	b. Deleted								
	c. Reactor Trip, P-4	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3

**TABLE NOTATIONS**

- (1) Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (2) For the Containment Ventilation Exhaust Radiation-High monitor only, a CHANNEL FUNCTIONAL TEST shall be performed at least once every 31 days.
- (3) Trip function automatically blocked above P-11 (Pressurizer Pressure Interlock) setpoint and is automatically blocked below P-11 when Safety Injection on Steam Line Pressure-Low is not blocked.
- (4) Except relays K612A, K614B, K615A, and K615B, which shall be tested, at a minimum, once per 18 months during refueling and during each Cold Shutdown unless they have been tested within the previous 92 days.
- (5) For Mode 3, the Trip Time Delay associated with the Steam Generator Water Level-Low-Low channel must be less than or equal to 464.1 seconds.

Unit 1 - Amendment No. 61, 84, 87, 103, 114  
 Unit 2 - Amendment No. 60, 83, 85, 102, 112

## INSTRUMENTATION

### 3/4.3.3 MONITORING INSTRUMENTATION

#### RADIATION MONITORING FOR PLANT OPERATIONS

##### LIMITING CONDITION FOR OPERATION

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3.3.3.1 The radiation monitoring instrumentation channels for plant operations shown in Table 3.3-6 shall be OPERABLE with their Alarm/Trip Setpoints within the specified limits.

APPLICABILITY: As shown in Table 3.3-6.

##### ACTION:

- a. With a radiation monitoring channel Alarm/Trip Setpoint for plant operations exceeding the value shown in Table 3.3-6, adjust the Setpoint to within the limit within 4 hours or declare the channel inoperable.
- b. With one or more radiation monitoring channels for plant operations inoperable, take the ACTION shown in Table 3.3-6.
- c. The provisions of Specification 3.0.3 are not applicable.

##### SURVEILLANCE REQUIREMENTS

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4.3.3.1 Each radiation monitoring instrumentation channel for plant operations shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST for the MODES and at the frequencies shown in Table 4.3-3.

## CONTAINMENT SYSTEMS

### 3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

#### CONTAINMENT SPRAY SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.6.2.1 Two Containment Spray Systems shall be OPERABLE with each Spray System capable of taking suction from the RWST and transferring spray function to a RHR System taking suction from the containment sump.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one Containment Spray System inoperable, restore the inoperable Spray System to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the inoperable Spray System to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.6.2.1 Each Containment Spray System shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position;
- b. By verifying that on recirculation flow, each pump develops a differential pressure of greater than or equal to 205 psid when tested pursuant to Specification 4.0.5;
- c. At least once per 18 months by:
  - 1) Verifying that each automatic valve in the flow path actuates to its correct position on an actual or simulated actuation signal, and
  - 2) Verifying that each spray pump starts automatically on an actual or simulated actuation signal.
- d. At least once per 10 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.

## CONTAINMENT SYSTEMS

### SPRAY ADDITIVE SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.6.2.2 The Spray Additive System shall be OPERABLE with:

- a. A spray additive tank with a contained volume of between 2025 and 4000 gallons of between 30 and 32% by weight NaOH solution, and
- b. Two spray additive eductors each capable of adding NaOH solution from the chemical additive tank to a Containment Spray System pump flow.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With the Spray Additive System inoperable, restore the system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours; restore the Spray Additive System to OPERABLE status within the next 48 hours or be in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.6.2.2 The Spray Additive System shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position;
- b. At least once per 6 months by:
  - 1) Verifying the contained solution volume in the tank, and
  - 2) Verifying the concentration of the NaOH solution by chemical analysis.
- c. At least once per 18 months by verifying that each automatic valve in the flow path actuates to its correct position on a Containment Spray actuation test signal; and
- d. At least once per 5 years by verifying both spray additive and RWST full flow from the test valve 8993 in the Spray Additive System.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-80  
AND AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-82  
PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By application dated May 9, 1996, Pacific Gas and Electric Company (or the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License Nos. DPR-80 and DPR-82) for the Diablo Canyon Nuclear Power Plant, Units 1 and 2. The proposed changes revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant (DCPP), Unit Nos. 1 and 2 to revise Technical Specifications (TS) 3/4.3.2, "Engineered Safety Features Actuation System Instrumentation," and 3/4.6.2, "Containment Spray System." The changes clarify the description of the initiation signal required for operation of the containment spray (CS) system at DCPP and correctly incorporate changes made in previous license amendments (LAs). All of the changes are administrative in nature.

2.0 EVALUATION

2.1 Containment Spray

The licensee proposed to change TS Tables 3.3-3, 3.3-4, and 4.3-2 to provide uniformity across the instrumentation TS regarding CS initiation. TS 3.3.2, Table 3.3-5, "Engineered Safety Features Response Times," Initiating Signal and Function 1.b.1), already lists the spray initiating signal as "Containment Spray (Coincident with SI Signal)." Table 3.3-5 was reviewed and approved as an administrative change as documented in the original Unit 1 and 2 Combined TS, issued as NUREG-1151 in August 1985.

The change to TS 4.6.2.1.c.1) and 2) revises the TS from verification of CS pump and valve actuation on "a Phase B Isolation test" signal to verification on "an actual or simulated actuation" signal. This wording envelopes the interlocking SI signal which was previously understood to be part of the Phase B test signal. The proposed signal descriptions conform to NUREG-1431, Rev. 1, TS 3.6.6A, "Containment Spray and Cooling Systems (Atmospheric and Dual)," Surveillance Requirements (SRs) 3.6.6A5 and 3.6.6A.6.

Containment Phase B isolation signals are generated for two purposes: CS initiation and essential process line isolation. Automatic Phase B isolation signals are generated when two out of four containment pressure channels exceed 22 psig. Manual Phase B signals are generated by actuating two control board switches simultaneously. The SI signal on containment high pressure is generated when two-out-of-three containment pressure channels exceed 3 psig. Containment spray pumps start and automatic discharge valves open upon receipt of a phase B isolation signal only when a coincident SI signal is present. The SI signal functions to initiate the automatic timers for all of the engineered safety feature (ESF) pumps, including the CS pumps. The interlocking SI signal is required for either automatic or manual actuation of the CS system. The required interlock is generated by any manual or automatic SI actuation. The interlock signal is physically provided by solid state protection system slave relays which actuate on a SI signal. No SI interlock is required for essential process line (Phase B) isolation to occur. An interlocking SI signal is not required to initiate spray using the individual pump and valve control switches located in the control room. The interlock is used to ensure the CS pumps will load onto the vital 4kV busses in the proper sequence with other ESF pumps. A secondary function is to prevent personnel from inadvertently spraying the containment during testing.

The staff has evaluated this change and concludes that these changes are administrative and are therefore acceptable.

## 2.2 Administrative Corrections to Table 4.3-2

The proposed administrative revisions to TS Table 4.3-2, pages 3/4 3-32, 3/4 3-34, and 3/4 3-35 correct the headings that were inadvertently revised in various LAs. Also, on page 3/4 3-32, a notation is restored defining the frequency for specific slave relay tests.

For TS page 3/4 3-32, changes made in LA 84/83 to have the fourth and fifth column headings read "CHANNEL OPERATIONAL TEST" and "TRIP ACTUATING DEVICE OPERATIONAL TEST," respectively, and in LA 87/86 to add notation "(4)" to Functional Unit 1.b were inadvertently revised in LA 89/88 issued on March 2, 1994. For pages 3/4 3-34 and 3/4 3-35, changes made in LA 84/83 to have the fourth and fifth column headings read "CHANNEL OPERATIONAL TEST" and "TRIP ACTUATING DEVICE OPERATIONAL TEST," respectively, were inadvertently revised in LA 103/102 issued on June 2, 1995.

The staff has evaluated this change and concludes that these changes are administrative and are therefore acceptable.

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 31184). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Bloom

Date: August 1, 1996

August 1, 1996

Mr. Gregory M. Rueger  
Pacific Gas and Electric Company  
NPG - Mail Code A10D  
P. O. Box 770000  
San Francisco, California 94177

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER PLANT,  
UNIT NO. 1 (TAC NO. M95434) AND UNIT NO. 2 (TAC NO. M95435)

Dear Mr. Rueger:

The Commission has issued the enclosed Amendment No. 114 to Facility Operating License No. DPR-80 and Amendment No. 112 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant (DCPP), Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated May 9, 1996.

These amendments revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2 to revise Technical Specifications (TS) 3/4.3.2, "Engineered Safety Features Actuation System Instrumentation," and 3/4.6.2, "Containment Spray System." The changes clarify the description of the initiation signal required for operation of the containment spray system at DCPP and correctly incorporate changes made in previous license amendments. All of the changes are administrative in nature.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed By  
Steven D. Bloom, Project Manager  
Project Directorate IV-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-275  
and 50-323

Enclosures: 1. Amendment No. 114 to DPR-80  
2. Amendment No. 112 to DPR-82  
3. Safety Evaluation

cc w/encls: See next page

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