

July 29, 1988

Docket No. STN 50-530

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Mr. Donald B. Karner
Executive Vice President
Arizona Nuclear Power Project
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Dear Mr. Karner:

SUBJECT: ISSUANCE OF AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. NPF-74 FOR THE PALO VERDE NUCLEAR GENERATING STATION, UNIT 3 (TAC NO. 67456)

The Commission has issued the subject Amendment, which is enclosed, to the Facility Operating License for Palo Verde Nuclear Generating Station, Unit 3. The Amendment consists of a change to the Technical Specifications (Appendix A to the license) in response to your application transmitted by letter dated March 1, 1988.

The Amendment revises the Technical Specifications for Palo Verde Unit 3, as follows. Surveillance Requirement 4.7.9.b has been revised to postpone the first inservice visual inspection for all inaccessible snubbers in Unit 3 until the first refueling outage.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

George Knighton for

Michael J. Davis, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures

1. Amendment No. 10 to NPF-74
2. Safety Evaluation

cc: See next page

*See previous concurrence

*DRSP/PDV
MJDavis:dr
7/5/88

*DRSP/PDV
JLee
7/11/88

*OGC
7/21/88

DRSP/PDV
GWKnighton
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 29, 1988

Docket No. STN 50-530

Mr. Donald B. Karner
Executive Vice President
Arizona Nuclear Power Project
Post Office Box 52034
Phoenix, Arizona 85072-2034

Dear Mr. Karner:

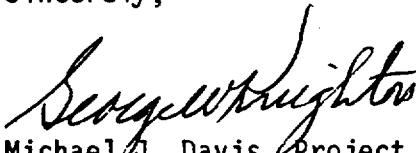
SUBJECT: ISSUANCE OF AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO.
NPF-74 FOR THE PALO VERDE NUCLEAR GENERATING STATION, UNIT 3
(TAC NO. 67456)

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Michael J. Davis, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures

1. Amendment No. 10 to NPF-74
2. Safety Evaluation

cc: See next page

Mr. Donald B. Karner
Arizona Nuclear Power Project
executive Vice President
Post Office Box 52034
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Palo Verde

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Arizona Nuclear Power Project - 2 - Palo Verde

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Mr. Charles Tedford, Director
Arizona Radiation Regulatory Agency
4814 South 40 Street
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Chairman
Maricopa County Board of Supervisors
111 South Third Avenue
Phoenix, Arizona 85003



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ARIZONA PUBLIC SERVICE COMPANY, ET AL.

DOCKET NO. STN 50-530

PALO VERDE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. NPF-74

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment, dated March 1, 1988, by the Arizona Public Service Company (APS) on behalf of itself and the Salt River Project Agricultural Improvement and Power District, El Paso Electric Company, Southern California Edison Company, Public Service Company of New Mexico, Los Angeles Department of Water and Power, and Southern California Public Power Authority (licensees), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-74 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 10, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. APS shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: July 29, 1988

ENCLOSURE TO LICENSE AMENDMENT

AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. NPF-74

DOCKET NO. STN 50-530

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Also to be replaced is the following overleaf page to the amended page.

Amendment Page

3/4 7-21

Overleaf Page

3/4 7-22

PLANT SYSTEMS

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All hydraulic and mechanical snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3, and 4. MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES.

ACTION:

With one or more snubbers inoperable on any system, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9g. on the attached component or declare the attached system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Snubber Types

As used in this specification, type of snubber shall mean snubbers of the same design and manufacturer, irrespective of capacity.

b. Visual Inspections

Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these groups (inaccessible and accessible) may be inspected independently according to the schedule below. The first inservice visual inspection of each type of snubber shall be performed after 4 months but within 10 months of commencing POWER OPERATION and shall include all hydraulic and mechanical snubbers.* If all snubbers of each type are found OPERABLE during the first inservice visual inspection, the second inservice visual inspection of that type shall be performed at the first refueling outage. Otherwise, subsequent visual inspections of a given type shall be performed in accordance with the following schedule:

*With the exception that the first inservice visual inspection for all snubbers that are inaccessible during reactor operation shall be conducted no later than the first refueling outage.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

<u>No. of Inoperable Snubbers of Each Type per Inspection Period</u>	<u>Subsequent Visual Inspection Period*#</u>
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
8 or more	31 days ± 25%

c. Visual Inspection Acceptance Criteria

Visual inspections shall verify that: (1) there are no visible indications of damage or impaired OPERABILITY and (2) attachments to the foundation or supporting structure are secure, and (3) fasteners for attachment of the snubber to the component and to the snubber anchorage are secure. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, provided that: (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers irrespective of type on that system that may be generically susceptible; and (2) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specifications 4.7.9f. When a fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be declared inoperable and cannot be determined OPERABLE via functional testing unless the test is started with the piston in the as-found setting, extending the piston rod in the tension mode direction. Snubbers which appear inoperable during an area post maintenance inspection, area walkdown, or Transient Event Inspection shall not be considered inoperable for the purpose of establishing the Subsequent Visual Inspection Period provided that the cause of the inoperability is clearly established and remedied for that particular snubber and for the other snubbers, irrespective of type, that may be generally susceptible.

d. Transient Event Inspection

An inspection shall be performed of all hydraulic and mechanical snubbers attached to sections of systems that have experienced unexpected, potentially damaging transients as determined from a review of operational data. A visual inspection of the systems shall be made within 6 months following such an event. In addition

*The inspection interval for each type of snubber on a given system shall not be lengthened more than one step at a time unless a generic problem has been identified and corrected; in that event the inspection interval may be lengthened one step the first time and two steps thereafter if no inoperable snubbers of that type are found on that system.

#The provisions of Specification 4.0.2 are not applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. NPF-74
ARIZONA PUBLIC SERVICE COMPANY, ET AL
PALO VERDE NUCLEAR GENERATING STATION, UNIT 3
DOCKET NO. STN 50-530

1.0 INTRODUCTION

By letter dated March 1, 1988, Arizona Nuclear Power Project (ANPP) requested an amendment to change the Palo Verde Nuclear Generating Station (PVNGS) Unit 3 Technical Specifications. The proposed change revises Technical Specification Surveillance Requirement 4.7.9.b to postpone the first inservice visual inspection for all the inaccessible snubbers in Unit 3 until the first refueling outage. The remainder of the snubbers (those accessible during power operations) will be visually inspected in accordance with the current Technical Specification requirements.

2.0 DISCUSSION

Presently, the Technical Specifications require the first inservice visual inspection to be conducted after 4 months but within 10 months after commencing power operations. The Technical Specifications also require categorization of the snubbers as either accessible or inaccessible during reactor operations. The accessibility determination is based upon factors such as existing radiation levels, duration of time required to perform an inspection at the snubber location, temperature, atmosphere, and the recommendations of Regulatory Guides 8.8 and 8.10. There are a total of 720 mechanical snubbers and 12 hydraulic snubbers in Unit 3. Of the 720 mechanical snubbers, 455 are inaccessible during reactor operations and are located inside containment. They are more or less evenly distributed among the chemical and volume control, reactor coolant, main steam and safety injection systems. The 12 hydraulic snubbers, which are also inaccessible, are all located in the reactor coolant system.

Although the snubbers are categorized as inaccessible during reactor operations, many of the snubbers can be visually inspected when the plant is in Mode 3. ANPP states it will perform visual inspections on as many of the inaccessible snubbers as possible during any unplanned reactor shutdown until such time as the reactor is ready to be returned to service. However, not all of the inaccessible snubbers can be inspected in this manner as scaffolding must be installed to inspect the containment dome snubbers.

3.0 EVALUATION

The inaccessible snubbers (mechanical and hydraulic) that are the subject of this amendment request function to ensure the structural integrity of the RCS and other safety related systems. Proper operation of the snubbers during accident situations helps to ensure the structural integrity of the safety related systems which ensures that the consequences of previously evaluated accidents are not increased. Proper operation of these inaccessible snubbers without surveillance is a reasonable expectation due to the following considerations. There is a relatively short time involved with this amendment request. The extension is estimated as approximately 6 months, since the required 10 month surveillance interval will expire near the end of September 1988 and shutdown for the first refueling outage will occur at the beginning of April 1989.

Previous experience with snubber inservice visual inspections on PVGNS Units 1 and 2 indicates that no inoperable snubbers have been found during the performance of the visual inspections. Two inservice visual inspections have been conducted on Unit 2 which is significant when applied to Unit 3 since Unit 3 has not experienced the number of transient events and heatup/cooldown cycles that Units 1 and 2 have. Since Unit 2 has had no failures during adverse circumstances, it is unlikely Unit 3 would have failures in a more benign environment.

A portion of the inaccessible Unit 3 snubbers (those located in the dome region of the containment building) are part of the containment spray system. This system has not been operated with water, subjected to transients, or subjected to the thermal and mechanical stresses associated with system operation. Therefore, it is unlikely that any failures have occurred. The other inaccessible snubbers similarly are not expected to have experienced failures, based on the inspection results on Units 1 and 2 and the more favorable operating conditions previously discussed for Unit 3.

The basis for Technical Specification 4.7.9 concerning regular inspection of snubbers is to ensure the structural integrity of the RCS and other safety related systems during and following a seismic event or another event initiating dynamic loads. There is adequate assurance that the inaccessible snubbers will perform as required to ensure the structural integrity of the RCS and other safety related systems due to experience with snubbers at Units 1 and 2 described in the preceding paragraph. Therefore, an extension of approximately six months for surveillance of inaccessible snubbers at Unit 3 is acceptable.

The change proposed by the licensee has been reviewed by the staff and found to be acceptable because postponement of inspection of inaccessible snubbers at Unit 3 will eliminate unnecessary man-rem exposure without an unreasonable delay in the overall surveillance program.

4.0 CONTACT WITH STATE OFFICIAL

The Arizona Radiation Regulatory Agency was advised of the proposed determination of no significant hazards consideration with regard to this change. No comments were received.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change to a surveillance requirement with respect to the installation or use of the facility components located within the restricted areas as defined in 10 CFR 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendment involves no significant hazards consideration, and there has been no public comment on such findings. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The staff has concluded, based on the consideration discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the proposed change is acceptable.

Principal Contributor: Jai Rajan

Dated: July 10, 1988