

February 6, 1992

Docket Nos. 50-275
and 50-323

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Mr. Gregory M. Rueger
Senior Vice President
and General Manager
Nuclear Power Generation Business Unit
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, California 94106

Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER PLANT,
UNIT NO. 1 (TAC NO. M81710) AND UNIT NO. 2 (TAC NO. M81711)

The Commission has issued the enclosed Amendment No. 68 to Facility Operating License No. DPR-80 and Amendment No. 67 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated August 7, 1991, as supplemented by letter dated November 22, 1991.

These amendments revise Section 6, Administrative Controls, of the Diablo Canyon TS to change: (1) the name of the offsite review committee from General Office Nuclear Plant Review and Audit Committee (GONPRAC) to Nuclear Safety Oversight Committee (NSOC), (2) the description of the NSOC membership, and (3) eliminate the use of alternates on the committee.

A copy of the related Safety Evaluation is enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by
Harry Rood

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

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Enclosures:

1. Amendment No. 68 to DPR-80
2. Amendment No. 67 to DPR-82
3. Safety Evaluation

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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Senior Vice President
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Sincerely,

A handwritten signature in cursive script that reads "Harry Rood".

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 68 to DPR-80
2. Amendment No. 67 to DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. Gregory M. Rueger
Pacific Gas and Electric Company

Diablo Canyon

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated August 7, 1991, as supplemented by letter dated November 22, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 68, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1992



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated August 7, 1991, as supplemented by letter dated November 22, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 67, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1992

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 67 TO FACILITY OPERATING ICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages are also included, as appropriate.

REMOVE

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ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

6.5.2.6 The Plant Staff Review Committee shall be responsible for:

- a. Review of: (1) Administrative procedures, Security Plan implementing procedures, Emergency Plan implementing procedures, and changes thereto; (2) the safety evaluations for: (a) changes to procedures and (b) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question and (3) proposed procedures or changes thereto that have been initially determined to constitute an unreviewed safety question or require a change to the Technical Specifications;
- b. Review of all proposed changes to Appendix "A" Technical Specifications;
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
- d. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Senior Vice President and General Manager, Nuclear Power Generation and to the Chairman of the Nuclear Safety Oversight Committee (NSOC); the investigation shall include an assessment of the safety significance of each violation.
- e. Review of all REPORTABLE EVENTS;
- f. Review of plant operations to detect potential nuclear safety hazards;
- g. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of NSOC;
- h. Review of the Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the NSOC or the Plant Manager, as appropriate;
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the NSOC or the Plant Manager, as appropriate;
- j. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence to the Senior Vice President and General Manager, Nuclear Power Generation and to NSOC; and
- k. Review of changes to the PROCESS CONTROL PROGRAM, ODCP, ERMP, and the Radwaste Treatment Systems.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

6.5.2.7 The Plant Staff Review Committee shall:

- a. Recommend to the Plant Manager written approval or disapproval of items considered under Specification 6.5.2.6a. through d. above;
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.2.6a. through e. above constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Senior Vice President and General Manager, Nuclear Power Generation and NSOC of disagreement between the PSRC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.2.8 The Plant Staff Review Committee shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Senior Vice President and General Manager, Nuclear Power Generation and NSOC.

6.5.3 NUCLEAR SAFETY OVERSIGHT COMMITTEE (NSOC)

FUNCTION

6.5.3.1 The NSOC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

NSOC shall report to and advise the Senior Vice President and General Manager, Nuclear Power Generation on those areas of responsibility specified in Specifications 6.5.3.7 and 6.5.3.8.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.3.2 NSOC shall be composed of a chairman and a minimum of four members. The NSOC Chairman and members shall be appointed in writing by the Senior Vice President and General Manager, Nuclear Power Generation. The NSOC Chairman shall have a minimum of six years of professional level managerial experience in the power field and the NSOC members shall have a minimum of five years of professional level experience in the field of their specialty. The NSOC Chairman and all members shall have qualifications that meet or exceed the requirements and recommendations of Section 4.7 of ANSI/ANS 3.1-1978.

ALTERNATES

6.5.3.3 This section deleted.

CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the NSOC Chairman to provide expert advice to NSOC.

MEETING FREQUENCY

6.5.3.5 NSOC shall meet at least once per 6 months.

QUORUM

6.5.3.6 A quorum of NSOC necessary for the performance of the NSOC functions of these Technical Specifications shall be a majority (one-half or more) of the members, but no less than four. No more than a minority of the quorum shall have line responsibility for operation of the plant.

REVIEW

6.5.3.7 NSOC shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- g. ALL REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components that could affect nuclear safety; and
- i. Reports and meetings minutes of the Plant Staff Review Committee and the Onsite Safety Review Group.

AUDITS

6.5.3.8 Audits of plant activities shall be performed under the cognizance of NSOC. These audits shall encompass:

- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training and qualifications of the entire plant staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in plant equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection program and implementing procedures at least once per 24 months by qualified personnel;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. Any other area of plant operation considered appropriate by NSOC or the Senior Vice President and General Manager, Nuclear Power Generation;
- h. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- i. The GDCP and ERMP and implementing procedures at least once per 24 months;
- j. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months; and
- k. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

RECORDS

6.5.3.9 Records of NSOC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NSOC meeting shall be prepared, approved and forwarded to the Senior Vice President and General Manager, Nuclear Power Generation within 14 working days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.3.7 above, shall be prepared, approved and forwarded to the Senior Vice President and General Manager, Nuclear Power Generation within 14 working days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.3.8 above, shall be forwarded to the Senior Vice President and General Manager, Nuclear Power Generation and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6. The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.73; and

ADMINISTRATIVE CONTROLS

REPORTABLE EVENT ACTION (Continued)

- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and the results of this review submitted to NSOC and the Senior Vice President and General Manager, Nuclear Power Generation.

6.7 SAFETY LIMIT VIOLATION

6.7 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Senior Vice President and General Manager, Nuclear Power Generation and NSOC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems or structures, and (3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, NSOC and the Senior Vice President and General Manager, Nuclear Power Generation within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;
- e. PROCESS CONTROL PROGRAM implementation;
- f. ODCP and ERMP implementation; and
- g. Quality Assurance Program for effluent and environmental monitoring.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, and all proposed tests and experiments that affect nuclear safety shall be approved prior to implementation by the Plant Manager or by a technically qualified manager who reports directly to the Plant Manager, as previously designated by the Plant Manager. Prior to approval, the Plant Manager or designated manager shall ensure that all necessary reviews and cross-disciplinary reviews, if appropriate, have been completed. If deemed necessary, such cross-disciplinary reviews shall be performed by the appropriate designated plant review personnel. Administrative procedures, procedures implementing the Security Plan, Emergency Plan, and Process Control Program, the ODCP and ERMP, and changes thereto, shall be reviewed by the PSRC and approved by the Plant Manager prior to implementation. Each procedure of Specification 6.8.1 above shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and
- c. The change is documented, reviewed and approved by the Plant Manager, or by a technically qualified manager who reports directly to the Plant Manager as previously designated by the Plant Manager, within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include portions of the Recirculation Spray System, Safety Injection System, Chemical and Volume Control System, Residual Heat Removal System, RCS Sample System, and Liquid and Gaseous Radwaste Treatment Systems. The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

ADMINISTRATIVE CONTROLS

3. WCAP-8385, Power Distribution Control and Load Following Procedures, September 1974 (Westinghouse Proprietary).
- c. The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as SHUTDOWN MARGIN, and transient and accident analysis limits) of the safety analysis are met.
- d. The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided, upon issuance for each reload cycle, to the NRC Document Control Desk, with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the NRC in accordance with 10 CFR 50.4 within the time period specified for each report.

6.10 RECORD RETENTION

In Addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

- 6.10.1 The following records shall be retained for at least 5 years:
 - a. Records and logs of unit operation covering time interval at each power level;
 - b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety;
 - c. ALL REPORTABLE EVENTS;

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications;
- e. Records of changes made to procedures required by Specification 6.8.1;
- f. Records of radioactive shipments;
- g. Records of sealed source and fission detector leak tests and results; and
- h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report;
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories;
- c. Records of radiation exposure for all individuals entering radiation control areas;
- d. Records of gaseous and liquid radioactive material released to the environs;
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1;
- f. Records of reactor tests and experiments;
- g. Records of training and qualification for current members of the unit staff;
- h. Records of in-service inspections performed pursuant to these Technical Specifications;
- i. Records of Quality Assurance activities required by the QA Manual;
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- k. Records of meetings of the PSRC and NSOC;



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated August 7, 1991, as supplemented by letter dated November 22, 1991, Pacific Gas and Electric Company (or the licensee) submitted a request for changes to the Technical Specifications (TS). The proposed amendments would revise the combined TS for the Diablo Canyon Power Plant Unit Nos. 1 and 2 to change Section 6.0 regarding the offsite review committee. The amendment includes changing the committee name from General Office Nuclear Plant Review and Audit Committee (GONPRAC) to Nuclear Safety Oversight Committee (NSOC), changing the committee composition, and eliminating the use of alternates on the committee.

2.0 EVALUATION

The NRC staff has reviewed the TS changes proposed by the licensee and finds them acceptable based on the following evaluation:

- (a) TS 6.5, "Review and Audit," 6.6, "Reportable Event Action," 6.7, "Safety Limit Violation," and 6.10, "Record Retention," - The licensee requested that these TS be revised to change the name of the General Office Nuclear Plant Review and Audit Committee" (GONPRAC) to "Nuclear Safety Oversight Committee" (NSOC).

The NRC staff has review these changes and found them acceptable because the changes are editorial.

- (b) TS 6.5.3.2, "Composition" - The licensee requested that the composition of GONPRAC/NSOC be redefined by deleting the specific titles of members and implementing the following requirements: (1) the committee chairman and members shall be appointed in writing by the Senior Vice President and General Manager, Nuclear Power Generation, (2) the chairman will have a minimum of six years of professional level managerial experience in the power generation field, (3) the committee members shall have a minimum of five years of professional level experience in the field of their specialty, and (4) the chairman and committee shall have qualifications that meet or exceed the requirements and recommendations of Section 4.7 of ANSI/ANS 3.1-1978.

The NRC staff finds this requested change acceptable because it meets the appropriate acceptance criteria of Section 13.4 of NUREG 0800, the Standard Review Plan. Reference to the 1978 version of ANSI 3.1 is consistent with other Diablo Canyon TS references to this ANSI standard. The additional experience requirements have been included to satisfy the staff's requirement for years of experience.

- (c) TS 6.5.3.3, "Alternates" - The requested amendment would eliminate the use of alternate offsite review committee members.

The NRC staff finds this requested change acceptable because it places more restrictive controls on the committee membership necessary for the performance of GONPRAC/NSOC functions identified in the TS.

- (d) TS 6.5.3.5, "Meeting Frequency" - The requested amendment would eliminate reference to the committee meeting frequency during the first year of plant operation.

The NRC staff finds this requested change acceptable because the information is no longer applicable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting and administrative requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Brian McDermott

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