

12 JUN 1987

Docket No. 50-323

Mr. J. D. Shiffer, Vice President  
Nuclear Power Generation  
c/o Nuclear Power Generation, Licensing  
Pacific Gas and Electric Company  
77 Beale Street, Room 1451  
San Francisco, California 94106

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Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 65297)

The Commission has issued the enclosed Amendment No. 12 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit No 2. The amendment consists of a change to license condition 2.C.(9) in response to your application dated March 17, 1987, as supplemented May 6, 1987.

The amendment extends the time for submittal of a steam generator tube rupture analysis to April 1988.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

**Original signed by**

Charles M. Trammell, Project Manager  
Project Directorate V  
Division of Reactor Projects - III/IV/V  
& Special Projects

Enclosures:

1. Amendment No. 12 to DPR-82
2. Safety Evaluation

cc w/enclosures:

See next page

\*See previous concurrence

\*DRSP/PD5

JLee

5/22/87

\*DRSP/PD5

CTrammell

5/22/87

\*RSB *6/8/87* \*OGC

WHodges

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Mr. J. D. Shiffer  
Pacific Gas and Electric Company

Diablo Canyon

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Diablo Canyon

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2  
DOCKET NO. 50-323  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12  
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated March 17, 1987, as supplemented May 6, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, paragraph 2.C.(9) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(9) Steam Generator Tube Rupture Analysis (SSER 31, Section 4.25)

By April 1988, PG&E shall submit for NRC review and approval an analysis which demonstrates that the steam generator tube rupture (SGTR) analysis presented in the FSAR is the most severe case with respect to the release of fission products and calculated doses. Consistent with the analytical assumptions, PG&E shall propose all necessary changes to the Technical Specifications (Appendix A) to this license.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
George W. Knighton, Director  
Project Directorate V  
Division of Reactor Projects - III/IV/V  
& Special Projects

Date of Issuance: June 12, 1987



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 12 TO FACILITY OPERATING LICENSE NO. DPR-82  
PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2  
DOCKET NO. 50-323

INTRODUCTION

By letter dated March 17, 1987, as supplemented May 6, 1987, Pacific Gas and Electric Company (PG&E or the licensee) requested an amendment to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit No. 2. The proposed amendment would allow an extension of time for the submittal of a steam generator tube rupture (SGTR) analysis. The extension would allow the analysis to be submitted by April 1988, approximately ten months later than currently required.

EVALUATION

In Facility Operating License No. DPR-82 issued on August 26, 1985, license condition 2.C.(9) stated that: "Prior to restart following the first refueling outage, PG&E shall submit for NRC review and approval an analysis which demonstrates that the steam generator tube rupture (SGTR) analysis presented in the FSAR is the most severe case with respect to the release of fission products and calculated doses. Consistent with the analytical assumptions, PG&E shall propose all necessary changes to the Technical Specifications (Appendix A) to this license."

The proposed amendment would extend the time for the submittal from approximately June 15, 1987 (the approximate date for startup from the first refueling outage) to April 1988, an extension of about ten months.

As a result of a SGTR event at Ginna, the staff raised questions about the assumptions used in the SGTR analysis presented in the Diablo Canyon and other Westinghouse-designed FSARs, including the capability of plant operators to terminate the event (terminate break flow) in 30 minutes, and whether systems and components credited to mitigate the accident consequences are safety grade. Accordingly, all operating licenses issued for Westinghouse-designed reactors in the 1985 time frame and later contained a condition similar to the one in the Diablo Canyon Unit 2 license. When the license was issued on August 26, 1985, it was felt that the time for submittal of the analysis would be adequate since the startup from the first refueling would be about June 1987, a period of almost two years.

In response to the staff concerns expressed above, a subgroup of utilities in the Westinghouse Owners Group (WOG), of which PG&E is a member, was formed to address the licensing issues associated with the SGTR event on a generic basis. The subgroup submitted: (1) WCAP-10698, "SGTR Analysis Methodology to Determine Margin to Steam Generator Overfill" in December 1984; (2) Supplement 1 to WCAP-10698, "Evaluation of Offsite Radiation Doses for SGTR Accident" in May 1985; and (3) WCAP-11002, "Evaluation of Steam Generator Overfill due to SGTR Accident" in February 1986.

The NRC staff has now issued its safety evaluation report (SER) on this matter (March 30, 1987) and the SER has been distributed to all the members of the subgroup (April 1987). The SER on the generic analysis was issued beyond the time possible for PG&E and others to submit a plant-specific analysis based on the generic reports.

PG&E plans to utilize the results of the Subgroup program for its plant-specific analysis and estimates that one year would be needed to do so.

PG&E further states that the Subgroup's generic evaluation shows that (1) operators can respond to a design-basis SGTR and perform required actions to locate and terminate primary-to-secondary leakage before steam generator overfill occurs, and (2) offsite radiation doses for a design-basis SGTR will be less than the allowable limits.

The staff concludes that there is reasonable assurance that the SGTR analysis can be delayed as proposed without endangering the health and safety of the public because the probability of a design-basis SGTR during the requested period (10 months beyond the first fuel cycle) is low and in the event of the design-basis SGTR, the offsite consequences can be expected to be within 10 CFR 100 guidelines. This extension of time is similar to that granted other facilities, e.g. Catawba Unit 1, Amendment No. 12, September 18, 1986.

Based on the foregoing, the NRC staff concludes that the proposed extension of time for submittal of a plant-specific SGTR analysis is reasonable, does not adversely affect plant safety and should be granted.

#### CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Chief of the Radiological Health Branch, State Department of Health Services, State of California, of the proposed determination of no significant hazards consideration. No comments were received.



### ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Trammell

Dated: June 12, 1987