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October 17, 1988

DO NOT REMOVE

Docket Nos.: STN 50-528. STN 50-529 and STN 50-530

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Mr. Donald B. Karner **Executive Vice President** Arizona Nuclear Power Project Post Office Box 52034 Phoenix, Arizona 85072-2034

DHagan Region V(4) AKM/LFMB

Dear Mr. Karner:

ISSUANCE OF AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-41, AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-51, SUBJECT: AND AMENDMENT NO. 14 TO FACILITY OPERATING LICENSE NO. NPF-74, FOR THE PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3, RESPECTIVELY (TAC NOS. 68394, 68395 AND 68396)

The Commission has issued the subject Amendments, which are enclosed, to the Facility Operating Licenses for Palo Verde Nuclear Generating Station, Units 1, 2, and 3. The Amendments consist of a change to the Technical Specifications in response to your application dated May 27, 1988.

The amendments revise Technical Specifications 3.2.3 and 4.2.3.2 and adds Figure 3.2-1A to modify the azimuthal power tilt TS to require the measured power tilt to be equal to or less than the Core Protection Calculator allowance and the limit in Figure 3.2-1A when the Core Operating Limit Supervisory System is in service. In addition, the azimuthal power tilt limit is increased for Unit 2 only.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely.

original signed by

DE THE SH ICC. C. A.

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Michael J. Davis, Project Manager Project Directorate V Division of Reactor Projects - III, IV, V and Special Projects

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10/17/88

Enclosures: 1. Amendment No. 38 to NPF-41 2. Amendment No. 25 to NPF-51 3. Amendment No. 14 to NPF-74

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

October 17, 1988

Docket Nos.: STN 50-528, STN 50-529 and STN 50-530

Mr. Donald B. Karner Executive Vice President Arizona Nuclear Power Project Post Office Box 52034 Phoenix, Arizona 85072-2034

Dear Mr. Karner:

SUBJECT: ISSUANCE OF AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-41, AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-51, AND AMENDMENT NO. 14 TO FACILITY OPERATING LICENSE NO. NPF-74, FOR THE PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND-3, RESPECTIVELY (TAC NOS. 68394, 68395 AND 68396)

The Commission has issued the subject Amendments, which are enclosed, to the Facility Operating Licenses for Palo Verde Nuclear Generating Station, Units 1, 2, and 3. The Amendments consist of a change to the Technical Specifications in response to your application dated May 27, 1988.

The amendments revise Technical Specifications 3.2.3 and 4.2.3.2 and adds Figure 3.2-1A to modify the azimuthal power tilt TS to require the measured power tilt to be equal to or less than the Core Protection Calculator allowance and the limit in Figure 3.2-1A when the Core Operating Limit Supervisory System is in service. The wording of TS 4.2.3.2 was revised for clarity. In addition, the azimuthal power tilt limit is increased for Unit 2 only.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

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Michael J. Daviś, Project Manager Project Directorate V Division of Reactor Projects - III, IV. V and Special Projects

En	closures:				
1.	Amendment	No.	38	to	NPF-41
2.	Amendment	No.	25	to	NPF-51
2	Amondmont	No	14	to	NPF_74

cc: See next page



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Mr. Donald B. Karner Arizona Nuclear Power Project Executive Vice President Post Office Box 52034 Phoenix, Arizona 85072-2034

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Mr. Charles Tedford, Director Arizona Radiation Regulatory Agency 4814 South 40 Street Phoenix, Arizona 85040

Chairman Maricopa County Board of Supervisors 111 South Third Avenue Phoenix, Arizona 85003



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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-528

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38 License No. NPF-41

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated May 27, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-41 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 38, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

FOR THE NUCLEAR REGULATORY COMMISSION

George W Knighton, Director Project Directorate V Division of Reactor Projects - III, IV, V and Special Projects

Enclosure: Changes to the Technical Specifications

Date of Issuance: October 17, 1988

- 2 -

ENCLOSURE TO LICENSE AMENDMENT

TO FACILITY OPERATING LICENSE NO. NPF-41 AMENDMENT NO. 38

DOCKET NO. STN 50-528

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page	<u>Overleaf Page</u>
xix	XX
 3/4 2-3	
3/4 2-4	
3/4 2-4a	
B3/4 2-2	B3/4 2-1

INDEX



4

~

LIST OF FIGURES

				PAGE	1
3.1-1A	SHUTDOWN MARGIN VERSUS COLD LEG	TEMPERATURE		3/4	1-2a
3.1-1	ALLOWABLE MTC MODES 1 AND 2		• • • • • •	3/4	1-5
3.1-2	MINIMUM BORATED WATER VOLUMES			3/4.	1-12
3.1-2A	CORE POWER LIMIT AFTER CEA DEVIA	TION		3/4	1-24
3.1-3	CEA INSERTION LIMITS VS THERMAL (COLSS IN SERVICE)	POWER		3/4	1-31
3.1-4	CEA INSERTION LIMITS VS THERMAL (COLSS OUT OF SERVICE)	POWER		3/4	1-32
3.1-5	PART LENGTH CEA INSERTION LIMIT	VS THERMAL POWER.		3/4	1-34
3.2-1A	AZIMUTHAL POWER TILT LIMIT VS TH (COLSS IN SERVICE)	IERMAL POWER		3/4	2-4a
3.2-1	COLSS DNBR POWER OPERATING LIMIT CEACS INOPERABLE	ALLOWANCE FOR BO	TH	3/4	2-6
3.2-2	DNBR MARGIN OPERATING LIMIT BASE CALCULATORS (COLSS OUT OF SERVIC	D ON CORE PROTECT E, CEACs OPERABLE	ION)	3/4	2-7
3.2-2A	DNBR MARGIN OPERATING LIMIT BASE CALCULATORS (COLSS OUT OF SERVIC	D ON CORE PROTECT E, CEACs INOPERAB	ION LE)	3/4	2-7a
3 .2-3	REACTOR COOLANT COLD LEG TEMPERA	TURE VS CORE POWE	R 	3/4	2-10
3.4-1	DOSE EQUIVALENT I-131 PRIMARY CO ACTIVITY LIMIT VERSUS PERCENT OF POWER WITH THE PRIMARY COOLANT S > 1.0 µCi/GRAM DOSE EQUIVALENT I	OLANT SPECIFIC RATED THERMAL SPECIFIC ACTIVITY -131		3/4	4-27
3.4-2	REACTOR COOLANT SYSTEM PRESSURE FOR 0 TO 10 YEARS OF FULL POWER	TEMPERATURE LIMIT OPERATION	ATIONS	3/4	4-29
4.7-1	SAMPLING PLAN FOR SNUBBER FUNCTI	IONAL TEST	• •.• • • •	3/4	7-26
B 3/4.4-1	NIL-DUCTILITY TRANSITION TEMPERA FUNCTION OF FAST (E > 1 MeV) NEU (550°F IRRADIATION)	ATURE INCREASE AS	A	B 3/	′4 4-10
5 .1 - 1	SITE AND EXCLUSION BOUNDARIES		• • • • • •	5-2	
5.1-2	LOW POPULATION ZONE		• • • • • •	5-3	
5.1-3	GASEOUS RELEASE POINTS			5-4	
6.2-1	OFFSITE ORGANIZATION		• • • • • •	6-3	
6.2-2	ONSITE UNIT ORGANIZATION		• • • • • •	6-4	
PALO VERDE	- UNIT 1 XIX		AMENDMENT	NO.	38



INDEX

LIST OF TABLES

		PAGE
1.1	FREQUENCY NOTATION	1-8
1.2	OPERATIONAL MODES	1-9
2.2-1	REACTOR PROTECTIVE INSTRUMENTATION TRIP SETPOINT LIMITS	2-3
	REQUIRED MONITORING FREQUENCIES FOR BACKUP BORON DILUTION DETECTION AS A FUNCTION OF OPERATING CHARGING PUMPS AND PLANT OPERATIONAL MODES	
3.1-1	FOR K === > 0.98	3/4 1-16
3.1-2	FOR 0.98 > K_{ref} > 0.97	3/4 1-17
3.1-3	FOR 0.97 $\geq K_{aff} > 0.96$	3/4 1-18
3.1-4	FOR 0.96 \geq K < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 < 0.95 $< 0.$	- 3/4 1-19
3.1-5	FOR $K_{aff} \leq 0.95$	3/4 1-20
3.3-1	REACTOR PROTECTIVE INSTRUMENTATION	3/4 3-3
3.3-2	REACTOR PROTECTIVE INSTRUMENTATION RESPONSE TIMES	3/4_3-11
4.3-1	REACTOR PROTECTIVE INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-14
3.3-3	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION	3/4 3-18
3.3-4	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES	3/4 3-25
3.3-5	ENGINEERED SAFETY FEATURES RESPONSE TIMES	3/4 3-28
4.3-2	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-31
3.3-6	RADIATION MONITORING INSTRUMENTATION	3/4 3-38
4.3-3	RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-40
3.3 - 7	SEISMIC MONITORING INSTRUMENTATION	3/4 3-43
4.3-4	SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-44
3.3-8	METEOROLOGICAL MONITORING INSTRUMENTATION	3/4 3-46
4.3-5	METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-47
3.3-9A	REMOTE SHUTDOWN INSTRUMENTATION	3/4 3-49
3.3-9 8	REMOTE SHUTDOWN DISCONNECT SWITCHES	3/4 3-50

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3/4.2.3 AZIMUTHAL POWER TILT - Tg

LIMITING CONDITION FOR OPERATION

- 3.2.3 The AZIMUTHAL POWER TILT (T_q) shall be less than or equal to the following limits:
 - a. The AZIMUTHAL POWER TILT Allowance used in the Core Protection Calculators (CPCs), and
 - b.1. The limit in Figure 3.2-1A with COLSS in service, or

b.2. $T_{\alpha} \leq 0.10$ with COLSS out of service.

APPLICABILITY: MODE 1 above 20% of RATED THERMAL POWER*.

ACTION:

- a. With the measured AZIMUTHAL POWER TILT determined to exceed the AZIMUTHAL POWER TILT Allowance used in the CPCs, within 2 hours either correct the power tilt or adjust the AZIMUTHAL POWER TILT Allowance used in the CPCs to greater than or equal to the measured value.
- b. With the measured AZIMUTHAL POWER TILT determined to exceed the limit in Figure 3.2-1A with COLSS in service or 0.10 with COLSS out of service:
 - 1. Due to misalignment of either a part-length or full-length CEA, within 30 minutes verify that the Core Operating Limit Supervisory System (COLSS) (when COLSS is being used to monitor the core power distribution per Specifications 4.2.1 and 4.2.4) is detecting the CEA misalignment.
 - 2. Verify that the AZIMUTHAL POWER TILT is within its limit within 2 hours after exceeding the limit or reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within the next 2 hours and verify that the Variable Overpower Trip Setpoint has been reduced as appropriate within the next 4 hours.
 - 3. Identify and correct the cause of the out of limit condition prior to increasing THERMAL POWER; subsequent POWER OPERATION above 50% of RATED THERMAL POWER may proceed provided that the AZIMUTHAL POWER TILT is verified within its limit at least once per hour for 12 hours or until verified acceptable at 95% or greater RATED THERMAL POWER.

*See Special Test Exception 3.10.2.

PALO VERDE - UNIT 1

SURVEILLANCE REQUIREMENTS

4.2.3.1 The provisions of Specification 4.0.4 are not applicable.

4.2.3.2 The AZIMUTHAL POWER TILT shall be determined to be within its limits above 20% of RATED THERMAL POWER by:

- a. Continuously monitoring the tilt with COLSS when the COLSS is in service.
- b. Calculating the tilt at least once per 12 hours when the COLSS is out of service.
- c. Verifying at least once per 31 days, that the COLSS Azimuthal Tilt Alarm is actuated at an AZIMUTHAL POWER TILT less than or equal to the AZIMUTHAL POWER TILT Allowance used in the CPCs.
- d. Using the incore detectors at least once per 31 EFPD to independently confirm the validity of the COLSS calculated AZIMUTHAL POWER TILT.



FIGURE 3.2 - 1A AZIMUTHAL POWER TILT LIMIT VS THERMAL POWER (COLSS IN SERVICE)

PALO VERDE - UNIT 1

AMENDMENT NO. 38

BASES

3/4.2.1 LINEAR HEAT RATE

The limitation on linear heat rate ensures that in the event of a LOCA, the peak temperature of the fuel cladding will not exceed 2200°F.

Either of the two core power distribution monitoring systems, the Core Operating Limit Supervisory System (COLSS) and the Local Power Density channels in the Core Protection Calculators (CPCs), provide adequate monitoring of the core power distribution and are capable of verifying that the linear heat rate does not exceed its limits. The COLSS performs this function by continuously monitoring the core power distribution and calculating a core power operating limit corresponding to the allowable peak linear heat rate. Reactor operation at or below this calculated power level assures that the limits of 13.5 kW/ft are not exceeded.

The COLSS calculated core power and the COLSS calculated core power operating limits based on linear heat rate are continuously monitored and displayed to the operator. A COLSS alarm is annunciated in the event that the core power exceeds the core power operating limit. This provides adequate margin to the linear heat rate operating limit for normal steady-state operation. Normal reactor power transients or equipment failures which do not require a reactor trip may result in this core power operating limit being exceeded. In the event this occurs, COLSS alarms will be annunciated. If the event which causes the COLSS limit to be exceeded results in conditions which approach the core safety limits, a reactor trip will be initiated by the Reactor Protective Instrumentation. The COLSS calculation of the linear heat rate includes appropriate penalty factors which provide, with a 95/95 probability/ confidence level, that the maximum linear heat rate calculated by COLSS is conservative with respect to the actual maximum linear heat rate existing in the core. These penalty factors are determined from the uncertainties associated with planar radial peaking measurement, engineering heat flux uncertainty, axial densification, software algorithm modelling, computer processing, rod bow, and core power measurement.

Parameters required to maintain the operating limit power level based on linear heat rate, margin to DNB, and total core power are also monitored by the CPCs. Therefore, in the event that the COLSS is not being used, operation within the linear heat rate limit can be maintained by utilizing any operable CPC channel. The above listed uncertainty and penalty factors plus those associated with the CPC startup test acceptance criteria are also included in the CPCs.



BASES

3/4.2.2 PLANAR RADIAL PEAKING FACTORS

Limiting the values of the PLANAR RADIAL PEAKING FACTORS (F_{XY}^{C}) used in the COLSS and CPCs to values equal to or greater than the measured PLANAR RADIAL PEAKING FACTORS (F_{XY}^{m}) provides assurance that the limits calculated by COLSS and the CPCs remain valid. Data from the incore detectors are used for determining the measured PLANAR RADIAL PEAKING FACTORS. A minimum core power at 20% of RATED THERMAL POWER is assumed in determining the PLANAR RADIAL PEAKING FACTORS. The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. The periodic surveillance requirements for determining the measured PLANAR RADIAL PEAKING FACTORS used in COLSS and the CPCs remain valid throughout the fuel cycle. Determining the measured PLANAR RADIAL PEAKING FACTORS after each fuel loading prior to exceeding 70% of RATED THERMAL POWER provides additional assurance that the core was properly loaded.

3/4.2.3 AZIMUTHAL POWER TILT - Tq

The limitations on the AZIMUTHAL POWER TILT are provided to ensure that design safety margins are maintained. An AZIMUTHAL POWER TILT greater than the limit in Figure 3.2-1A with COLSS in service or 0.10 with COLSS out of service is not expected and if it should occur, operation is restricted to only those conditions required to identify the cause of the tilt. The tilt is normally calculated by COLSS. A minimum core power of 20% of RATED THERMAL POWER is assumed by the CPCs in its input to COLSS for calculation of AZIMUTHAL POWER TILT. The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. The surveillance requirements specified when COLSS is out of service provide an acceptable means of detecting the presence of a steady-state tilt. It is necessary to explicitly account for power asymmetries because the radial peaking factors used in the core power distribution calculations are based on an untilted power distribution.

The AZIMUTHAL POWER TILT is equal to (P_{tilt}/P_{untilt}) -1.0 where:

AZIMUTHAE POWER TILT is measured by assuming that the ratio of the power at any core location in the presence of a tilt to the untilted power at the location is of the form:

$$P_{tilt}/P_{untilt} = 1 + T_{a} g \cos (\theta - \theta_{0})$$

where:

 T_{α} is the peak fractional tilt amplitude at the core periphery

g is the radial normalizing factor

 θ is the azimuthal core location

 θ_0 is the azimuthal core location of maximum tilt

PALO VERDE - UNIT 1

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AMENDMENT NO. 38



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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-529

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25 License No. NPF-51

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment, dated May 27, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-51 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 25, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

FOR THE NUCLEAR REGULATORY COMMISSION

LA

George W Knighton, Director Project Directorate V Division of Reactor Projects - III, IV. V and Special Projects

Enclosure: Changes to the Technical Specifications

Date of Issuance: October 17, 1988

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-51

DOCKET NO. STN 50-529

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page	<u>Overleaf</u> Page
xix	xx
3/4 2-3	
3/4 2-4	
3/4 2-4a	
B3/4 2-2	B3/4 2-1

INDEX

		PAGE
3.1-1A	SHUTDOWN MARGIN VERSUS COLD LEG TEMPERATURE	3/4 1-2a
3.1 - 1	ALLOWABLE MTC MODES 1 AND 2	3/4 1-5
3.1-2	MINIMUM BORATED WATER VOLUMES	3/4 1-12
3.1-2A	CORE POWER LIMIT AFTER CEA DEVIATION	3/4 1-24
3.1-3	CEA INSERTION LIMITS VS THERMAL POWER (COLSS IN SERVICE)	3/4 1-31
3.1-4	CEA INSERTION LIMITS VS THERMAL POWER (COLSS OUT OF SERVICE)	3/4 1-32
3.1-5	PART LENGTH CEA INSERTION LIMIT VS THERMAL POWER	3/4 1-34
3.2-1A	AZIMUTHAL POWER TILT LIMIT VS THERMAL POWER (COLSS IN SERVICE)	3/4 2-4a
3.2-1	COLSS <u>DNBR POWER OPERATING LIMIT ALLOWANCE FOR BOTH</u> CEACS INOPERABLE	3/4 2-6
3.2-2	DNBR MARGIN OPERATING LIMIT BASED ON CORE PROTECTION CALCULATORS (COLSS OUT OF SERVICE, CEACs OPERABLE)	3/4 2-7
3.2-2A	DNBR MARGIN OPERATING LIMIT BASED ON CORE PROTECTION CALCULATORS (COLSS OUT OF SERVICE, CEACs INOPERABLE)	3/4 2-7a
3.2-3	REACTOR COOLANT COLD LEG TEMPERATURE VS CORE POWER LEVEL	3/4 2-10
3.4-1	DOSE EQUIVALENT I-131 PRIMARY COOLANT SPECIFIC ACTIVITY LIMIT VERSUS PERCENT OF RATED THERMAL POWER WITH THE PRIMARY COOLANT SPECIFIC ACTIVITY > 1.0 µCi/GRAM DOSE EQUIVALENT I-131	3/4 4-27
3.4-2	REACTOR COOLANT SYSTEM PRESSURE TEMPERATURE LIMITATIONS FOR 0 TO 10 YEARS OF FULL POWER OPERATION	3/4 4-29
4.7-1	SAMPLING PLAN FOR SNUBBER FUNCTIONAL TEST	3/4 7-26
B 3/4.4-1	NIL-DUCTILITY TRANSITION TEMPERATURE INCREASE AS A FUNCTION OF FAST (E > 1 MeV) NEUTRON FLUENCE	R 3/4 4-1
F 1 1	(550°F IKKADIATION)	5-2
J. 1-1	DIE AND EACLUSION DUGNDARIES	5-3
5 1-3	GASFOUS RELEASE POINTS	5-4
J. 7 J		
6.2-1	OFFSITE ORGANIZATION	6-3
6.2-2	ONSITE UNIT ORGANIZATION	6-4

PALO VERDE - UNIT 2

٠.

7

AMENDMENT NO. 25

I	N	D	E	X
	_	_	_	

1

LIST OF TABLES

	· /	PAGE
1.1	FREQUENCY NOTATION	1-8
1.2	OPERATIONAL MODES	1-9
2. 2-1	REACTOR PROTECTIVE INSTRUMENTATION TRIP SETPOINT LIMITS	2-3
	REQUIRED MONITORING FREQUENCIES FOR BACKUP BORON DILUTION DETECTION AS A FUNCTION OF OPERATING CHARGING PUMPS AND PLANT OPERATIONAL MODES	
3.1-1	FOR K _{eff} > 0.98	3/4 1-16
3.1-2	FOR $0.98 \ge K_{eff} > 0.97$	3/4 1-17
3.1-3	FOR 0.97 > K + 0.96	3/4 1-18
3.1-4	FOR 0.96 > K > 0.95	3/4 1-19
3.1-5	FOR K 0.95	3/4 1-20
3.3-1	REACTOR PROTECTIVE INSTRUMENTATION	3/4 3-3
3.3-2	REACTOR PROTECTIVE INSTRUMENTATION RESPONSE TIMES	3/4 3-11
4. 3-1	REACTOR PROTECTIVE INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-14
3.3-3	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION	3/4 3-18
3.3-4	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES	3/4 3-25
3. 3-5	ENGINEERED SAFETY FEATURES RESPONSE TIMES	3/4 3-28
4.3-2	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-31
3. 3-6	RADIATION MONITORING INSTRUMENTATION.	3/4 3-38
4.3-3	RADIATION MONITORING INSTRUMENTATION SURVEILLANCE	2/4 2-40
3 3-7		3/4 3-40
A 2-A	SETSHE NONTROTING INSTRUMENTATION CUDVET(LANCE	J/4 J-43
T.J T	REQUIREMENTS	3/4 3-44
3.3-8	METEOROLOGICAL MONITORING INSTRUMENTATION	3/4 3-46
4.3-5	METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-47
3. 3- 9A	REMOTE SHUTDOWN INSTRUMENTATION	3/4 3-49
3. 3- 98	REMOTE SHUTDOWN DISCONNECT SWITCHES	3/4 3-50

XX

3/4.2.3 AZIMUTHAL POWER TILT - T

LIMITING CONDITION FOR OPERATION

3.2.3 The AZIMUTHAL POWER TILT (T_q) shall be less than or equal to the following limits:

a. The AZIMUTHAL POWER TILT Allowance used in the Core Protection Calculators (CPCs), and

b.1. The limit in Figure 3.2-1A with COLSS in service, or

b.2. $T_{q} \leq$ 0.10 with COLSS out of service.

APPLICABILITY: MODE 1 above 20% of RATED THERMAL POWER*.

ACTION: ____

- a. With the measured AZIMUTHAL POWER TILT determined to exceed the AZIMUTHAL POWER TILT Allowance used in the CPCs within 2 hours either correct the power tilt or adjust the AZIMUTHAL POWER TILT Allowance used in the CPCs to greater than or equal to the measured value.
- b. With the measured AZIMUTHAL POWER TILT determined to exceed the limit in Figure 3.2-1A with COLSS in service or 0.10 with COLSS out of service:
 - 1. Due to misalignment of either a part-length or full-length CEA, within 30 minutes verify that the Core Operating Limit Supervisory System (COLSS) (when COLSS is being used to monitor the core power distribution per Specifications 4.2.1 and 4.2.4) is detecting the CEA misalignment.
 - 2. Verify that the AZIMUTHAL POWER TILT is within its limit within 2 hours after exceeding the limit or reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within the next 2 hours and verify that the Variable Overpower Trip Setpoint has been reduced as appropriate within the next 4 hours.
 - 3. Identify and correct the cause of the out of limit condition prior to increasing THERMAL POWER; subsequent POWER OPERATION above 50% of RATED THERMAL POWER may proceed provided that the AZIMUTHAL POWER TILT is verified within its limit at least once per hour for 12 hours or until verified acceptable at 95% or greater RATED THERMAL POWER.



^{*}See Special Test Exception 3.10.2.

PALO VERDE - UNIT 2

SURVEILLANCE REQUIREMENTS

4.2.3.1 The provisions of Specification 4.0.4 are not applicable.

4.2.3.2 The AZIMUTHAL POWER TILT shall be determined to be within its limits above 20% of RATED THERMAL POWER by:

- a. Continuously monitoring the tilt with COLSS when the COLSS is in service.
- b. Calculating the tilt at least once per 12 hours when the COLSS is out of service.
- c. Verifying at least once per 31 days, that the COLSS Azimuthal Tilt Alarm is actuated at an AZIMUTHAL POWER TILT less than or equal to the AZIMUTHAL POWER TILT Allowance used in the CPCs.
- d. Using the incore detectors at least once per 31 EFPD to independently confirm the validity of the COLSS calculated AZIMUTHAL POWER TILT.

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٧S THERMAL POWER (COLSS IN SERVICE)

AMENDMENT NO. 25

BASES

3/4.2.1 LINEAR HEAT RATE

The limitation on linear heat rate ensures that in the event of a LOCA, the peak temperature of the fuel cladding will not exceed 2200°F.

Either of the two core power distribution monitoring systems, the Core Operating Limit Supervisory System (COLSS) and the Local Power Density channels in the Core Protection Calculators (CPCs), provide adequate monitoring of the core power distribution and are capable of verifying that the linear heat rate does not exceed its limits. The COLSS performs this function by continuously monitoring the core power distribution and calculating a core power operating limit corresponding to the allowable peak linear heat rate. Reactor operation at or below this calculated power level assures that the limits of 13.5 kW/ft are not exceeded.

The COLSS calculated core power and the COLSS calculated core power operating limits based on linear heat rate are continuously monitored and displayed to the operator. A COLSS alarm is annunciated in the event that the core power exceeds the core power operating limit. This provides adequate margin to the linear heat rate operating limit for normal steady-state operation. Normal reactor power transients or equipment failures which do not require a reactor trip may result in this core power operating limit being exceeded. In the event this occurs, COLSS alarms will be annunciated. If the event which causes the COLSS limit to be exceeded results in conditions which approach the core safety limits, a reactor trip will be initiated by the Reactor Protective Instrumentation. The COLSS calculation of the linear heat rate includes appropriate penalty factors which provide, with a 95/95 probability/ confidence level, that the maximum linear heat rate calculated by COLSS is conservative with respect to the actual maximum linear heat rate existing in the core. These penalty factors are determined from the uncertainties associated with planar radial peaking measurement, engineering heat flux uncertainty, axial densification, software algorithm modelling, computer processing, rod bow, and core power measurement.

Parameters required to maintain the operating limit power level based on linear heat rate, margin to DNB, and total core power are also monitored by the CPCs. Therefore, in the event that the COLSS is not being used, operation within the linear-heat rate limit can be maintained by utilizing any operable CPC channel. The above listed uncertainty and penalty factors plus those associated with the CPC startup test acceptance criteria are also included in the CPCs.

PALO VERDE - UNIT 2

BASES

3/4.2.2 PLANAR RADIAL PEAKING FACTORS

S74.2.2 PLANAR RADIAL PLANAR RADIAL PLANAR RADIAL PEAKING FACTORS (F_{XY}^{C}) used in the COLSS and CPCs to values equal to or greater than the measured PLANAR RADIAL PEAKING FACTORS (F_{XY}^{m}) provides assurance that the limits calculated by COLSS and the CPCs remain valid. Data from the incore detectors are used for determining the measured PLANAR RADIAL PEAKING FACTORS. A minimum core power at 20% of RATED THERMAL POWER is assumed in determining the PLANAR RADIAL PEAKING FACTORS. The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. The periodic surveillance requirements for determining the measured PLANAR RADIAL PEAKING FACTORS provides assurance that the PLANAR RADIAL PEAKING FACTORS used in COLSS and the CPCs remain valid throughout the fuel cycle. Determining the measured PLANAR RADIAL PEAKING FACTORS after each fuel loading prior to exceeding 70% of RATED THERMAL POWER provides additional assurance that the core was properly loaded.

3/4.2.3 AZIMUTHAL POWER TILT - T

The limitations on the AZIMUTHAL POWER TILT are provided to ensure that design safety margins are maintained. An AZIMUTHAL POWER TILT greater than the limit in Figure 3.2-1A with COLSS in service or 0.10 with COLSS out of service is not expected and if it should occur, operation is restricted to only those conditions required to identify the cause of the tilt. The tilt is normally calculated by COLSS. A minimum core power of 20% of RATED THERMAL POWER is assumed by the CPCs in its input to COLSS for calculation of AZIMUTHAL POWER TILT. The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. The surveillance requirements specified when COLSS is out of service provide an acceptable means of detecting the presence of a steady-state tilt. It is necessary to explicitly account for power asymmetries because the radial peaking factors used in the core power distribution calculations are based on an untilted power distribution.

The AZIMUTHAL POWER TILT is equal to (Ptilt/Puntilt)-1.0 where:

AZIMUTHAL POWER TILT is measured by assuming that the ratio of the power at any core location in the presence of a tilt to the untilted power at the location is of the form:

$$P_{\pm i \pm}/P_{\mu \sigma \pm i \pm} = 1 + T_{\sigma} g \cos (\theta - \theta_0)$$

where:

T is the peak fractional tilt amplitude at the core periphery q is the radial normalizing factor

g 15 one realer normer englisher.

θ is the azimuthal core location

 Θ_0 is the azimuthal core location of maximum tilt

PALO VERDE - UNIT 2



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14 License No. NPF-74

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment, dated May 27, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 14, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director Project Directorate V Division of Reactor Projects - III, IV, V and Special Projects

Enclosure: Changes to the Technical Specifications

Date of Issuance: October 17, 1988

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 14 TO FACILITY OPERATING LICENSE NO. NPF-74

DOCKET NO. STN 50-530

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page	<u>Cverleaf Page</u>
xix	XX
3/4 2-3	•••
3/4 2-4	
3/4 2-4a	
B3/4 2-2	B3/4 2-1



INDEX

LIST OF FIGURES

;

۰,

		PAGE
3.1-1A	SHUTDOWN MARGIN VERSUS COLD LEG TEMPERATURE	3/4 1-2a
3.1-1	ALLOWABLE MTC MODES 1 AND 2	3/4 1-5
3.1-2	MINIMUM BORATED WATER VOLUMES	3/4 1-12
3.1-2A	PART LENGTH CEA INSERTION LIMIT VS THERMAL POWER	3/4-1-23
3.1-2B	CORE POWER LIMIT AFTER CEA DEVIATION	3/4 1-24
3.1-3	CEA INSERTION LIMITS VS THERMAL POWER (COLSS IN SERVICE)	3/4 1-31
3.1-4	CEA INSERTION LIMITS VS THERMAL POWER (COLSS OUT OF SERVICE)	3/4 1-32
3.2-1A	AZIMUTHAL POWER TILT LIMIT VS. THERMAL POWER (COLSS IN SERVICE)	3/4 2-4a
3.2-1	DNBR MARGIN OPERATING LIMIT BASED ON COLSS _(COLSS_IN_SERVICE)	3/4 2-6
3.2-2	DNBR MARGIN OPERATING LIMIT BASED ON CORE PROTECTION CALCULATOR (COLSS OUT OF SERVICE)	3/4 2-7
3.2-3	REACTOR COOLANT COLD LEG TEMPERATURE VS CORE POWER LEVEL	3/4 2-10
3.3-1	DNBR MARGIN OPERATING LIMIT BASED ON COLSS FOR BOTH CEAC'S INOPERABLE	3/4 3-10
3.4-1	DOSE EQUIVALENT I-131 PRIMARY COOLANT SPECIFIC ACTIVITY LIMIT VERSUS PERCENT OF RATED THERMAL POWER WITH THE PRIMARY COOLANT SPECIFIC ACTIVITY > 1.0 µCi/GRAM DOSE EQUIVALENT I-131	3/4 4-27
3.4-2	REACTOR COOLANT SYSTEM PRESSURE TEMPERATURE LIMITATIONS FOR 0 TO 10 YEARS OF FULL POWER OPERATION	3/4 4-29
4.7-1	SAMPLING PLAN FOR SNUBBER FUNCTIONAL TEST	3/4 7-26
B 3/4.4-1	NIL-DUCTILITY TRANSITION TEMPERATURE INCREASE AS A . FUNCTION OF FAST (E > 1 MeV) NEUTRON FLUENCE (550°F IRRADIATION)	B 3/4 4-10
5.1-1	SITE AND EXCLUSION BOUNDARIES	5-2
5.1-2	LOW POPULATION ZONE	5-3
5.1-3	GASEOUS RELEASE POINTS	5-4
6. 2 - 1	OFFSITE ORGANIZATION	6-3
6.2-2	ONSITE UNIT ORGANIZATION	6-4

PALO VERDE - UNIT 3

XIX

AMENDMENT NO. 14

INDEX	ļ

LI	ST.	OF	TAB	LES
	· · · · · · · ·			

į

		PAGE	
1.1	FREQUENCY NOTATION	1-8	
1.2	OPERATIONAL MODES	1-9	
2.2-1	REACTOR PROTECTIVE INSTRUMENTATION TRIP SETPOINT LIMITS	2-3	
	REQUIRED MONITORING FREQUENCIES FOR BACKUP BORON DILUTION DETECTION AS A FUNCTION OF OPERATING CHARGING PUMPS AND PLANT OPERATIONAL MODES		
3.1-1	FOR K_** > 0.98	3/4 1-16	
3.1-2	FOR 0.98 > K > 0.97	3/4 1-17	
3.1-3	FOR 0.97 > K > 0.96	3/4 1-18	
3.1-4	FOR 0.96 > K	3/4 1-19	
3.1-5	FOR K 0.95	3/4 1-20	
3.3-1	REACTOR PROTECTIVE INSTRUMENTATION	3/4 3-3	
3.3-2	REACTOR PROTECTIVE INSTRUMENTATION RESPONSE TIMES	3/4 3-11	
3.3-2a	INCREASES IN BERRO, BERR2, AND BERR4 VERSUS RTD DELAY TIMES	3/4 3-13	
4.3-1	REACTOR PROTECTIVE INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-14	and the second
3.3-3	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION	3/4 3-18	
3.3-4	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES	3/4 3-25	
3.3-5	ENGINEERED SAFETY FEATURES RESPONSE TIMES	3/4 3-28	
4.3-2	ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-31	
3.3-6	RADIATION MONITORING INSTRUMENTATION	3/4 3-38	
4.3-3	RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-40	
3.3-7	SEISMIC MONITORING INSTRUMENTATION	3/4 3-43	
4.3-4	SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-44	
3.3-8	METEOROLOGICAL MONITORING INSTRUMENTATION	3/4 3-46	
4.3-5	METEOROLOGICAL MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	3/4 3-47	
3.3-9A	REMOTE SHUTDOWN INSTRUMENTATION	3/4 3-49	(1)
3.3-9B	REMOTE SHUTDOWN DISCONNECT SWITCHES	3/4 3-50	N.S.

6

3/4.2.3 AZIMUTHAL POWER TILT - T

LIMITING CONDITION FOR OPERATION

3.2.3 The AZIMUTHAL POWER TILT (T_q) shall be less than or equal to the following limits:

a. The AZIMUTHAL POWER TILT Allowance used in the Core Protection Calculators (CPCs), and

b.1. The limit in Figure 3.2-1A with COLSS in service, or

b.2. $T_a \leq$ 0.10 with COLSS out of service.

APPLICABILITY: MODE 1 above 20% of RATED THERMAL POWER*.

ACTION:

- a. With the measured AZIMUTHAL POWER TILT determined to exceed the AZIMUTHAL POWER TILT Allowance used in the CPCs within 2 hours either correct the power tilt or adjust the AZIMUTHAL POWER TILT Allowance used in the CPCs to greater than or equal to the measured value.
- b. With the measured AZIMUTHAL POWER TILT determined to exceed the limit in Figure 3.2-1A with COLSS in service or 0.10 with COLSS out of service.
 - Due to misalignment of either a part-length or full-length CEA, within 30 minutes verify that the Core Operating Limit Supervisory System (COLSS) (when COLSS is being used to monitor the core power distribution per Specifications 4.2.1 and 4.2.4) is detecting the CEA misalignment.
 - 2. Verify that the AZIMUTHAL POWER TILT is within its limit within 2 hours after exceeding the limit or reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within the next 2 hours and verify that the Variable Overpower Trip Setpoint has been reduced as appropriate within the next 4 hours.
 - 3. Identify and correct the cause of the out of limit condition prior to increasing THERMAL POWER; subsequent POWER OPERATION above 50% of RATED THERMAL POWER may proceed provided that the AZIMUTHAL POWER TILT is verified within its limit at least once per hour for 12 hours or until verified acceptable at 95% or greater RATED THERMAL POWER.



*See Special Test Exception 3.10.2.

PALO VERDE - UNIT 3

AMENDMENT NO. 14

SURVEILLANCE REQUIREMENTS

4.2.3.1 The provisions of Specification 4.0.4 are not applicable.

4.2.3.2 The AZIMUTHAL POWER TILT shall be determined to be within its limits above 20% of RATED THERMAL POWER by:

- a. Continuously monitoring the tilt with COLSS when the COLSS is in service.
- b. Calculating the tilt at least once per 12 hours when the COLSS is out of service.
- c. Verifying at least once per 31 days, that the COLSS Azimuthal Tilt Alarm is actuated at an AZIMUTHAL POWER TILT less than or equal to the AZIMUTHAL POWER TILT Allowance used in the CPCs.
- d. Using the incore detectors at least once per 31 EFPD to independently confirm the validity of the COLSS calculated AZIMUTHAL POWER TILT.



PERCENT OF RATED THERMAL POWER

FIGURE 3.2-1A AZIMUTHAL POWER TILT LIMIT VS THERMAL POWER (COLSS IN SERVICE)

BASES

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3/4.2.1 LINEAR HEAT RATE

The limitation on linear heat rate ensures that in the event of a LOCA, the peak temperature of the fuel cladding will not exceed 2200°F.

Either of the two core power distribution monitoring systems, the Core Operating Limit Supervisory System (COLSS) and the Local Power Density channels in the Core Protection Calculators (CPCs), provide adequate monitoring of the core power distribution and are capable of verifying that the linear heat rate does not exceed its limits. The COLSS performs this function by continuously monitoring the core power distribution and calculating a core power operating limit corresponding to the allowable peak linear heat rate. Reactor operation at or below this calculated power level assures that the limits of 14.0 kW/ft are not exceeded.

The COLSS calculated core power and the COLSS calculated core power operating limits based on linear heat rate are continuously monitored and displayed to the operator. A COLSS alarm is annunciated in the event that the core power exceeds the core power operating limit. This provides adequate margin to the linear heat rate operating limit for normal steady-state operation. Normal reactor power transients or equipment failures which do not require a reactor trip may result in this core power operating limit being exceeded. In the event this occurs, COLSS alarms will be annunciated. If the event which causes the COLSS limit to be exceeded results in conditions which approach the core safety limits, a reactor trip will be initiated by the Reactor Protective Instrumentation. The COLSS calculation of the linear heat rate includes appropriate penalty factors which provide, with a 95/95 probability/ confidence level, that the maximum linear heat rate calculated by COLSS is conservative with respect to the actual maximum linear heat rate existing in the core. These penalty factors are determined from the uncertainties associated with planar radial peaking measurement, engineering heat flux uncertainty, axial densification, software algorithm modelling, computer processing, rod bow, and core power measurement.

Parameters required to maintain the operating limit power level based on linear heat rate, margin to DNB, and total core power are also monitored by the CPCs (assuming minimum core power of 20% of RATED THERMAL POWER). The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. Therefore, in the event that the COLSS is not being used, operation within the limits of Figure 3.2-2 can be maintained by utilizing a predetermined local power density margin and a total core power limit in the CPC trip channels. The above listed uncertainty and penalty factors plus those associated with the CPC startup test acceptance criteria are also included in the CPCs.

BASES

3/4.2.2 PLANAR RADIAL PEAKING FACTORS

Limiting the values of the PLANAR RADIAL PEAKING FACTORS (F_{xy}^{C}) used in the COLSS and CPCs to values equal to or greater than the measured PLANAR RADIAL PEAKING FACTORS (F_{xy}^{m}) provides assurance that the limits calculated by COLSS and the CPCs remain valid. Data from the incore detectors are used for determining the measured PLANAR RADIAL PEAKING FACTORS. A minimum core power at 20% of RATED THERMAL POWER is assumed in determining the PLANAR RADIAL PEAKING FACTORS. The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. The periodic surveillance requirements for determining the measured PLANAR RADIAL PEAKING FACTORS provides assurance that the PLANAR RADIAL PEAKING FACTORS used in COLSS and the CPCs remain valid throughout the fuel cycle. Determining the measured PLANAR RADIAL PEAKING FACTORS after each fuel loading prior to exceeding 70% of RATED THERMAL POWER provides additional assurance that the core was properly loaded.

3/4.2.3 AZIMUTHAL POWER TILT - T

The limitations on the AZIMUTHAL POWER TILT are provided to ensure that design safety margins are maintained. An AZIMUTHAL POWER TILT greater than the limit in Figure 3.2-1A with COLSS in service or 0.10 with COLSS out of service is not expected and if it should occur, operation is restricted to only those conditions required to identify the cause of the tilt. The tilt is normally calculated by COLSS. A minimum core power of 20% of RATED THERMAL POWER is assumed by the CPCs in its input to COLSS for calculation of AZIMUTHAL POWER TILT. The 20% RATED THERMAL POWER threshold is due to the neutron flux detector system being inaccurate below 20% core power. Core noise level at low power is too large to obtain usable detector readings. The surveillance requirements specified when COLSS is out of service provide an acceptable means of detecting the presence of a steady-state tilt. It is necessary to explicitly account for power asymmetries because the radial peaking factors used in the core power distribution calculations are based on an untilted power distribution.

The AZIMUTHAL POWER TILT is equal to (P_{tilt}/P_{untilt})-1.0 where:

AZIMUTHAL POWER TILT is measured by assuming that the ratio of the power at any core location in the presence of a tilt to the untilted power at the location is of the form:

 $P_{tilt}/P_{untilt} = 1 + T_q g \cos (\theta - \theta_0)$

where:

 T_q is the peak fractional tilt amplitude at the core periphery

g is the radial normalizing factor

0 is the azimuthal core location

 Θ_0 is the azimuthal core location of maximum tilt

PALO VERDE - UNIT 3



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-41,

AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-51

AND AMENDMENT NO. 14 TO FACILITY OPERATING LICENSE NO. NPF-74

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

PALO VERDE NUCLEAR GENERATING STATION, UNIT NOS. 1, 2 AND 3

DOCKET NOS. STN 50-528, STN 50-529 AND STN 50-530

1.0 INTRODUCTION

By letter dated May 27, 1988, the Arizona Public Service Company (APS) on behalf of itself, the Salt River Project Agricultural Improvement and Power District, Southern California Edison Company, El Paso Electric Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), requested a change to the Technical Specifications for the Palo Verde Nuclear Generating Station, Units 1, 2 and 3 (Appendix A to Facility Operating License Nos. NPF-41, NPF-51 and NPF-74, respectively).

The proposed change modifies the azimuthal power tilt TS to require the measured azimuthal power tilt to be equal to or less than the CPC (Core Protection Calculator) allowance and the limit in Fig 3.2-1A when the Core Operating Limit Supervisory System (COLSS) is in service. The words "in service" and "out of service" have also been used to replace the terms "operable" and "inoperable" for the COLSS. Also an increase in the azimuthal power tilt limit was proposed for Unit 2 only.

2.0 EVALUATION

The proposed changes were in response to questions raised during review of the PVNGS Unit 2 Cycle 2 reload submittal and the NRC enhanced operational inspection of PVNGS Unit 3 conducted on December 10-18, 1987. The concern was whether the CPC azimuthal power tilt allowance could become non-conservative when the measured azimuthal power tilt exceeds the TS limit. The proposed changes eliminate this possibility by maintaining needed safety margins and thus make the TS more conservative.

The wording change regarding the status of the COLSS adds clarity to the TS and is acceptable. Increasing Unit 2's azimuthal power tilt limits for COLSS in service for operation below 40% power will allow the operators to better mitigate the consequence of xenon transients occurring below 40%

- 2 -

power. The supporting analyses have been performed for Unit 2 only. These analyses are cycle and unit specific and will be performed during the next reload analysis for Units 1 and 3. The Unit 2 Cycle 2 analyses performed physics calculations for all reactivity insertion events for which the azimuthal power tilt is an explicit input. These analyses include Control Element Assembly (CEA) Ejection, Single Full Length CEA Withdrawal, and Single Part Length CEA drop events. The Unit 2 Cycle 2 analysis assuming the higher tilt values showed sufficient margin for the most limiting Design Bases Events. At greater than 40% power, the proposed limits are identical to the present values.

The staff has reviewed the material submitted by the licensee and, on the basis of the above evaluation, concludes that the proposed changes to Technical Specifications are acceptable.

3.0 CONTACT WITH STATE OFFICIAL

The Arizona Radiation Regulatory Agency was advised of the proposed determination of no significant hazards consideration with regard to this change. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATIONS

The amendments involve changes in the use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public. We therefore, conclude that the proposed changes are acceptable.



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Dated: October 17, 1988