

50-275/373



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 25, 1996

Mr. Gregory M. Rueger
Pacific Gas and Electric Company
NPG - Mail Code A10D
P. O. Box 770000
San Francisco, California 94177

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER PLANT,
UNIT NO. 1 (TAC NO. M94353) AND UNIT NO. 2 (TAC NO. M94354)

Dear Mr. Rueger:

The Commission has issued the enclosed Amendment No. 117 to Facility Operating License No. DPR-80 and Amendment No. 115 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated December 19, 1995, as supplemented by letter dated August 8, 1996.

These amendments would revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2 to relocate Technical Specification (TS) 6.5, "Review and Audit," 6.8, "Procedures and Programs," Sections 6.8.1c., 6.8.1d., 6.8.2, and 6.8.3, in accordance with guidance in an NRC letter dated October 25, 1993, from William T. Russell, Director, Office of Nuclear Reactor Regulation, to the chairpersons of industry owners groups and the Commission's Final Policy Statement on TS Improvements for Nuclear Power Reactors on relocation of TS that do not satisfy the retention criteria. As part of the relocation of TS 6.8.2, TS 6.1.1 would be revised to require that proposed tests, experiments, or modifications that affect nuclear safety be approved by the plant manager or his designee prior to implementation.

The technical specification requirements relocated to licensee-controlled documents (Final Safety Analysis Report Update, Quality Assurance Program, Security Plan, and Emergency Plan) shall be controlled by 10 CFR 50.59 50.54(a), 50.54(p), and 50.54(q). The license amendment is effective as of its date of issuance to be implemented within 90 days from date of issuance. Implementation shall include relocation of technical specification requirements to the appropriate licensee-controlled document (Updated Final Safety Analysis Report, Quality Assurance Program, Security Plan, and Emergency Plan) as identified in the enclosed safety evaluation.

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Mr. Gregory M. Rueger

- 2 -

October 25, 1996

The Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed By

Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosures: 1. Amendment No. 117 to DPR-80
2. Amendment No. 115 to DPR-82
3. Safety Evaluation

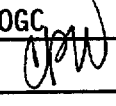
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CGrimes, 011E22
JBianchi, WCFO (2)
GHill (4), T5C3
OGC, 015B18
ACRS, T2E26
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DOCUMENT NAME: DC94353.AMD *See Previous Concurrence Sheet

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Mr. Gregory M. Rueger

- 2 -

October 25, 1996

The Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original Signed By

Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosures: 1. Amendment No. 117 to DPR-80
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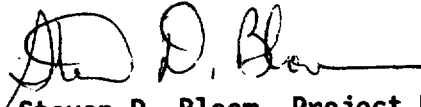
Mr. Gregory M. Rueger

- 2 -

October 25, 1996

The Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Steven D. Bloom", with a horizontal line extending from the end of the signature.

Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosures: 1. Amendment No. 117 to DPR-80
2. Amendment No. 115 to DPR-82
3. Safety Evaluation

cc w/encs: See next page

cc w/encls:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 117
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated December 19, 1995, as supplemented by letter dated August 8, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-80 is hereby amended to approve the relocation of certain technical specification requirements to licensee-controlled documents, as described in the licensee's application dated December 19, 1995, as supplemented by letter dated August 8, 1996, and reviewed in the staff's safety evaluation dated October 25, 1996. This license is also hereby amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

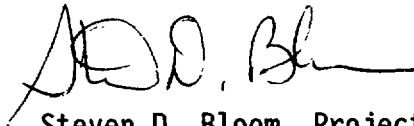
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(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 117, are hereby incorporated in the license. Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance to be implemented within 90 days from date of issuance. Implementation shall include the relocation of technical specification requirements to the appropriate licensee-controlled document as identified in the licensee's application dated December 19, 1995, as supplemented by letter dated August 8, 1996, and reviewed in the staff's safety evaluation dated October 25, 1996.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: October 25, 1996



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 115
License No. DPR-82

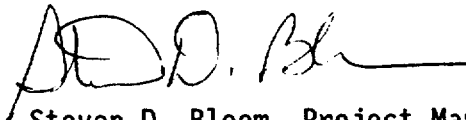
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated December 19, 1995, as supplemented by letter dated August 8, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-82 is hereby amended to approve the relocation of certain technical specification requirements to licensee-controlled documents, as described in the licensee's application dated December 19, 1995, as supplemented by letter dated August 8, 1996, and reviewed in the staff's safety evaluation dated October 25, 1996. This license is also hereby amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 115, are hereby incorporated in the license. Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. The license amendment is effective as of its date of issuance to be implemented within 90 days from date of issuance. Implementation shall include the relocation of technical specification requirements to the appropriate licensee-controlled document as identified in the licensee's application dated December 19, 1995, as supplemented by letter dated August 8, 1996, and reviewed in the staff's safety evaluation dated October 25, 1996.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: October 25, 1996

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 115 TO FACILITY OPERATING LICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President, Diablo Canyon Operations and Plant Manager, hereinafter called Plant Manager, shall be responsible for overall plant operation and shall delegate in writing the succession to this responsibility during his absence. The Plant Manager or his designee shall approve, prior to implementation, each proposed test, experiment or modification to systems or equipment that affects nuclear safety.

6.1.2 The Shift Foreman (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room Command function. A management directive to this effect signed by the Senior Vice President and General Manager, Nuclear Power Generation shall be reissued to all plant personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for plant operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR Update.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Senior Vice President and General Manager - Nuclear Power Generation shall have corporate responsibilities for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.0 ADMINISTRATIVE CONTROLS

6.2.2 PLANT STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the Control Room;
- c. A Health Physics Technician* shall be on site when fuel is in the reactor;

*The Health Physics Technician position may be unfilled for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the position.

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4 A retraining and replacement training program for the plant staff shall be maintained under the direction of a designated member of the facility staff and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.

6.5 DELETED

6.6 REPORTABLE EVENT ACTION

6.6. The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.73; and
- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and the results of this review submitted to NSOC and the Senior Vice President and General Manager, Nuclear Power Generation.

6.7 SAFETY LIMIT VIOLATION

6.7 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Senior Vice President and General Manager, Nuclear Power Generation and NSOC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe:
(1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems or structures, and
(3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, NSOC and the Senior Vice President and General Manager, Nuclear Power Generation within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;

ADMINISTRATIVE CONTROLS

6.8 PROCEDURES AND PROGRAMS (Continued)

- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Deleted;
- d. Deleted;
- e. PROCESS CONTROL PROGRAM implementation;
- f. ODCP and ERMP implementation;
- g. Quality Assurance Program for effluent and environmental monitoring; and
- h. Fire Protection Program Implementation.

6.8.2 Deleted.

6.8.3 Deleted.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include portions of the Recirculation Spray System, Safety Injection System, Chemical and Volume Control System, Residual Heat Removal System, RCS Sample System, and Liquid and Gaseous Radwaste Treatment Systems. The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 117 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 115 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By application dated December 19, 1995, as supplemented by letter dated August 8, 1996, Pacific Gas and Electric Company (or the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License Nos. DPR-80 and DPR-82) for the Diablo Canyon Nuclear Power Plant, Units 1 and 2. The proposed changes would revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2 to revise Section 6.1, "Responsibilities," and relocate Technical Specification (TS) 6.5, "Review and Audit," 6.8, "Procedures and Programs," Sections 6.8.1c., 6.8.1d., 6.8.2, and 6.8.3, in accordance with guidance in an NRC letter dated October 25, 1993, from William T. Russell to the chairpersons of industry owners groups and the Commission's Final Policy Statement on TS Improvements for Nuclear Power Reactors on relocation of TS that do not satisfy the retention criteria. As part of the relocation of TS 6.8.2, TS 6.1.1 would be revised to require that proposed tests, experiments, or modifications that affect nuclear safety be approved by the plant manager or his designee prior to implementation.

The August 8, 1996, supplemental letter provided additional clarifying information and did not change the initial no significant hazards consideration determination published in the Federal Register on January 22, 1996 (61 FR 1633).

2.0 EVALUATION

The provisions of the current TSs described below are not required by 10 CFR 50.36, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to public health and safety. Further, they do not fall within any of the four criteria set forth in the Commission's Final Policy Statement. Accordingly, the staff has concluded that these requirements may be relocated from the TSs to the plant procedures, which implement the regulations, or the QA Policy, as applicable. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.59 and 50.54(a) to control future changes and to assure continued protection of public health and safety. The documentation for future changes to those provisions relocated from the TSs will be maintained by the licensee

in accordance with the record retention requirements specified in their QA Policy. This approach ensures an auditable and appropriate control over the relocated requirements and future changes to these provisions.

2.1 Reviews and Audits

The licensee proposes that the review and audit functions specified in existing TS 6.5 be relocated from the improved TS on the basis that they are adequately controlled by the QA program. These TS provisions are not necessary to assure safe operation of the facility, given the requirements in the Quality Assurance (QA) Program implementing 10 CFR 50.54 and 10 CFR Part 50, Appendix B to control the requirements for all review and audit functions except those associated with the security and emergency plans. The security and emergency plan review and audit functions are relocated to their respective plans in accordance with GL 93-07, "Modification of the Technical Specification Administrative Control Requirements for Emergency and Security Plans." Additionally, NRC Administrative Letter, 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance," dated December 12, 1995, provides guidance for relocating TS administrative requirements. Such an approach would result in an equivalent level of regulatory authority while providing for a more appropriate change control process. The level of safety of plant operation is unaffected by this change and NRC and licensee resources associated with processing license amendments to this administrative control may be used more effectively. In addition, the following considerations support relocating these items from the TS:

- a. The Nuclear Safety Oversight Committee (NSOC) function, composition, alternate membership, meeting frequency, quorum, responsibilities, authority and records are all covered in equivalent detail in American National Standards Institute (ANSI) N18.7-1976. Since the NSOC requirements will be relocated to Chapter 17 of the Final Safety Analysis Report (FSAR) Update, which defines the QA program requirement, changes to the requirements will be controlled effectively under 10 CFR 50.54(a).
- b. The Plant Staff Review Committee (PSRC) is also addressed, although with less detail, in ANSI N18.7-1976. Since the PSRC requirements will be relocated to Chapter 17 of the FSAR Update, which defines the QA Program requirements, changes to the requirements will be controlled effectively under 10 CFR 50.54(a). Therefore, duplicating the review and audit function of NSOC in the TS is unnecessary.

In relocating TS Section 6.5.2 to the FSAR Chapter 17.2, the responsibilities of the PSRC will be revised to permit qualified reviewers to perform reviews of procedure and program changes. All proposed changes are required to be evaluated by the qualified reviewers to determine if a 10 CFR 50.59 safety evaluation is required in order to make the change. If a 10 CFR 50.59 safety evaluation is required, it will be reviewed by the PSRC. Qualified Reviewers will also be required to determine whether additional cross-disciplinary reviews are necessary. If necessary, the cross-discipline reviews will be performed by appropriately qualified reviewers.

- c. TS Section 6.5.4 requirements for the independent technical review function will be relocated, without change, to FSAR Section 17.2.5. Since requirements of this program will be relocated to Chapter 17 of the FSAR Update, changes to the requirements will be controlled effectively under 10 CFR 50.54(a). Chapter 17 of the FSAR Update defines QA Program requirements. The revisions to Chapter 17 will include minimum qualification requirements of review personnel.
- d. Audit requirements are specified in the QA program description to satisfy 10 CFR Part 50, Appendix B, Criterion XVIII. Audits requirements are also covered by ANSI N18.7, ANSI N45.2, 10 CFR 50.54(t), 10 CFR 50.54(p), and 10 CFR Part 73. Therefore, the audit requirements in TS 6.5 duplicate requirements contained in regulations and can be relocated. Since the audit requirements will be relocated to Chapter 17 of the FSAR Update, which defines the QA Program requirements, changes to the requirements will be controlled effectively under 10 CFR 50.54(a). Therefore duplicating of these requirements does not enhance the level of safety of the plant, nor are the provisions relating to audits necessary to assure safe operation of the facility.

TS 6.5.3.8 requirements for auditing plant activities has been relocated to Chapter 17 of the FSAR with the following audit frequencies changes

1. The Frequency for the conformance of plant operation to provisions contained within the Technical Specifications and applicable licenses was changed from 12 months to 24 months.
2. The Frequency for the performance, training, and qualifications of the entire plant staff was changed from 12 months to 24 months.
3. The Frequency for the results of actions taken to correct deficiencies occurring in plant equipment, structures, systems, or method of operation that affect nuclear safety was changed from 6 months to 24 months.
4. The Frequency for the Fire Protection and Loss Prevention Program will be performed in accordance with the annual (1 year) and biennial (2 year) audit and inspection requirements of NRC Generic Letter (GL) 82-21. Further the audit for the Fire Protection and Loss Prevention Program will be performed in accordance with the annual and triennial (3 year) audit requirements of NRC GL 82-21.
5. The Frequency for the Radiological Environmental Monitoring Program, its implementing procedures, and program results was changed from 12 months to 24 months.

6. The Frequency for Non-radiological Environmental Monitoring Program was changed from 12 to 24 months.

- e. The requirements for record retention in TS 6.5 are addressed in 10 CFR 50, Appendix B, Criterion XVII, and in Chapter 17 of the FSAR Update. Since the requirements are specified in regulations and the QA Program, changes to the requirements will be controlled effectively under 10 CFR 50.54(a).

Facility operations are performed in accordance with approved written procedures. These and other records regarding plant operation are retained to allow for review compliance with procedure requirements and regulations. However, post-compliance reviews of records do not affect safe plant operation since activities described in these documents have already been performed. Additionally, other regulations, including 10 CFR 20 and 10 CFR 50.71, require the retention of records related to plant operation. Existing regulatory requirements provide sufficient control of record retention requirements.

The licensee has proposed to relocate the provisions that are not otherwise covered by regulatory requirements to the FSAR Update. The staff concludes that the sufficient regulatory controls exist for the FSAR Update such that removing these provisions from the TS and relocating them to the FSAR Update under the controls of 10 CFR 50.54 is acceptable.

2.2 Security Plan and Emergency Plan Implementation

The licensee proposes to relocate the requirements to establish, implement, and maintain procedures related to the Security Plan (existing TS 6.8.1.c) and Emergency Plan (existing TS 6.8.1.d). Since the security plan requirements are specified in 10 CFR 50.54, 73.40, 73.55, and 73.56 and the emergency plan requirements are specified in 10 CFR 50.47, 50.54(q) and 10 CFR Part 50, Appendix E, Section V, the staff in GL 93-07 removed the requirements from the STS and relocated them to their respective plans.

The requirements in the existing TS for the review of the security program and implementing procedures, and for the review of the station emergency plan and implementing procedures, will be included in the FSAR Update. Further changes in these review requirements must be made in accordance with 10 CFR 50.54(p) for the Security Plan and 10 CFR 50.54(q) for the Emergency Plan.

The staff concludes that the extensive requirements for emergency planning in 10 CFR 50.47, 50.54, 10 CFR Part 50 Appendix E and for security in 10 CFR 50.54 and 73.55, for drills, exercises, testing, and maintenance of the program, provide adequate assurance that the objective of the previous TS for a periodic review of the program and changes to the programs will be met. Therefore, duplication of the requirements contained in the regulations would not enhance the level of safety for the facility. The staff concludes that other regulatory requirements provide sufficient control of these provisions and removing them from TS is acceptable.

2.3 Review and Approval Process

The licensee is proposing to relocate the requirements of existing TS 6.8.2 for the review and approval process for procedures to the FSAR Update. This proposal is based on the existence of the following requirements which duplicate 10 CFR Part 50 Appendix B in these areas.

The requirement for procedure control is mandated by 10 CFR Part 50, Appendix B, Criterion II and Criterion V. ANSI N18.7-1976, which is endorsed by the NRC in Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operations)," is used in the development of many licensee QA program descriptions, and contains specific requirements related to procedures. The licensee has committed to follow ANSI N18.7-1976 as a means to comply with 10 CFR Part 50, Appendix B. ANSI N18.7-1976, Section 5.2.2 discusses procedure adherence. Section 5.2.2 clearly states that procedures shall be followed, and the requirements for use of procedures shall be prescribed in writing. ANSI N18.7-1976 also requires review and approval of procedures to be defined. ANSI N18.7-1976, Section 5.2.15 describes the review, approval and control of procedures. The ANSI standard describes the requirements to provide measures to control and coordinate the approval and issuance of documents, including changes thereto, which prescribe all activities affecting quality. The ANSI standard further states that each procedure shall be reviewed and approved prior to initial use and describes the required reviews. ANSI N45.2-1971, Section 6, also specifies that the QA Program describe procedure requirements.

In relocating TS 6.5.1, "Technical Review and Control," administrative controls for processing procedures to FSAR 17.5, "Instructions, Procedures, and Drawings," the licensee clarified the intent of the relocated TS text by identifying which types of procedures implement the activities referenced in TS Section 6.8.1. The clarification, as described in FSAR 17.5, is that where reference is made to procedures in TS Section 6.8.1, the relocated TS 6.5.1.1 review and approval requirements only apply to procedures that are under the plant manager's control, affect plant operations, or affect the operating status of safety-related system, structures, or components. Also, in relocating TS 6.5.1.2 requirements to FSAR 17.5, the licensee has modified the relocated text to permit a responsible organization instead of a discipline qualified reviewer to be designated for reviewing and approving procedures. The responsible organization would then assign a qualified reviewer of the appropriate discipline to perform the review activity.

The licensee will continue to implement a QA program description in accordance with the requirements of 10 CFR Part 50, Appendix B, which provides appropriate controls for the review and approval of procedure changes. Changes to the QA program description that is incorporated in the FSAR Update by reference, including departures from the ANSI standard that constitute a reduction in commitment, will be governed by 10 CFR 50.54(a). The staff concludes that these regulatory requirements provide sufficient control of these provisions and removing them from the TS is acceptable.

2.4 Temporary Change Process

The licensee proposes to relocate the requirements of existing TS 6.8.3 for the temporary change process for procedures to the FSAR Update. This proposal is based on the existence of the following requirements which duplicate 10 CFR Part 50, Appendix B in these areas.

The requirement for procedure control is mandated by 10 CFR Part 50, Appendix B, Criterion II and Criterion V. ANSI N18.7-1976, which is endorsed by NRC in Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operations)," is used in the development of many licensee QA program descriptions and contains specific requirements related to procedures. The licensee has committed to follow ANSI N18.7-1976 as a means to comply with 10 CFR Part 50, Appendix B. ANSI N18.7-1976, Section 5.2.2 discusses procedure adherence. Section 5.2.2 clearly states that procedures shall be followed, and the requirements for use of procedures shall be prescribed in writing. ANSI N18.7-1976 also discusses temporary changes to procedures. ANSI N18.7-1976, Section 5.2.15 describes the review, approval and control of procedures. The ANSI standard describes the requirements for the licensee's QA Program to provide measures to control and coordinate the approval and issuance of documents, including changes thereto, which prescribe all activities affecting quality. The ANSI standard further states that each procedure shall be reviewed and approved prior to initial use, and describes the required reviews. ANSI N45.2-1971, Section 6, also specifies that the QA Program describe procedure requirements.

In relocating TS 6.8.3, the licensee has modified its text to remove the requirement for an individual with a senior reactor operator license to approve certain temporary procedure changes. The existing TS requirement would be revised in FSAR 17.5 to specify that a senior reactor operator is only required to approve temporary changes for the following types of procedures: (1) all operations section procedures, (2) surveillance test procedures, (3) emergency plan implementing procedures, and (4) any other procedure if the proposed change affects equipment of system operating status.

The licensee will continue to implement a QA program description in accordance with the requirements of 10 CFR Part 50, Appendix B, which provides appropriate controls for the review and approval of procedure changes. Changes to the QA program description that is incorporated in the FSAR Update by reference, including departures from the ANSI standard that constitute a reduction in commitment, will be governed by 10 CFR 50.54(a). The staff concludes that these regulatory requirements provide sufficient control of these provisions and removing them from TS is acceptable.

2.5 Administrative Change

As part of the relocation of TS 6.8.2, TS 6.1.1 would be revised to require that proposed tests, experiments, or modifications to systems or equipment that affects nuclear safety be approved by the plant manager or his designee prior to implementation. FSAR 17.5 has modified the relocated TS text to permit the plant manager to designate individuals to approve plant procedures

required by TS 6.8.1 on behalf of the plant manager. Designated approvers shall have an equivalent level of knowledge and experience to that of the plant manager. This change is an administrative change that incorporates information consistent with NUREG-1431, Revision 1, "Standard Technical Specifications Westinghouse Plants," dated April 7, 1995, and the staff finds it acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Bloom

Date: October 25, 1996