

September 13, 2002

The Honorable Harry Reid, Chairman
Subcommittee on Transportation,
Infrastructure, and Nuclear Safety
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510

Dear Mr. Chairman:

The Fiscal Year (FY) 2002 Energy and Water Development Appropriations Act, House Report 107-258, directed the Nuclear Regulatory Commission (NRC) to continue to provide a monthly report on the status of its licensing and regulatory duties. The initial reporting requirement arose in the FY 1999 Energy and Water Development Appropriations Act, Senate Report 105-206. The FY 2000 Energy and Water Development Appropriations Act, House Report 106-253, expanded the scope of the report requirement to include regulatory reform efforts affecting power reactor operations beyond 10 CFR Part 50, particularly NRC efforts to evaluate NRC security regulations. In FY 2000, we also expanded the monthly report to include the status of all license renewal applications that are under active review and other NRC initiatives in developing implementation guidance for the license renewal rule. In response to increased Congressional interest, in the May 2001 report, we began to provide information regarding the status of activities involving power uprate licensing actions. On behalf of the Commission, I am pleased to transmit the forty-fourth report, which covers the month of July (Enclosure 1).

The June report provided information on a number of significant NRC activities, including an update of our actions taken following the terrorist attacks of September 11, 2001. In particular, we discussed NRC efforts to develop a threat advisory system to implement the Office of Homeland Security (OHS) Homeland Security Advisory System (HSAS) for NRC-licensed facilities. We also provided a status report on the reactor vessel head corrosion at the Davis-Besse Nuclear Power Station in Oak Harbor, Ohio.

We would like to provide further information to you on both of these issues. In regard to the physical security and safeguards for NRC-licensed facilities, the NRC continues to monitor the current threat environment closely and to work extensively with other government agencies in developing coordinated threat assessments, and coordinating security and emergency plan responsibilities. The NRC has developed a new Threat Advisory and Protective Measures System in response to Homeland Security Presidential Directive-3. When a new Homeland Security Advisory System (HSAS) threat condition is declared, the NRC will promptly notify affected licensees of the condition and refer them to the predefined protective measures that we have developed for each threat level. The new system for NRC licensees was formally communicated to licensees, Governors, State Homeland Security Advisors, Federal agency administrators and other appropriate officials on August 19, 2002. The new system supercedes the NRC's 1998 threat advisory system and covers additional classes of licensees not included in NRC's 1998 system.

In regard to Davis-Besse, the NRC special oversight panel for the Davis-Besse Nuclear Power Plant continues to monitor the licensee activities. The panel was established to coordinate the Agency's activities in assessing the performance problems associated with the corrosion damage to the reactor vessel at the Davis-Besse Nuclear Power Plant. The plant will not restart until the NRC is satisfied that all safety concerns have been resolved. On August 9, 2002, the NRC issued Bulletin 2002-02, "Reactor Pressure Vessel Head and Vessel Head Penetration Nozzle Inspection Programs," to the licensees of the Nation's 69 operating pressurized water reactors (PWRs) advising them they may need to supplement their inspection programs for reactor vessel heads. Following the discovery of vessel head degradation at Davis-Besse, the NRC issued a bulletin on March 18, 2002, requiring PWR licensees to supply information on the structural integrity of the reactor vessel head and the basis for concluding that the vessel head will continue to perform its function as a coolant pressure boundary. NRC staff analyzed the data supplied by the PWR licensees and concluded that inspection programs that primarily rely on visual examinations may need to be supplemented with volumetric or surface examinations. The August 9, 2002 bulletin provides suggested inspection frequencies and examples of acceptable supplemental inspection techniques, including the use of ultrasound, electric currents, and liquid dyes to check for cracking and corrosion in metal structures. The bulletin is an interim measure to help manage oversight of vessel head and vessel head penetration inspection programs while additional technical issues are resolved, and suitable inspection plans are developed and implemented for the long-term. The staff has assigned resolution of these issues a high priority. In addition, the task force established by the NRC Executive Director for Operations continues its work in assessing lessons-learned related to the degradation of the reactor vessel head at the Davis-Besse Nuclear Power Plant. We will continue to keep you informed of the status of this issue.

Since our last report, the Commission and the NRC staff also:

- participated, on July 9, in a radiological dispersal device (RDD) tabletop exercise planning meeting with representatives from the Department of Energy (DOE).
- participated, on July 14-17, in the 2002 Gordon Conference on Aqueous Corrosion conducted in New London, New Hampshire. The conference addressed recent developments in the mechanistic understanding of aqueous corrosion, as well as advances in experimental techniques.
- hosted, on July 23-24, the first cooperative nuclear safety research meeting with the Ministry of Science and Technology (MOST) from the Republic of Korea for the purpose of facilitating bilateral cooperation in the area of nuclear safety research.
- published a proposed rule in the Federal Register (67 FR 51501), on August 8, that would amend the Commission's regulations concerning compliance with the Federal Advisory Committee Act. The proposed amendments would conform the NRC's regulations with newly issued regulations of the General Services Administration.
- published a proposed rule in the Federal Register (67 FR 50374), on August 2, that would amend the standards for combustible gas control in nuclear power reactors by setting new containment system baseline regulations for current licensees, as well as consolidating such regulations for future applicants and licensees. This proposal would impact current and future nuclear power licensees by relaxing or eliminating the

requirements for equipment presently used to control the concentration of combustible gases (hydrogen and oxygen) in the containment following a design-basis loss-of-coolant-accident, commensurate with its safety significance.

- requested public comments on the development of a pilot program to evaluate the possible use of alternative dispute resolution (ADR) in the NRC's enforcement program. ADR is defined as any procedure that is used to resolve issues in controversy. It can involve the use of a neutral third party to resolve conflicts and can include facilitated discussion, mediation, fact-finding, mini-trials and arbitration. The Environmental Protection Agency, the U.S. Navy and the Federal Energy Regulatory Commission are among those agencies that have used these techniques effectively.
- received, on August 1, a license renewal application for the R. E. Ginna nuclear power plant for an additional 20 years. The current license for the plant expires in September 2009. The Ginna nuclear power plant is located near Rochester, New York.
- received, on August 6, a license renewal application for the V. C. Summer nuclear power plant for an additional 20 years. The current license for the plant expires in August 2022. The Summer nuclear power plant is located near Columbia, South Carolina.
- published a proposed rule that amends 10 CFR Parts 72 and 73 to change several event notification requirements that apply to an independent spent fuel storage installation (ISFSI) and a monitored retrievable storage installation (MRS). The proposed rule also amends safeguards event notification requirements that apply to a reactor facility, a fuel cycle facility, an ISFSI, an MRS, licensees who possess or transport special nuclear material or spent fuel, a geological repository operations area, and a gaseous diffusion plant. These proposed changes are intended to align these requirements with recent changes to the power reactor event notification requirements in 10 CFR Part 50. These changes would reduce licensee burden through the consolidation of some notifications and lengthening the reporting period for other notifications. Some new requirements are also being added to permit the NRC to carry out its responsibilities during emergencies and to respond to public, media and other stakeholder inquiries during events at licensees' facilities more effectively.
- issued, on July 31, a conforming license amendment related to an Order of May 17, 2002, which approved the transfer of ownership and operating authority under the facility operating license for the Vermont Yankee nuclear power plant. Issuance of the conforming amendment completed the transfer of the license to the extent held by the Vermont Yankee Nuclear Power Corporation to Entergy Nuclear Vermont Yankee, LLC, and Entergy Nuclear Operations, Incorporated.
- issued license amendments which replace the Prairie Island, Units 1 and 2 technical specifications in their entirety with new technical specifications based on the improved Standard Technical Specifications (iSTS). Sixty-eight units have been approved to convert to the iSTS.
- conducted, on July 16 - 26, an Integrated Materials Performance Evaluation Program (IMPEP) review for the State of New York. The IMPEP team is comprised of NRC

regional staff and Agreement State personnel from the States of California and Texas. The team provided the State of New York a draft report for review in early September.

I have enclosed (Enclosure 2) the update to the Tasking Memorandum which delineates the schedules for accomplishing high priority initiatives.

Please do not hesitate to contact me if I may provide additional information.

Sincerely,

/RA/

Richard A. Meserve

Enclosures:

1. Monthly Report
2. Tasking Memorandum

cc: Senator James M. Inhofe

Identical letter to:

The Honorable Harry Reid, Chairman
Subcommittee on Transportation,
Infrastructure, and Nuclear Safety
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510
cc: Senator James M. Inhofe

The Honorable Joe Barton, Chairman
Subcommittee on Energy and Air Quality
Committee on Energy and Commerce
United States House of Representatives
Washington, D.C. 20515
cc: Representative Rick Boucher

The Honorable Sonny Callahan, Chairman
Subcommittee on Energy and Water Development
Committee on Appropriations
United States House of Representatives
Washington, D.C. 20515
cc: Representative Peter J. Visclosky

The Honorable Harry Reid, Chairman
Subcommittee on Energy and Water Development
Committee on Appropriations
United States Senate
Washington, D.C. 20510
cc: Senator Pete V. Domenici

The Honorable W.J. "Billy" Tauzin, Chairman
Committee on Energy and Commerce
United States House of Representatives
Washington, D.C. 20515
cc: Representative John D. Dingell

The Honorable James M. Jeffords, Chairman
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510
cc: Senator Bob Smith

The Honorable Pete V. Domenici
United States Senate
Washington, D.C. 20510

MONTHLY STATUS REPORT ON THE
LICENSING ACTIVITIES AND REGULATORY DUTIES OF THE
UNITED STATES NUCLEAR REGULATORY COMMISSION

JULY 2002

Enclosure 1

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¹Note: The period of performance covered by this report includes activities occurring between the first and last day of July 2002. The transmittal letter to Congress accompanying this report may provide more recent information in order to keep Congress fully and currently informed of NRC's licensing and regulatory activities.

XIII. Implementing Risk-Informed Regulations

The staff continues to make progress on tasks involving use of probabilistic risk information in many areas. The milestone schedule for significant risk-informed activities is included in the Chairman's Tasking Memorandum (Enclosure 2). Recent activities include:

- a. The staff issued an update to the Risk-Informed Regulation Implementation Plan (SECY-02-0131) on July 12, 2002, which delineates the Agency's actions to risk-inform its regulatory activities.
- b. The staff provided a proposed risk-informed change to the fire protection rule to the Commission on July 15, 2002. This document, SECY-02-0132, provides an alternative set of performance-based fire protection requirements that are risk-informed.
- c. On June 28, 2002, The Nuclear Energy Institute (NEI) submitted Revision C to NEI 00-04, "10 CFR 50.69 SSC Categorization Guideline," for staff review and endorsement. NEI revised the document to resolve the staff's comments and to incorporate lessons learned from pilot plant activities. The staff plans to endorse the document with clarifications and exceptions, if needed, in draft Regulatory Guide (RG) 1121, "Implementation of Structures, Systems and Components Categorization Process in Nuclear Power Plants."

XIV. Revised Reactor Oversight Process

The NRC continues to implement the Reactor Oversight Process (ROP) at all nuclear power plants. The NRC has continued meeting with interested stakeholders on a periodic basis to collect feedback on the efficacy of the process and considers this feedback in making refinements to the ROP. Recent activity includes:

NRC staff conducted another of a continuing series of public meetings on July 23-25, 2002, in Chicago, IL, to discuss the pilot test of the Mitigating Systems Performance Index (MSPI). The MSPI is a new risk informed performance indicator index that the NRC and the industry have jointly proposed to replace the current set of safety system unavailability performance indicators. The purpose of the MSPI is to monitor the risk impact of changes in the unavailability and unreliability of selected risk-significant systems. The key feature of this index is the use of a risk-importance measure which accounts for the existence of diverse trains and unique design features of a plant. The MSPI should more accurately represent the risk significance of system unavailability and unreliability than the current performance indicators and should result in a resource savings because MSPI findings will not need to be evaluated using the Significance Determination Process of the ROP.

The goal of the workshop was to familiarize the workshop participants with the concept of MSPI and to define the scope of the MSPI in each participating pilot plant. About 100 people, including many representatives from the industry, attended the workshop. As a result of the workshop, some issues were identified that need to be resolved and discussed in future public meetings prior to the start of the pilot test, scheduled September 1, 2002. The data collection phase of the pilot will last 6 months (September 1, 2002 through March 1, 2003). Assessment of the results of the pilot test is estimated to require another six months.

XV. Status of Issues in the Reactor Generic Issue Program

Generic Safety Issue (GSI) 168, "Environmental Qualification of Electrical Equipment"

The staff concluded that the current equipment qualification process for low voltage instrumentation and control cables is adequate for the duration of a 40-year license. However, the staff also recommended that a generic communication be considered to notify the industry of research results and to encourage the implementation of appropriate monitoring programs. After receiving concurrence from the Advisory Committee on Reactor Safeguards, the staff has begun to develop an action plan to close GSI-168.

XVI. Licensing Actions and Other Licensing Tasks

Licensing actions are defined as requests for: license amendments, exemptions from regulations, relief from inspection or surveillance requirements, topical reports submitted on a plant-specific basis, notices of enforcement discretion, or other licensee requests requiring NRC review and approval before it can be implemented by the licensee. The FY 2002 NRC Performance Plan incorporates three output measures related to licensing actions: the number of the licensing action completions per year; the age of the licensing action inventory; and the size of the licensing action inventory. In January 2002, the goal for the size of the licensing action inventory was restored to the Performance Plan and the goal for the percent of the licensing action inventory less than or equal to 1 year old was increased from 95% to 96%.

Other licensing tasks are defined as: licensee responses to NRC requests for information through generic letters or bulletins; NRC responses to 2.206 petitions; NRC review of licensee topical reports; NRR responses to regional requests for assistance; NRC review of licensee 10 CFR 50.59 analyses and FSAR updates; or other licensee requests not requiring NRC review and approval before they can be implemented by the licensee. The FY 2002 NRC Performance Plan incorporates one output measure related to other licensing tasks, which is the number of other licensing tasks completed.

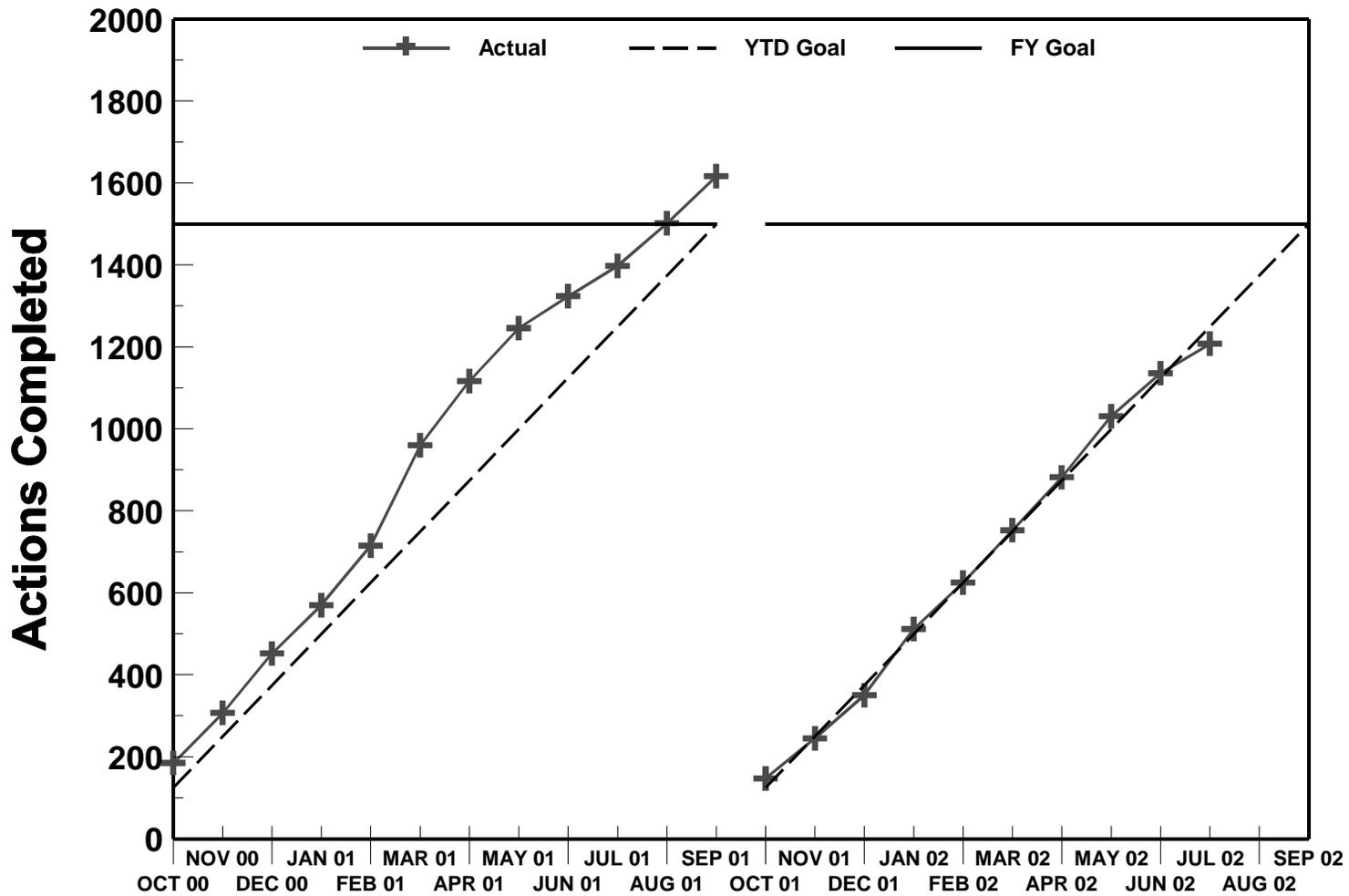
The actual FY 2000 and FY 2001 results, the FY 2002 goals and the actual FY 2002 results, as of July 31, 2002, for the four NRC Performance Plan output measures for licensing actions and other licensing tasks are shown in the following table.

PERFORMANCE PLAN				
Output Measure	FY 2000 Actual	FY 2001 Actual	FY 2002 Goals	FY 2002 Actual (thru 07/31/2002)
Licensing actions completed/year	1574	1617	≥ 1500	1208
Age of licensing action inventory	98.3% ≤ 1 year; 100% ≤ 2 years	96.9% ≤ 1 year; 100% ≤ 2 years	96% ≤ 1 year; 100% ≤ 2 years	94.5% ≤ 1 year; 100% ≤ 2 years
Size of licensing action inventory	962	877	≤ 1000	878
Other licensing tasks completed/year	1100	523	≥ 350	375

The following charts demonstrate NRC's FY 2002 trends for the four licensing action and other licensing task output measure goals.

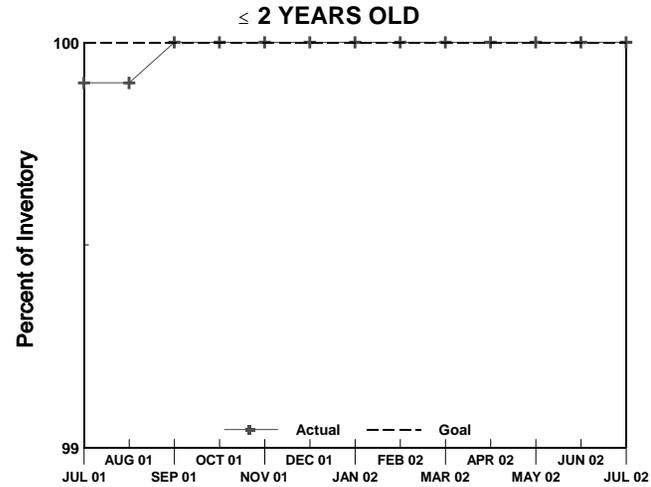
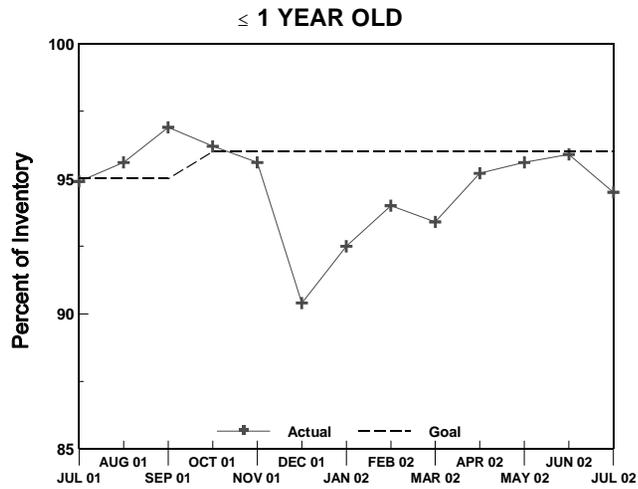
Nuclear Reactor Safety - Reactor Licensing

Performance Plan Target: Completed Licensing Actions



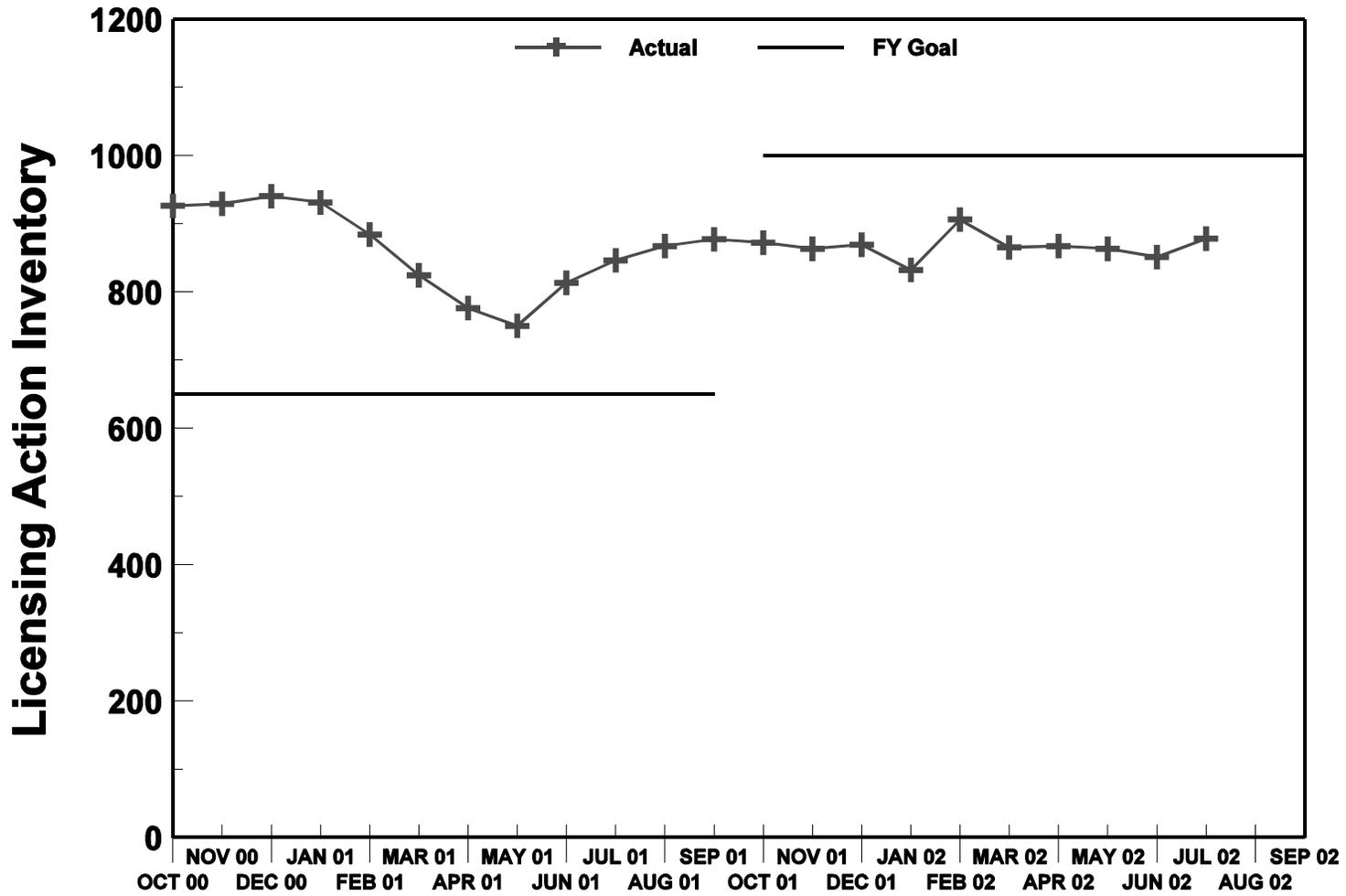
Nuclear Reactor Safety - Reactor Licensing

Performance Plan Target: Age of Licensing Action Inventory



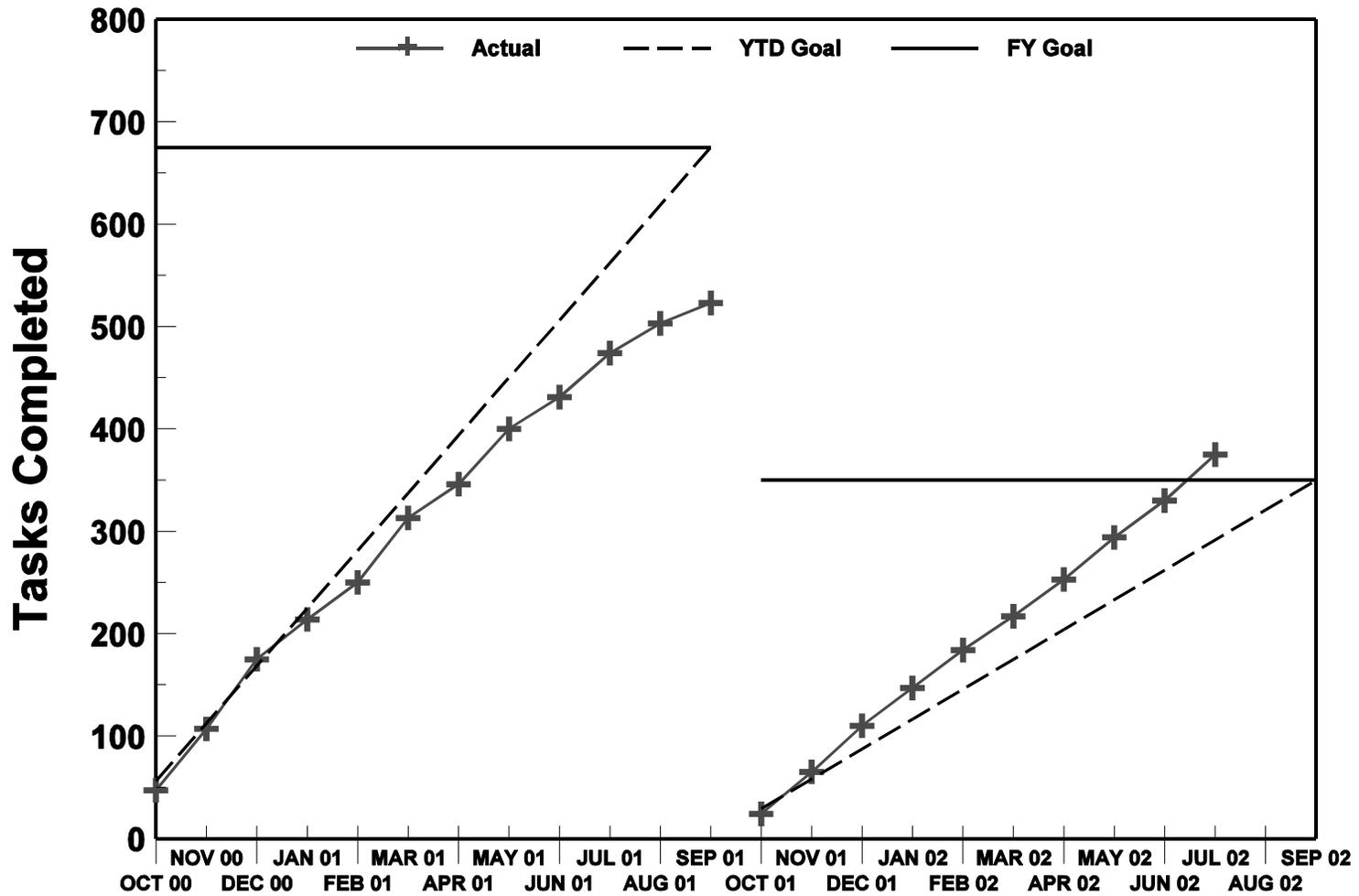
Nuclear Reactor Safety - Reactor Licensing

Performance Plan: Size of Licensing Action Inventory



Nuclear Reactor Safety - Reactor Licensing

Performance Plan Target: Completed Other Licensing Tasks



XVII. Status of License Renewal Activities

Surry, Units 1 and 2, and North Anna, Units 1 and 2, Combined Renewal Applications

The staff issued the draft supplemental environmental impact statement (SEIS) for Surry in April 2002 and North Anna in May 2002 and conducted public meetings in June 2002. The public comment period on the draft SEISs has ended and the staff is preparing the final SEISs. The safety evaluation report identifying any open items was issued in June 2002. The NRC staff and applicant are currently working to resolve the open items.

McGuire, Units 1 and 2, and Catawba, Units 1 and 2, Combined Renewal Applications

The staff issued the draft SEISs for McGuire and Catawba in May 2002. Public comments on both draft SEISs are due in August 2002. Public meetings were conducted in June 2002 to obtain comments on the draft SEISs. Responses to the safety requests for information were received and the staff is currently preparing the safety evaluation report and identifying any open items.

On January 24, 2002, the Atomic Safety and Licensing Board (ASLB) admitted contentions filed by Nuclear Information and Resource Service and the Blue Ridge Environmental Defense League, petitioners in the Catawba and McGuire license renewal proceeding. They contended that the applicant's severe accident mitigation alternative (SAMA) analysis was incomplete. The staff and Duke subsequently filed appeals to the Commission. On July 23, 2002, the Commission issued an Order (CLI 02-17), admitting to an extent, the SAMA contention. An earlier Commission Order reversed the ASLB's decision to admit a second contention pertaining to the potential use of mixed-oxide fuel in the McGuire and Catawba reactors. A certified question from the ASLB relating to the admissibility of a contention pertaining to terrorism is still under review by the Commission. In May 2002, the staff received eight late-filed contentions pertaining to the environmental review for the McGuire and Catawba license renewal application.

Peach Bottom, Units 2 and 3, Renewal Application

The staff issued the draft SEIS in June 2002 and held public meetings on the SEIS in July 2002. Public comments on the draft SEIS are due by September 2002. Responses to safety requests for additional information were received and the staff is currently preparing the safety evaluation report and identifying any open items.

St. Lucie, Units 1 and 2, Renewal Application

Environmental requests for additional information were issued in May 2002 and the responses were received in June 2002. The safety requests for additional information were issued in July 2002 and the responses are due by October 2002.

Fort Calhoun Renewal Application

The Fort Calhoun renewal application is currently under review and the staff is preparing requests for additional information. All environmental requests for additional information are

scheduled to be issued by August 2002 and the safety requests by October 2002. The environmental review and scoping process have begun and a public meeting was held in the vicinity of Fort Calhoun on June 18, 2002.

Robinson Unit 2 Renewal Application

On June 17, 2002, the NRC received an application for renewal of the Robinson Unit 2 operating license. The staff is currently performing the required acceptance review and, if found acceptable, will docket the application, notice an opportunity for hearing, and issue the review schedule.

XVIII. Status of Review of Private Fuel Storage, Limited Liability Corporation's Application for a License to Operate an Independent Spent Fuel Storage Installation on the Reservation of the Skull Valley Band of Goshute Indians

During this reporting period, the adjudicatory hearings before the Atomic Safety and Licensing Board (ASLB) were concluded at NRC headquarters in Rockville, Maryland. The hearings were adjourned on July 3, 2002. The parties to the adjudication of the geotechnical and aircraft crash contentions have eight weeks to file proposed findings of fact with the ASLB. Each of the parties then has until early October 2002 to comment on the findings submitted by the other parties. The ASLB's findings are expected to be released in December 2002. Any party can appeal the ASLB's decision to the NRC Commissioners who would sit as an appeals panel. A licensing decision by the Commissioners would not be made until after any appeals were heard and resolved.

XIX. Enforcement Process and Summary of Reactor Enforcement by Region

Reactor Enforcement by Region

Reactor Enforcement Actions*						
		Region I	Region II	Region III	Region IV**	TOTAL
Severity Level I	June 2002	0	0	0	0	0
	FY 02 YTD	0	0	0	0	0
	FY 01 Total	0	0	0	0	0
	FY 00 Total	0	0	0	0	0
Severity Level II	June 2002	1	0	0	0	1
	FY 02 YTD	1	0	0	0	1
	FY 01 Total	0	1	0	0	1
	FY 00 Total	1	2	0	0	3
Severity Level III	June 2002	0	0	0	0	0
	FY 02 YTD	2	0	0	0	2
	FY 01 Total	1	1	1	1	4
	FY 00 Total	5	0	4	4	13
Severity Level IV	June 2002	0	0	0	0	0
	FY 02 YTD	0	0	2	0	2
	FY 01 Total	1	0	2	1	4
	FY 00 Total	4	1	3	5	13
Non-Cited Severity Level IV	June 2002	21	0	3	7	0
	FY 02 YTD	175	88	148	120	531
	FY 01 Total	279	105	201	139	724
	FY 00 Total	313	190	289	258	1050

* Numbers of violations are based on enforcement action tracking system (EATS) data that maybe subject to minor changes following verification. The number of Severity Level I, II, III listed refers to the number of Severity Level I, II, III violations or problems. The monthly totals generally lag by 30 days due to inspection report and enforcement development.

** Violation totals for Regions II & IV reflect a shift from a 6 week inspection period to a quarterly inspection period.

Escalated Reactor Enforcement Actions Associated with the Reactor Oversight Process						
		Region I	Region II	Region III	Region IV	Total
NOVs Related to White, Yellow or Red Findings	6/02 Red	0	0	0	0	0
	6/02 Yellow	0	0	0	0	0
	6/02 White	1	0	1	1	3
	FY 02 YTD	2	2	3	6	13
	FY 01 Total	8	4	4	3	19
	FY 00 Total	6	1	0	0	7

Description of Significant Actions taken in June 2002

Dominion Nuclear Connecticut, Inc. (Millstone 1) EA-02-014

On June 25, 2002, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of \$288,000 was issued for a Severity Level II violation involving the failure to: (1) keep adequate records of special nuclear material (SNM); (2) establish adequate procedures for control and accounting of SNM; and (3) conduct adequate physical inventories of SNM.

FirstEnergy Nuclear Operating Company (Beaver Valley) EA-02-041

On June 24, 2002, a Notice of Violation was issued for a violation associated with a White Significance Determination Process (SDP) finding involving the public alert and notification system. The violation cited the failure to establish a means to provide early notification to the public because a majority of personal home alerting devices were degraded or removed.

Energy Northwest (Columbia Generating Station) EA-02-107

On June 24, 2002, a Notice of Violation was issue for a violation associated with a White SDP finding involving the degradation of multiple safety-related and important-to-safety breakers that were replaced during a refueling outage in June 2001. The violation cited the licensee's failure to establish adequate design control measures and the failure to assure that conditions adverse to quality be promptly identified and corrected.

Nuclear Management Company, LLC (Point Beach Nuclear Plant) EA-02-090

On June 13, 2002, a Notice of Violation was issued for a violation associated with a White SDP finding involving the self-revealing failure of safety injection system pump 2P-15B due to nitrogen gas binding. The violation cited the licensee's failure to promptly identify and correct a significant condition adverse to quality regarding leakage from the 2T-34A safety injection accumulator.

XX. Power Reactor Security Regulations

In response to the terrorist attacks on September 11, 2001, the NRC and the nuclear industry have taken a number of actions to ensure the security at nuclear power plants. Immediately following the terrorist attacks on the World Trade Center and the Pentagon, the NRC advised nuclear power plant licensees to go to the highest level of security (i.e., Level 3), and all promptly did so.

The NRC will issue a revised threat advisory system that will implement the Office of Homeland Security's (OHS) Homeland Security Advisory System (HSAS) for NRC-licensed facilities in August 2002. The NRC Threat Advisory and Protective Measures System describes specific protective measures recommended to NRC licensees that correspond with each of the five OHS color-coded threat levels in the HSAS. The revised threat advisory system will supercede the existing threat advisory system which has been in place since 1998.

In SECY-02-0104, dated June 14, 2002, the staff informed the Commission of its strategy for completing a comprehensive review of the NRC's safeguards and security program. The staff is continuing this integrated review, which includes threat definition, vulnerability assessments, and regulatory improvements.

The NRC continues to interact with the FBI, other intelligence and law enforcement agencies, the Department of Defense, and the Office of Homeland Security to ensure any changes to the NRC's programs are informed by pertinent input from all relevant U.S. agencies.

XXI. Power Uprates

The staff has assigned power uprate license amendment reviews a high priority. The staff considers power uprate applications among the most significant licensing actions and is, therefore, conducting power uprate reviews on accelerated schedules.

Licensees have been applying for and implementing power uprates since the 1970s as a way to increase the power output of their plants. The staff has been conducting power uprate reviews since then and to date, has completed 81 such reviews. Approximately 11,560 MWt (3850 MWe) or an equivalent of over three nuclear power plant units has been gained through implementation of power uprates at existing plants. During the month of June, the staff reported completing its review of one General Electric Nuclear Energy topical report for power uprates. However, during the month of July, based on discussions with General Electric Nuclear Energy, the staff identified disagreements between the staff and General Electric Nuclear Energy on how the topical report is to be applied. Therefore, the staff is withdrawing its approval of the topical report until the disagreements are resolved. No previously approved power uprate has been based on the General Electric Nuclear Energy topical report. Since the last report, the staff received two applications for measurement uncertainty recapture power uprates of approximately 1.5 percent. The staff currently has 13 plant-specific applications and one General Electric Nuclear Energy topical report for power uprate under review. The staff is also addressing the disagreements on the General Electric Nuclear Energy topical report. The staff has assigned these reviews a high priority.

The staff conducted a survey in July 2002 to obtain information regarding industry's plans related to power uprate applications. The survey requested information for planned power uprates over the next 5 years. Based on this survey and information obtained since the survey, licensees plan to submit 51 additional power uprate applications in the next 5 years. These include 27 measurement uncertainty recapture power uprates (i.e., power uprates less than 2 percent), 4 stretch power uprates (i.e., power uprates up to about 7 percent), and 20 extended power uprates (i.e., power uprates greater than about 7 percent). Planned power uprates are expected to result in an increase of over 5900 MWt (1970 MWe) (equivalent to more than one large nuclear power plant unit). Licensees also indicated that they are currently studying the feasibility of power uprates for two units. The staff will utilize this information for future planning.