



Entergy Nuclear Operations, Inc.  
Pilgrim Station  
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Director, Nuclear Assessment

August 14, 2002

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: Entergy Nuclear Operations, Inc.  
Pilgrim Nuclear Power Station  
Docket No. 50-293  
License No. DPR-35

Correction of a Technical Specification Typographical Error

- REFERENCES:
1. Issuance of Amendment No. 176 to Facility Operating License No DPR-35, Pilgrim Nuclear Power Station, Containment Cooling Ultimate Heat Sink Inlet Temperature (TAC MA1150), dated July 28, 1998
  2. Proposed Guidance for Correction of Technical Specification Typographical Errors; SECY-96-238, Dated November 19, 1996

LETTER NUMBER: 2.02.074

Dear Sir or Madam:

This letter requests NRC approval of a correction of an inadvertent typographical error in Pilgrim Technical Specifications which was inadvertently introduced during License Amendment 176 (Reference 1).

It was not addressed in the notice to the public nor reviewed by the NRC and falls within the scope of the guidance provided in SECY-96-238 (Reference 2) for corrections.

Attachment 1 describes the typographical error and correction. Attachment 2 provides a mark-up of Technical Specification page. Entergy will issue change notifications to all holders of controlled copies of Pilgrim Technical Specifications upon receipt of NRC approval of this correction.

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If you have any questions regarding the information contained in this letter, please contact Mr. Bryan Ford (508) 830-8403.

Sincerely,



W.J. Riggs

- Attachments:
1. Description of Technical Specification Typographical Error - (1 page)
  2. Marked-up Technical Specification Page Correcting the Typographical Error - (1 page)

cc:

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# **ATTACHMENT 1**

**DESCRIPTION OF TECHNICAL SPECIFICATION**

**TYPOGRAPHICAL ERROR**

A. REQUESTED ACTION

Consistent with the information contained in SECY-96-238, Pilgrim Station is requesting a correction to an inadvertent typographical error that was introduced into the Pilgrim Technical Specification.

B. TYPOGRAPHICAL ERROR

A typographical error was inadvertently introduced into the Pilgrim Technical Specification (TS) during License Amendment 176 (Reference 1). The specifics of this error and proposed correction is described below.

TS page 3/4 .5-8

Amendment 176 approved TS 4.5.D. with "HPCI system testing shall be as follows." This Specification is on TS page 3/4 .5-8. During the word processing of this page, "HPCI" was accidentally used instead of "RCIC". With the correction, TS 4.5.D should read as "RCIC system testing shall be as follows:"

This error was not noticed to the public nor approved by the NRC during review and approval of Amendment 176.

The above error was discovered during the use of applicable specifications. This typographical error did not cause any safety issues.

Attachment 2 provides the marked-up TS page with the correction to the typographical error.

C. CORRECTIONS TO THE AFFECTED TECHNICAL SPECIFICATION PAGE

SECY-96-238 (Reference 2) provides guidance to correct inadvertent typographical errors in the Technical Specification pages.

The above typographical error was not noticed to the public nor reviewed by the NRC as part of the applicable amendment process. Therefore it may be corrected without a license amendment.

Accordingly, upon approval from the NRC, corrected TS page 3/4 .5-8 will be distributed to all holders of controlled Technical Specifications.

## **ATTACHMENT 2**

**MARKED-UP TECHNICAL SPECIFICATION PAGE  
CORRECTING THE TYPOGRAPHICAL ERROR**

**TS PAGE 3/4.5-8**

LIMITING CONDITIONS FOR OPERATION

3.5 CORE AND CONTAINMENT COOLING SYSTEMS

D. Reactor Core Isolation Cooling (RCIC) System

1. The RCIC system shall be operable whenever there is irradiated fuel in the reactor vessel, reactor pressure is greater than 150 psig, and reactor coolant temperature is greater than 365°F, except as specified in 3.5.D.2 below.
2. From and after the date that the RCIC system is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 14 days unless such system is sooner made operable, providing that during such 14 days the HPCIS is operable.
3. If the requirements of 3.5.D cannot be met, an orderly shutdown of the reactor shall be initiated and the reactor shall be in the Cold Shutdown Condition within 24 hours.

SURVEILLANCE REQUIREMENTS

4.5 CORE AND CONTAINMENT COOLING SYSTEMS

D. Reactor Core Isolation Cooling (RCIC) System

1. ~~HPCIS~~<sup>RCIC</sup> system testing shall be as follows:

- |                                       |  |
|---------------------------------------|--|
| a. Simulated Automatic Actuation Test | Once/ Operating Cycle  |
| b. Pump Operability                   | When tested as specified in 3.13, verify that the RCIC pump delivers at least 400 GPM at a system head corresponding to a reactor pressure of 1000 psig. |
| c. Motor Operated Valve Operability   | As Specified in 3.13   |
| d. Flow Rate at 150 psig.             | Once/ operating cycle verify that the RCIC pump delivers at least 400 GPM at a system head corresponding to a reactor pressure of 150 psig.              |

The RCIC pump shall deliver at least 400 GPM for a system head corresponding to a reactor pressure of 1000 to 150 psig.