April 1, 1993

Docket Nos. 50-275
and 50-323

Mr. Gregory M. Rueger
Nuclear Power Generation, B14A
Pacific Gas and Electric Company
77 Beale Street, Room 1451
P.O. Box 770000
San Francisco, California 94177

Dear Mr. Rueger:

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1 (TAC NO. M84975) AND UNIT NO. 2 (TAC NO. M84976)

The Commission has issued the enclosed Amendment No. 79 to Facility Operating License No. DPR-80 and Amendment No. 78 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated October 27, 1992 (Reference LAR 92-06).

These amendments relocate TS Table 3.8-1, "Motor-Operated Valves (MOVs) Thermal Overload Protection and Bypass Devices," to Diablo Canyon Power Plant procedures in accordance with the guidance provided in Generic Letter (GL) 91-08, "Removal of Component Lists from Technical Specifications," dated May 6, 1991.

A copy of the related Safety Evaluation is enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Sheri R. Peterson, Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 79 to DPR-80
2. Amendment No. 78 to DPR-82
3. Safety Evaluation

cc w/enclosures:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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1. Amendment No. 79 to DPR-80
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cc w/enclosures:
See next page

Mr. Gregory M. Rueger
Pacific Gas and Electric Company

Diablo Canyon

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated October 27, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 79 , are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Charles M. Trammell
Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1993

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated October 27, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 78 , are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Charles M. Trammell

for Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1993

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO.78 TO FACILITY OPERATING LICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages are also included, as appropriate.

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5 of 6	5 of 6
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DIABLO CANYON NUCLEAR POWER PLANT UNITS 1 AND 2 TECHNICAL SPECIFICATIONS (NUREG-1151)

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

ELECTRICAL POWER SYSTEMS

3/4.8.4 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

MOTOR-OPERATED VALVES THERMAL OVERLOAD PROTECTION AND BYPASS DEVICES

LIMITING CONDITION FOR OPERATION

3.8.4.1 The thermal overload protection and bypass devices, integral with the motor starter, of each valve used in safety systems shall be OPERABLE.

APPLICABILITY: Whenever the motor-operated valve is required to be OPERABLE.

ACTION:

With one or more of the thermal overload protection and/or bypass devices inoperable, declare the affected valve(s) inoperable and apply the appropriate ACTION Statement(s) for the affected valves.

SURVEILLANCE REQUIREMENTS

4.8.4.1 The above required thermal overload protection and bypass devices shall be demonstrated OPERABLE:

a. At least once per 18 months, by the performance of a TRIP ACTUATION DEVICE OPERATIONAL TEST of the bypass circuitry for those thermal overload devices which are either:

   1) Continuously bypassed and temporarily placed in force only when the valve motors are undergoing periodic or maintenance testing, or

   2) Normally in force during plant operation and bypassed under accident conditions.

b. At least once per 18 months by the performance of a CHANNEL CALIBRATION of a representative sample of at least 25% of:

   1) All thermal overload devices which are not bypassed, such that each non-bypassed device is calibrated at least once per 6 years, and

   2) All thermal overload devices which are continuously bypassed, such that each continuously bypassed device is calibrated and each valve is cycled through at least one complete cycle of full travel with the motor-operator when the thermal overload device is OPERABLE and not bypassed at least once per 6 years.

CONTAINMENT PENETRATION CONDUCTOR OVERCURRENT PROTECTIVE DEVICES

LIMITING CONDITION FOR OPERATION

3.8.4.2 For each containment penetration provided with a containment penetration overcurrent protective device(s), each device shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one or more of the above required containment penetration conductor overcurrent protective device(s) inoperable:

a. Restore the protective device(s) to OPERABLE status or deenergize the circuit(s) by tripping the associated protective device or racking out or removing the inoperable protective device within 72 hours, declare the affected system or component inoperable, and verify the associated protective device to be tripped or removed, or the inoperable protective device racked out or removed at least once per 7 days thereafter; or

b. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.4.2 Protective devices required to be operable as containment penetration overcurrent protective devices shall be demonstrated OPERABLE:

a. At least once per 18 months:

1) By verifying that the medium voltage 12 kV circuit breakers are OPERABLE by selecting, on a rotating basis, at least 10% of the circuit breakers and performing the following:

a) A CHANNEL CALIBRATION of the associated protective relays,

b) An integrated system functional test which includes simulated automatic actuation of the system and verifying that each relay and associated circuit breaker and overcurrent control circuit function as designed, and

SURVEILLANCE REQUIREMENTS (Continued)

c) For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least 10% of all the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested.

2) By selecting and functionally testing a representative sample of at least 10% of each type of lower voltage circuit breakers. Circuit breakers selected for functional testing shall be selected on a rotating basis. Testing of "drawout" type circuit breakers shall consist of a CHANNEL CALIBRATION of the associated solid-state trip device for both the long-time delay trip element and the short-time delay element along with a breaker functional test. Testing of molded case circuit breakers shall consist of injecting a current with a value equal to 200% (for D.C. breakers) and 300% (for A.C. breakers) of the pickup of the time delay element and verifying that the circuit breaker operates within the time delay band for that current specified by the manufacturer. The instantaneous element of molded case circuit breakers shall be tested by injecting a current equal to -25%, +40% of the pickup value of the element and verifying that the circuit breaker trips with no intentional time delay. Circuit breakers found out-of-tolerance during functional testing shall be replaced prior to resuming operation. Circuit breakers that fail to trip magnetically before the withstand capability of the penetration conductor is reached shall be declared inoperable. Circuit breakers that fail to trip thermally before the manufacturer's maximum tolerance shall be declared inoperable. For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least 10% of all the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested; and

3) By verifying that the thermal overload devices integral with the motor starters, used for penetration overcurrent protection, are OPERABLE by selecting a representative sample of at least 10% of the motor overload devices and performing a CHANNEL CALIBRATION. Motor overloads found inoperable shall be restored to OPERABLE status prior to resuming operation. For each motor overload device found inoperable, a CHANNEL CALIBRATION shall be performed on an additional representative sample of at least 10% of all the motor overload devices of the inoperable type until no more failures are found or a CHANNEL CALIBRATION has been performed on all motor overload devices of that type.

b. At least once per 60 months by subjecting each circuit breaker to an inspection and preventive maintenance in accordance with procedures prepared in conjunction with its manufacturer's recommendations.

A.C. Sources, D.C. Sources, and ONSITE POWER DISTRIBUTION (Continued)

will not be more than 0.040 below the manufacturer's full charge specific gravity and that the overall capability of the battery will be maintained within an acceptable limit; and (4) the allowable value for an individual cell's float voltage, greater than 2.07 volts, ensures the battery's capability to perform its design function.

The OPERABILITY of the A.C. electrical power sources requires maintaining a supply of fuel oil to support the operation of the emergency diesel generators. The stored fuel oil supports the function of the A.C. power sources to provide power for the operation of emergency systems and engineered safety features (ESF) during and following the shutdown of the reactor in the event that offsite power sources are not available. The specified fuel oil quantity is based on the calculated fuel oil consumption necessary to support the operation of the emergency power source to power the minimum required ESF systems. Operation of minimum ESF systems is required to mitigate a design basis accident (LOCA) in one unit and those minimum required systems for a concurrent non-LOCA safe shutdown in the remaining unit (both units initially in Mode 1 operation). The fuel oil consumption is calculated for a period of 7 days operation of minimum ESF systems. This requirement provides a sufficient operating period within which offsite power can be restored and/or additional fuel can be delivered to the site.

The Surveillance Requirements applicable to diesel generator fuel oil storage requires cleaning the fuel oil storage tanks on a 10-year frequency. Conducting this surveillance requires the tank to be taken out of service. For this infrequent event, the inventory in the remaining tank is sufficient to support operation of the emergency diesel generator to power the minimum required loads to maintain safe conditions for a time period of 4 days, considering one unit in Mode 1 through 6 operation and one unit in Mode 6 operation with at least 23 feet of water above the reactor vessel flange or with the reactor defueled.

3/4.8.4 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

The OPERABILITY of the motor operated valves thermal overload protection and bypass devices ensures that these devices will not prevent safety related valves from performing their function. The Surveillance Requirements for demonstrating the OPERABILITY of these devices are in accordance with Regulatory Guide 1.106, "Thermal Overload Protection for Electric Motors on Motor Operated Valves," Revision 1, March 1977.

A list of the TS-controlled MOV thermal overload protection and bypass devices is maintained in the Diablo Canyon plant procedures. The administration of the list shall be conducted in accordance with Section 50.59 of 10 CFR Part 50 and the provisions in the Administrative Controls Section of the TS. Records of the changes to the valve list are maintained, and an annual report is made that includes a brief description of changes and a summary of the safety evaluation of each in accordance with 10 CFR 50.59.

Containment electrical penetrations and penetration conductors are protected by either deenergizing circuits not required during reactor operation or by demonstrating the OPERABILITY of primary and backup overcurrent protection circuit breakers during periodic surveillance.

ELECTRICAL EQUIPMENT PROTECTIVE DEVICES (Continued)

The Surveillance Requirements applicable to lower voltage circuit breakers provide assurance of breaker reliability by testing at least one representative sample of each manufacturer's brand of circuit breaker. Each manufacturer's molded case and metal case circuit breakers are grouped into representative samples which are then tested on a rotating basis to ensure that all breakers are tested. If a wide variety exists within any manufacturer's brand of circuit breakers, it is necessary to divide that manufacturer's breakers into groups and treat each group as a separate type of breaker for surveillance purposes.

A list of containment penetration conductor overcurrent protective devices, with information on location and size and equipment powered by the protected circuit, is maintained and controlled at the plant site. The list is limited to those overcurrent devices installed for the purpose of keeping circuit fault current below the penetration rating. It does not apply to other overcurrent devices associated with containment penetrations. The addition or deletion of any containment penetration conductor overcurrent protective device is governed by Section 50.59 of 10 CFR Part 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. DPR-82

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2

DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter of October 27, 1992, Pacific Gas and Electric Company (or the licensee) submitted a request for changes to the Technical Specifications (TS). The proposed amendments would remove TS Table 3.8-1, "Motor-Operated Valves (MOVs) Thermal Overload Protection and Bypass Devices," which is a list of components referenced in individual specifications. In addition, the TS requirements have been modified such that all references to this table have been removed. Finally, the applicable TS has been modified to state requirements in general terms that include the components listed in the table removed from the TS. Guidance on the proposed TS changes was provided by Generic Letter 91-08, dated May 6, 1991.

2.0 EVALUATION

The licensee has proposed the removal of Table 3.8-1, "Motor-Operated Valves (MOVs) Thermal Overload Protection and Bypass Devices," that provides a list of valves with bypass devices that is referenced in TS 3.8.4.1. With the removal of this table, the licensee has proposed to include the following statement of the limiting condition for operation (LCO) under TS 3.8.4.1:

> The thermal overload protection and bypass devices, integral with the motor starter, of each valve used in safety systems shall be OPERABLE.

The licensee has proposed changes to the above TS that are consistent with the guidance provided in Generic Letter 91-08. In addition, the licensee has provided an updated copy of the Bases Section of TS 3/4 8.4 that addresses appropriate considerations for administration of the component list in plant procedures. Finally, the licensee has confirmed that the component list removed from the TS will be located in controlled plant procedures.

On the basis of its review of this matter, the staff finds that the proposed changes to the TS for Diablo Canyon Units 1 and 2 are primarily an administrative change that does not alter the requirement set forth in the existing TS. Overall, these changes will allow the licensee to make

corrections and updates to the list of components for which these TS requirements apply, under the provisions that control changes to plant procedures as specified in the Administrative Controls Section of the TS. Therefore, the staff finds that the proposed TS changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (58 FR 598). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Date: April 1, 1993