



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 7, 1994

Docket Nos. 50-275  
and 50-323

Mr. Gregory M. Rueger  
Nuclear Power Generation, B14A  
Pacific Gas and Electric Company  
77 Beale Street, Room 1451  
P.O. Box 770000  
San Francisco, California 94177

Dear Mr. Rueger:

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER PLANT,  
UNIT NO. 1 (TAC NO. M87698) AND UNIT NO. 2 (TAC NO. M87699)

The Commission has issued the enclosed Amendment No. 91 to Facility Operating License No. DPR-80 and Amendment No. 90 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated September 8, 1993 (reference License Amendment Request (LAR) 93-06).

These amendments revise Technical Specifications 1.44, "Radiological Monitoring and Controls Program," 3/4.11, "Radioactive Effluents," and 6.14, "Radiological Monitoring and Controls Program (RMCP), Offsite Dose Calculation Procedure (ODCP) and Environmental Radiological Monitoring Procedure (ERMP)," to change the Semiannual Radioactive Effluent Release Report to Annual Radioactive Effluent Release report.

The LAR also proposes to revise TS 6.2.3, "Onsite Safety Review Group (OSRG)," 6.5.2, "Plant Staff Review Committee," and 6.5.3.7, "Nuclear Safety Oversight Committee Review," to implement organizational and functional changes.

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Mr. Gregory M. Rueger  
Pacific Gas and Electric Company

Diablo Canyon

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Diablo Canyon Independent Safety Committee  
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Legal Counsel  
857 Cass Street, Suite D  
Monterey, California 93940

Gregory M. Rueger

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A copy of the related Safety Evaluation is enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,



Sheri R. Peterson, Project Manager  
Project Directorate V  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 91 to DPR-80
2. Amendment No. 90 to DPR-82
3. Safety Evaluation

cc w/enclosures:  
See next page

Gregory M. Rueger

- 2 -

March 7, 1994

A copy of the related Safety Evaluation is enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Sheri R. Peterson, Project Manager  
Project Directorate V  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 91 to DPR-80
2. Amendment No. 90 to DPR-82
3. Safety Evaluation

cc w/enclosures:  
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DATE	<i>2/17/94</i>	<i>2/17/94</i>	<i>2/24/94</i>	<i>3/1/94</i>

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91  
License No. DPR-80

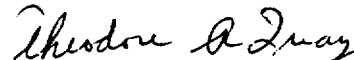
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated September 8, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 91, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director  
Project Directorate V  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 7, 1994



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90  
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated September 8, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 90 , are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director  
Project Directorate V  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 7, 1994



ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Revise Appendix A Technical Specification by removing the page identified below and inserting the enclosed page. The revised page is identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages are also included, as appropriate.

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## DEFINITIONS

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### RADIOLOGICAL MONITORING AND CONTROLS PROGRAM

- 1.44 The RADIOLOGICAL MONITORING AND CONTROLS PROGRAM (RMCP) shall contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Section 6.8.4 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Operating and Annual Radioactive Effluent Release Reports required by Specifications 6.9.1.5 and 6.9.1.6.

## RADIOACTIVE EFFLUENTS

### LIQUID HOLDUP TANKS

#### LIMITING CONDITION FOR OPERATION

---

3.11.1.1 - 3.11.1.3 Deleted

3.11.1.4 The quantity of radioactive material contained in any temporary outdoor tanks shall be limited to less than or equal to 10 curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY: At all times.

#### ACTION:

- a. With the quantity of radioactive material in any of the temporary outdoor tanks exceeding the above limit, immediately suspend all additions of radioactive material to the tank, within 48 hours reduce the tank contents to within the limit, and describe the events leading to this condition in the next Annual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.6.
- b. The provisions of Specification 3.0.3 are not applicable.

#### SURVEILLANCE REQUIREMENTS

---

4.11.1.1 - 4.11.1.3 Deleted

4.11.1.4 The quantity of radioactive material contained in each of the temporary outdoor tanks shall be determined to be within the above limit by analyzing a representative sample of the tank's contents at least once per 7 days when radioactive materials are being added to the tank.

RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

---

3.11.2.1 - 3.11.2.4 Deleted

3.11.2.5 The concentration of oxygen in the GASEOUS RADWASTE SYSTEM shall be limited to less than or equal to 2% by volume whenever the hydrogen concentration exceeds 4% by volume.

APPLICABILITY: At all times.

ACTION:

- a. With the concentration of oxygen in the GASEOUS RADWASTE SYSTEM greater than 2% by volume but less than or equal 4% by volume, reduce the oxygen concentration to the above limits within 48 hours.
- b. With the concentration of oxygen in the GASEOUS RADWASTE SYSTEM greater than 4% by volume and the hydrogen concentration greater than 4% by volume, immediately suspend all additions of waste gases to the system and reduce the concentration of oxygen to less than or equal to 4% by volume, then take ACTION a., above.
- c. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

---

4.11.2.1 - 4.11.2.4 Deleted

4.11.2.5 The concentration of hydrogen\* and oxygen in the GASEOUS RADWASTE SYSTEM shall be determined to be within the above limits by monitoring the waste gases in the GASEOUS RADWASTE SYSTEM with the hydrogen and continuous oxygen monitors required OPERABLE by Table 3.3-13 of Specification 3.3.3.10.

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\*If monitoring of the waste gases for hydrogen is not performed, the hydrogen concentration shall be assumed to be greater than 4% by volume.

## RADIOACTIVE EFFLUENTS

### GAS STORAGE TANKS

#### LIMITING CONDITION FOR OPERATION

---

3.11.2.6 The quantity of radioactivity contained in each gas decay tank shall be limited to less than or equal to  $10^5$  curies noble gases (considered as Xe-133 equivalent).

APPLICABILITY: At all times.

ACTION:

- a. With the quantity of radioactive material in any gas decay tank exceeding the above limit, immediately suspend all additions of radioactive material to the tank, within 48 hours reduce the tank contents to within the limit, and describe the events leading to this condition in the next Annual Radioactive Effluent Release Report, pursuant to Specification 6.9.1.6.
- b. The provisions of Specification 3.0.3 are not applicable.

3.11.3 Deleted

3.11.4 Deleted

#### SURVEILLANCE REQUIREMENTS

---

4.11.2.6 The quantity of radioactive material contained in each gas decay tank shall be determined to be within the above limit at least once per 24 hours when radioactive materials are being added to the tank.

4.11.3 Deleted

4.11.4 Deleted

## ADMINISTRATIVE CONTROLS

### PLANT STAFF (Continued)

- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. Deleted.
- f. Administrative procedures shall be developed and implemented to limit the working hours of plant staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, Health Physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a nominal 40-hour week while the unit is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications, on a temporary basis, the following guidelines shall be followed:

- 1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time;
- 2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time;
- 3) A break of at least 8 hours should be allowed between work periods, including shift turnover time; and
- 4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or his designee, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized; and

- g. The Operations Manager shall hold a senior reactor operator license.



## ADMINISTRATIVE CONTROLS

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### 6.2.3 DELETED

### 6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4 The Shift Technical Advisor shall provide advisory technical support to the Shift Foreman in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant.

### 6.3 PLANT STAFF QUALIFICATIONS

6.3 Each member of the plant staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 2, April 1987 for Radiation Protection Manager. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of 10 CFR Part 55 and the supplemental requirements specified in Section A of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

## ADMINISTRATIVE CONTROLS

### 6.4 TRAINING

6.4 A retraining and replacement training program for the plant staff shall be maintained under the direction of a designated member of the facility staff and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 TECHNICAL REVIEW AND CONTROL

##### ACTIVITIES

6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures, tests, and experiments that affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/group. Each such procedure, test, and experiment, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, test, or experiment, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, test, or experiment, or changes thereto.

6.5.1.2 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1 shall be previously designated by the Plant Manager to perform such reviews. Such designation shall include the disciplines or procedure categories for which each individual is qualified. Each individual designated to perform these reviews shall have at least 3 years of related experience and a baccalaureate degree in engineering or a related field, or shall have at least 8 years of related experience.

#### 6.5.2 PLANT STAFF REVIEW COMMITTEE (PSRC)

##### FUNCTION

6.5.2.1 The Plant Staff Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.2.2 The PSRC shall be chaired by the Plant Manager and shall be composed of a minimum of 8 senior management individuals whose responsibilities include the functional areas of: operations, maintenance, radiation protection, support services, technical services, and quality control. All members shall be appointed in writing by the PSRC Chairman. The qualifications of each PSRC member shall meet or exceed the requirements and recommendations of Section 4.7 of ANSI/ANS 3.1-1978.

##### ALTERNATES

6.5.2.3 The Chairman may designate in writing other regular members who may serve as the Acting Chairman of PSRC meetings. All alternate members shall be appointed in writing by the PSRC Chairman. Alternates may be designated for specific PSRC members and shall have expertise in the same general area as the regular PSRC member they represent. No more than two alternates shall participate as voting members in PSRC activities at any one time.

## ADMINISTRATIVE CONTROLS

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### MEETING FREQUENCY

6.5.2.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

### QUORUM

6.5.2.5 The minimum quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall be a majority (more than one-half) of the members of the PSRC. For purposes of the quorum, this majority shall include the Chairman or his designated alternate and no more than two alternate members.

## ADMINISTRATIVE CONTROLS

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### REVIEW (Continued)

- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components that could affect nuclear safety; and
- i. Reports and meetings minutes of the Plant Staff Review Committee and the Independent Technical Review program.

### AUDITS

6.5.3.8 Audits of plant activities shall be performed under the cognizance of NSOC. These audits shall encompass:

- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
- b. The performance, training and qualifications of the entire plant staff at least once per 12 months;
- c. The results of actions taken to correct deficiencies occurring in plant equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
- e. The fire protection program and implementing procedures at least once per 24 months by qualified personnel;

## ADMINISTRATIVE CONTROLS

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### AUDITS (Continued)

- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. Any other area of plant operation considered appropriate by NSOC or the Senior Vice President and General Manager, Nuclear Power Generation;
- h. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- i. The ODCP and ERMP and implementing procedures at least once per 24 months;
- j. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months; and
- k. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

### RECORDS

6.5.3.9 Records of NSOC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NSOC meeting shall be prepared, approved and forwarded to the Senior Vice President and General Manager, Nuclear Power Generation within 14 working days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.3.7 above, shall be prepared, approved and forwarded to the Senior Vice President and General Manager, Nuclear Power Generation within 14 working days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.3.8 above, shall be forwarded to the Senior Vice President and General Manager, Nuclear Power Generation and to the management positions responsible for the areas audited within 30 days after completion of the audit.

### 6.5.4 INDEPENDENT TECHNICAL REVIEW RESPONSIBILITIES

An Independent Technical Review Program shall be established and maintained to encompass the following Technical Review responsibilities:

## ADMINISTRATIVE CONTROLS

### FUNCTION

6.5.4.1 The Independent Technical Review Program responsibilities shall encompass:

- a. NRC issuances, industry advisories, Licensee Event Reports, and other sources that may indicate areas for improving plant safety;
- b. Internal and external operating experience information that may indicate areas for improving plant safety;
- c. Plant operating characteristics, plant operations, modifications, maintenance, and surveillance to verify independently that these activities are performed safely and correctly and that human errors are reduced as much as practical; and
- d. Making detailed recommendations to the Senior Vice President and General Manager, Nuclear Power Generation for revising procedures, equipment modifications or other means of improving nuclear safety and plant reliability.

The Independent Technical Review Program shall utilize several on-site personnel who are independent of the plant management chain to perform the reviews.

### RECORDS

6.5.4.2 Written records of technical reviews shall be maintained. As a minimum, these records shall include the results of the activities conducted, the status of recommendations made pursuant to Specification 6.5.4.1 and an assessment of plant operations related to the reviews performed.

### QUALIFICATIONS

6.5.4.3 Personnel performing reviews pursuant to Specification 6.5.4.1 shall have at least 3 years of related experience and a bachelor's degree in engineering or a related field; or shall have at least 8 years of related experience.

## 6.6 REPORTABLE EVENT ACTION

6.6. The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.73; and

## ADMINISTRATIVE CONTROLS

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### 6.14 RADIOLOGICAL MONITORING AND CONTROLS PROGRAM (RMCP), OFFSITE DOSE CALCULATION PROCEDURE (ODCP) and ENVIRONMENTAL RADIOLOGICAL MONITORING PROCEDURE (ERMP)

6.14.1 The RMCP, ODCP and ERMP shall be approved by the Commission prior to implementation.

6.14.2 Changes to the RMCP, ODCP, and ERMP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
  - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
  - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after review and acceptance by the PSRC and the approval of the Plant Manager.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire RMCP, ODCP and ERMP as part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the RMCP, ODCP or ERMP was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-80  
AND AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. DPR-82  
PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter of September 8, 1993, Pacific Gas and Electric Company (or the licensee) submitted a request for changes to the Technical Specifications (TS). The proposed amendments would revise TS 1.44, "Radiological Monitoring and Controls Program," 3/4.11, "Radioactive Effluents," and 6.14, "Radiological Monitoring and Controls Program (RMCP), Offsite Dose Calculation Procedure (ODCP) and Environmental Radiological Monitoring Procedure (ERMP)," to change the Semiannual Radioactive Effluent Release Report to Annual Radioactive Effluent Release Report.

The proposed amendments would also revise TS 6.2.3, "Onsite Safety Review Group (OSRG)," 6.5.2, "Plant Staff Review Committee," and 6.5.3.7, "Nuclear Safety Oversight Committee Review," to implement organizational and functional changes.

2.0 EVALUATION

Following is a brief description and our evaluation of the requested changes.

- a. TS Section 6.2.3 - Onsite Safety Review Group (OSRG) - PG&E proposes to delete this group and transfer the OSRG function to the new Nuclear Quality Services (NQS) department. The change will delete the requirement to maintain an onsite, five-person OSRG organization that is dedicated to performing independent technical reviews. The NQS department also includes the PG&E Quality Assurance (QA), Quality Control (QC), and other nuclear safety engineering functions. The Manager, NQS, reports to the Senior Vice President and General Manager, Nuclear Power Generation. The revised requirements will be incorporated into a new Section 6.5.4. See item d. below.



The staff finds these changes acceptable as the function of the OSRG is maintained and it is consistent with the Improved Standard Technical Specifications (ISTS).

- b. TS Section 6.5.2.2 - Plant Staff Review Committee (PSRC) Composition - PG&E proposes to delete the word "plant" from the phrase "senior management individuals" since a member of the committee, the Director QC, will report offsite to the Manager, NQS.

The staff finds this change acceptable as it does not reduce the qualification requirements for PSRC members.

- c. TS Section 6.5.3.7 - Nuclear Safety Oversight Committee (NSOC) Review - PG&E proposes to delete the review of OSRG reports and meeting minutes and add the review of reports of the Independent Technical Review Program.

The staff finds this change acceptable as it reflects the revised organizational provisions for the review of the Independent Technical Review Program activities.

- d. TS Section 6.5.4 - Independent Technical Review Responsibilities - PG&E proposes to add this new section to the TS. This section describes the Independent Technical Review Responsibilities (see item a. above). The description includes the review responsibilities, record keeping, and qualifications requirements for individuals performing these reviews.

The staff finds this requirement acceptable as it retains the functions of the OSRG and conforms to the general characteristic description of this function in the ISTS. The staff finds the commitment to utilizing "sufficient" onsite personnel unacceptable. PG&E has agreed to change this wording to "several" onsite personnel to meet the requirement of NUREG 0737, Item I.B.1.2 and to be consistent with the ISTS. The staff finds this acceptable.

Also, PG&E inadvertently failed to change references to the Semiannual Radioactive Effluent Release Report contained in TS 1.44, 3.11.1.4, 3.11.2.6 and 6.14.2 when LAR 92-10 (PG&E Letter No. DCL-92-283, dated December 22, 1992) was submitted to the NRC. License Amendments 77 and 78 changed the frequency of the Radioactive Effluent Release Report from semiannual to annual. This change is administrative in nature, but serves to clarify the TS to avoid any confusion.

Based on the above discussion, the staff finds the proposed TS amendment acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Allenspach

Date: March 7, 1994