

August 21, 2002

Mr. Valeri Tolstykh  
Regulatory Activities Unit  
Safety Assessment Section  
Division of Nuclear Installation Safety  
International Atomic Energy Agency  
Wagramer Strasse 5  
P.O. Box 100, A-1400  
Vienna, Austria

Dear Mr. Tolstykh:

The following operating experience reports from United States reactors are enclosed for your consideration for including in the AIRS database:

NRC INFORMATION NOTICE 2002-01: METALCLAD SWITCHGEAR FAILURES AND  
CONSEQUENT LOSSES OF OFFSITE POWER

NRC INFORMATION NOTICE 2002-02: RECENT EXPERIENCE WITH PLUGGED STEAM  
GENERATOR TUBES

NRC INFORMATION NOTICE 2002-02, SUPPLEMENT 1: RECENT EXPERIENCE WITH  
PLUGGED STEAM GENERATOR TUBES

NRC INFORMATION NOTICE 2002-03: HIGHLY RADIOACTIVE PARTICLE CONTROL  
PROBLEMS DURING SPENT FUEL POOL CLEANOUT

NRC INFORMATION NOTICE 2002-04: WIRE DEGRADATION AT BREAKER CUBICLE  
DOOR HINGES

NRC INFORMATION NOTICE 2002-05: FOREIGN MATERIAL IN STANDBY LIQUID  
CONTROL STORAGE TANKS

NRC INFORMATION NOTICE 2002-06: DESIGN VULNERABILITY IN BWR REACTOR  
VESSEL LEVEL INSTRUMENTATION BACKFILL MODIFICATION

NRC INFORMATION NOTICE 2002-08: PUMP SHAFT DAMAGE DUE TO EXCESSIVE  
HARDNESS OF SHAFT SLEEVE

NRC INFORMATION NOTICE 2002-09: POTENTIAL FOR TOP NOZZLE SEPARATION AND  
DROPPING OF A CERTAIN TYPE OF WESTINGHOUSE FUEL ASSEMBLY

NRC INFORMATION NOTICE 2002-10, SUPPLEMENT 1: DIABLO CANYON MANUAL  
REACTOR TRIP AND STEAM GENERATOR WATER LEVEL SETPOINT UNCERTAINTIES

NRC INFORMATION NOTICE 2002-12: SUBMERGED SAFETY-RELATED ELECTRICAL  
CABLES

NRC INFORMATION NOTICE 2002-18: EFFECT OF ADDING GAS INTO WATER STORAGE TANKS ON THE NET POSITIVE SUCTION HEAD FOR PUMPS

NRC INFORMATION NOTICE 2002-21: AXIAL OUTSIDE-DIAMETER CRACKING AFFECTING THERMALLY TREATED ALLOY 600 STEAM GENERATOR TUBING

NRC INFORMATION NOTICE 2002-22: DEGRADED BEARING SURFACES IN GM/EMD EMERGENCY DIESEL GENERATORS

NRC INFORMATION NOTICE 2002-24: POTENTIAL PROBLEMS WITH HEAT COLLECTORS ON FIRE PROTECTION SPRINKLERS

Each report is being submitted in the following two media: (1) a hard copy of the input file for the AIRS database; and (2) a 3.5-inch HD diskette containing the input file for the AIRS database in Microsoft Word 6.0 format.

If you have any questions regarding these reports, please call Jerry Dozier of my staff. He can be reached at 301-415-1014.

Sincerely,

*/RA/*

William D. Beckner, Program Director  
Operating Reactor Improvements Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/Enclosures:  
Mr. Lennart Carlsson  
Nuclear Safety Division  
Nuclear Energy Agency  
Organization for Economic  
Cooperation and Development  
Le Seine Saint Germain  
12, Boulevard des Iles  
92130, Issy-les-Moulineaux, France

NRC INFORMATION NOTICE 2002-18: EFFECT OF ADDING GAS INTO WATER STORAGE TANKS ON THE NET POSITIVE SUCTION HEAD FOR PUMPS

NRC INFORMATION NOTICE 2002-21: AXIAL OUTSIDE-DIAMETER CRACKING AFFECTING THERMALLY TREATED ALLOY 600 STEAM GENERATOR TUBING

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ADAMS ACCESSION NUMBER: ML022340233  
DOCUMENT NAME: G:\RORP\OES\STAFF FOLDERS\DOZIER\IRSREPORT2002.WPD

OFFICE	RSE:OES:RORP:DRIP	IMA:OES:RORP:DRIP	SC:OES:RORP:DRIP	PD:RORP:DRIP
NAME	IJDozier	KAGray	TReis	WDBeckner
DATE	08/09/2002	08/14/2002	08/16/2002	08/21/2002

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>2/3/2001</b>	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b>			
NRC INFORMATION NOTICE 2002-01:	METALCLAD SWITCHGEAR FAILURES AND CONSEQUENT LOSSES OF OFFSITE POWER		
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	San Onofre Unit 3	PWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
GEC	April 1, 1984		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) issued this information notice to inform addressees of electrical equipment failure modes and design vulnerabilities identified following recent transients at the San Onofre Nuclear Generating Station and at a foreign nuclear power station. The most interesting aspect in both events was propagation of damage from an electrical fault in one breaker cubicle to other breakers and buswork in the same enclosure.

NRC INFORMATION NOTICE 2002-01

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |  |              |                   |                   |
|--|--------------|-------------------|-------------------|
| 1. <u>Reporting Categories:</u>            | <u>1.3.1</u> | <u>1.4</u>        | <u>          </u> |
| 2. <u>Plant Status Prior to the Event:</u> | <u>2.1.3</u> | <u>          </u> | <u>          </u> |
| 3. <u>Failed/Affected Systems:</u>         | <u>3.EB</u>  | <u>          </u> | <u>          </u> |
| 4. <u>Failed/Affected Components:</u>      | <u>4.3.2</u> | <u>          </u> | <u>          </u> |
| 5. <u>Cause of the Event:</u>              | <u>5.1.2</u> | <u>          </u> | <u>          </u> |
| 6. <u>Effects on Operation:</u>            | <u>6.1</u>   | <u>          </u> | <u>          </u> |
| 7. <u>Characteristics of the Incident:</u> | <u>7.9</u>   | <u>          </u> | <u>          </u> |
| 8. <u>Nature of Failure or Error:</u>      | <u>8.4</u>   | <u>          </u> | <u>          </u> |
| 9. <u>Nature of Recovery Actions:</u>      | <u>9.2</u>   | <u>          </u> | <u>          </u> |

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>10/8/2001</b>	<b>DATE RECEIVED</b>
		<b>EVENT TITLE</b>	
NRC INFORMATION NOTICE 2002-02:		RECENT EXPERIENCE WITH PLUGGED STEAM GENERATOR TUBES	
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	Three Mile Island Unit 1	PWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
Babcock and Wilcox	September 2, 1974		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees about findings from recent inspections and examinations of steam generator tubes at Three Mile Island Unit 1 (TMI-1).

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1.	<u>Reporting Categories:</u>	<u>1.2.2</u>	_____	_____
2.	<u>Plant Status Prior to the Event:</u>	<u>2.3.2</u>	_____	_____
3.	<u>Failed/Affected Systems:</u>	<u>3.AH</u>	_____	_____
4.	<u>Failed/Affected Components:</u>	<u>4.2.6</u>	_____	_____
5.	<u>Cause of the Event:</u>	<u>5.1.1.1</u>	<u>5.1.1.2</u>	<u>5.1.1.3</u>
6.	<u>Effects on Operation:</u>	<u>6.9</u>	_____	_____
7.	<u>Characteristics of the Incident:</u>	<u>7.2</u>	_____	_____
8.	<u>Nature of Failure or Error:</u>	_____	<u>8.3</u>	<u>8.4</u>
9.	<u>Nature of Recovery Actions:</u>	<u>9.0</u>	_____	_____

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>10/8/2001</b>	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b>			
NRC INFORMATION NOTICE 2002-02 S1: RECENT EXPERIENCE WITH PLUGGED STEAM GENERATOR TUBES			
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	Oconee Nuclear Station Unit 1	PWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
Babcock and Wilcox	July 15, 1973		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees about findings from recent inspections and examinations of steam generator tubes at Oconee Nuclear Station Unit 1.

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1. <b><u>Reporting Categories:</u></b>	<b><u>1.2.2</u></b>	<u>          </u>	<u>          </u>
2. <b><u>Plant Status Prior to the Event:</u></b>	<b><u>2.3.2</u></b>	<u>          </u>	<u>          </u>
3. <b><u>Failed/Affected Systems:</u></b>	<b><u>3.AH</u></b>	<u>          </u>	<u>          </u>
4. <b><u>Failed/Affected Components:</u></b>	<b><u>4.2.6</u></b>	<u>          </u>	<u>          </u>
5. <b><u>Cause of the Event:</u></b>	<b><u>5.1.1.1</u></b>	<b><u>5.1.1.2</u></b>	<b><u>5.1.1.3</u></b>
6. <b><u>Effects on Operation:</u></b>	<b><u>6.9</u></b>	<u>          </u>	<u>          </u>
7. <b><u>Characteristics of the Incident:</u></b>	<b><u>7.2</u></b>	<u>          </u>	<u>          </u>
8. <b><u>Nature of Failure or Error:</u></b>	<b><u>8.3</u></b>	<b><u>8.4</u></b>	<u>          </u>
9. <b><u>Nature of Recovery Actions:</u></b>	<b><u>9.0</u></b>	<u>          </u>	<u>          </u>

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>10/12/2000</b>	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b>			
NRC INFORMATION NOTICE 2002-03:		HIGHLY RADIOACTIVE PARTICLE CONTROL PROBLEMS DURING SPENT FUEL POOL CLEANOUT	
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	Susquehanna Steam Electric Station	BWR/4	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
GE	June 6, 1983		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to alert addressees to recent issues involving evaluation and control of radioactive particles generated during removal of material from a spent fuel pool prior to shipping the material offsite for disposal. The issue emphasized in this notice is that highly radioactive (hot) particles represent a radiological hazard not just in terms of shallow dose to the skin or an extremity but also as a deep or whole-body dose.

NRC INFORMATION NOTICE 2002-03

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1. **Reporting Categories:**                    **1.4**        \_\_\_\_\_
  
2. **Plant Status Prior to the Event:**   **2.3.2**        \_\_\_\_\_
  
3. **Failed/Affected Systems:**            **3.KG**        \_\_\_\_\_
  
4. **Failed/Affected Components:**       **4.2.11**        \_\_\_\_\_
  
5. **Cause of the Event:**                    **5.4.16**        \_\_\_\_\_
  
6. **Effects on Operation:**                 **6.0**        \_\_\_\_\_
  
7. **Characteristics of the Incident:**      **7.14**        \_\_\_\_\_
  
8. **Nature of Failure or Error:**           **8.1**        \_\_\_\_\_
  
9. **Nature of Recovery Actions:**         **9.2**        \_\_\_\_\_

# INCIDENT REPORTING SYSTEM

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IRS NO.                      EVENT DATE      3/14/2001      DATE RECEIVED

**EVENT TITLE**

NRC INFORMATION NOTICE 2002-04: WIRE DEGRADATION AT BREAKER CUBICLE  
DOOR HINGES

**COUNTRY**

USA

**PLANT AND UNIT**

Diablo Canyon Unit 2

**REACTOR TYPE**

PWR

**INITIAL STATUS**

**RATED POWER (MWe NET)**

N/A

**DESIGNER**

Westinghouse

**1st COMMERCIAL OPERATION**

March 13, 1986

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**ABSTRACT**

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of degraded breaker cubicle wires found at the Diablo Canyon Nuclear Power Plant. These wires connect electrical equipment mounted on cubicle doors to equipment inside the breaker cubicles. Over time, the wires degraded due to cold-working and aging.

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |   |                       |                    |                   |
|---|-----------------------|--------------------|-------------------|
| 1. <b><u>Reporting Categories:</u></b>            | <b><u>1.4</u></b>     | <u>          </u>  | <u>          </u> |
| 2. <b><u>Plant Status Prior to the Event:</u></b> | <b><u>2.1.1</u></b>   | <u>          </u>  | <u>          </u> |
| 3. <b><u>Failed/Affected Systems:</u></b>         | <b><u>3.DD</u></b>    | <b><u>3.EB</u></b> | <u>          </u> |
| 4. <b><u>Failed/Affected Components:</u></b>      | <b><u>4.3.2</u></b>   | <u>          </u>  | <u>          </u> |
| 5. <b><u>Cause of the Event:</u></b>              | <b><u>5.1.1.2</u></b> | <u>          </u>  | <u>          </u> |
| 6. <b><u>Effects on Operation:</u></b>            | <b><u>6.0</u></b>     | <u>          </u>  | <u>          </u> |
| 7. <b><u>Characteristics of the Incident:</u></b> | <b><u>7.5</u></b>     | <u>          </u>  | <u>          </u> |
| 8. <b><u>Nature of Failure of Error:</u></b>      | <b><u>8.3</u></b>     | <b><u>8.4</u></b>  | <u>          </u> |
| 9. <b><u>Nature of Recovery Actions:</u></b>      | <b><u>9.0</u></b>     | <u>          </u>  | <u>          </u> |

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>9/14/2000</b>	<b>DATE RECEIVED</b>
NRC INFORMATION NOTICE 2002-05:		<b>EVENT TITLE</b> FOREIGN MATERIAL IN STANDBY LIQUID CONTROL STORAGE TANKS	
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> River Bend Unit 1	<b>REACTOR TYPE</b> BWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b> N/A		
<b>DESIGNER</b> GE	<b>1st COMMERCIAL OPERATION</b> June 16, 1986		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of equipment and design issues identified recently at the River Bend Nuclear Power Plant concerning debris found in the standby liquid control system storage tank. The aspect of this event considered of particular interest is the evident potential for systems initially filled with a combination of dry chemicals and water to have the wrapper material for the chemicals left in the system and to remain there undetected for an extended period.

NRC INFORMATION NOTICE 2002-05

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |  |                 |              |              |
|--|-----------------|--------------|--------------|
| 1. <u>Reporting Categories:</u>            | <u>1.3.1</u>    | _____        | _____        |
| 2. <u>Plant Status Prior to the Event:</u> | <u>2.1.1</u>    | _____        | _____        |
| 3. <u>Failed/Affected Systems:</u>         | <u>3.IN</u>     | _____        | _____        |
| 4. <u>Failed/Affected Components:</u>      | <u>4.1.3</u>    | _____        | _____        |
| 5. <u>Cause of the Event:</u>              | <u>5.1.10.2</u> | <u>5.6.2</u> | <u>5.7.1</u> |
| 6. <u>Effects on Operation:</u>            | <u>6.1.2</u>    | _____        | _____        |
| 7. <u>Characteristics of the Incident:</u> | <u>7.13</u>     | _____        | _____        |
| 8. <u>Nature of Failure or Error:</u>      | <u>8.2</u>      | <u>8.4</u>   | _____        |
| 9. <u>Nature of Recovery Actions:</u>      | <u>9.1</u>      | _____        | _____        |

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>8/1/2001</b>	<b>DATE RECEIVED</b>
NRC INFORMATION NOTICE 2002-06:		<b>EVENT TITLE</b> DESIGN VULNERABILITY IN BWR REACTOR VESSEL LEVEL INSTRUMENTATION BACKFILL MODIFICATION	
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> Pilgrim Nuclear Power Station	<b>REACTOR TYPE</b> BWR/3	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b> N/A		
<b>DESIGNER</b> GE	<b>1st COMMERCIAL OPERATION</b> December 1, 1972		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to alert addressees to a potential design vulnerability in a hardware modification to the reactor vessel water level instrumentation system.

NRC INFORMATION NOTICE 2002-06

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |  |              |                   |                   |
|--|--------------|-------------------|-------------------|
| 1. <u>Reporting Categories:</u>            | <u>1.3</u>   | <u>1.4</u>        | <u>          </u> |
| 2. <u>Plant Status Prior to the Event:</u> | <u>2.1.1</u> | <u>          </u> | <u>          </u> |
| 3. <u>Failed/Affected Systems:</u>         | <u>3.IN</u>  | <u>          </u> | <u>          </u> |
| 4. <u>Failed/Affected Components:</u>      | <u>4.1.3</u> | <u>          </u> | <u>          </u> |
| 5. <u>Cause of the Event:</u>              | <u>5.7.1</u> | <u>          </u> | <u>          </u> |
| 6. <u>Effects on Operation:</u>            | <u>6.0</u>   | <u>          </u> | <u>          </u> |
| 7. <u>Characteristics of the Incident:</u> | <u>7.5</u>   | <u>          </u> | <u>          </u> |
| 8. <u>Nature of Failure or Error:</u>      | <u>8.4</u>   | <u>          </u> | <u>          </u> |
| 9. <u>Nature of Recovery Actions:</u>      | <u>9.0</u>   | <u>          </u> | <u>          </u> |

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>4/1/2001</b>	<b>DATE RECEIVED</b>
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	<b>EVENT TITLE</b>
NRC INFORMATION NOTICE 2002-08:	PUMP SHAFT DAMAGE DUE TO EXCESSIVE HARDNESS OF SHAFT SLEEVE

<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>
USA	Columbia Generating Station	BWR

<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>
	N/A

<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>
GE	December 13, 1984

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to alert addressees of potentially serious damage that occurred to pump shafts manufactured by Bingham-Willamette. The damage resulted from harder-than-specified material used during the manufacture of the thrust sleeve installed on the pump shaft.

NRC INFORMATION NOTICE 2002-08

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1. <u>Reporting Categories:</u>	<u>1.3.4</u>	<u>1.4</u>	_____
2. <u>Plant Status Prior to the Event:</u>	<u>2.1.1</u>	_____	_____
3. <u>Failed/Affected Systems:</u>	<u>3.BA</u>	_____	_____
4. <u>Failed/Affected Components:</u>	<u>4.2.1</u>	_____	_____
5. <u>Cause of the Event:</u>	<u>5.1.1</u>	<u>5.1.1.7</u>	_____
6. <u>Effects on Operation:</u>	<u>6.9</u>	_____	_____
7. <u>Characteristics of the Incident:</u>	<u>7.5</u>	_____	_____
8. <u>Nature of Failure or Error:</u>	<u>8.1</u>	_____	_____
9. <u>Nature of Recovery Actions:</u>	<u>9.0</u>	_____	_____

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>3/24/2001</b>	<b>DATE RECEIVED</b>
		<b>EVENT TITLE</b>	
NRC INFORMATION NOTICE 2002-09:		POTENTIAL FOR TOP NOZZLE SEPARATION AND DROPPING OF A CERTAIN TYPE OF WESTINGHOUSE FUEL ASSEMBLY	
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	North Anna Power Station	PWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
Westinghouse	June 6, 1978		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to alert addressees to the recent nozzle separation and dropping of a Westinghouse fuel assembly during movement. Even though the nozzle separation affects only fuel of a type last manufactured almost 20 years ago, the fuel is perhaps being moved to dry storage or high-density racks and could drop during movement.

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1.	<b><u>Reporting Categories:</u></b>	<b><u>1.3.1</u></b>	<b><u>1.4</u></b>	<u>          </u>
2.	<b><u>Plant Status Prior to the Event:</u></b>	<b><u>2.0</u></b>	<u>          </u>	<u>          </u>
3.	<b><u>Failed/Affected Systems:</u></b>	<b><u>3.AA</u></b>	<u>          </u>	<u>          </u>
4.	<b><u>Failed/Affected Components:</u></b>	<b><u>4.2.10</u></b>	<u>          </u>	<u>          </u>
5.	<b><u>Cause of the Event:</u></b>	<b><u>5.1.1.1</u></b>	<u>          </u>	<u>          </u>
6.	<b><u>Effects on Operation:</u></b>	<b><u>6.0</u></b>	<u>          </u>	<u>          </u>
7.	<b><u>Characteristics of the Incident:</u></b>	<b><u>7.1</u></b>	<b><u>7.14</u></b>	<u>          </u>
8.	<b><u>Nature of Failure or Error:</u></b>	<b><u>8.3</u></b>	<u>          </u>	<u>          </u>
9.	<b><u>Nature of Recovery Actions:</u></b>	<b><u>9.1</u></b>	<u>          </u>	<u>          </u>

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>2/9/2002</b>	<b>DATE RECEIVED</b>
<b>EVENT TITLE</b>			
NRC INFORMATION NOTICE 2002-10, SUPPLEMENT 1:	DIABLO CANYON MANUAL REACTOR TRIP AND STEAM GENERATOR WATER LEVEL SETPOINT UNCERTAINTIES		
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	Diablo Canyon Unit 2	PWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
Westinghouse	March 13, 1986		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this supplement to give addressees further information about the manual reactor trip of Diablo Canyon Unit 2 which resulted from a failure of the main feedwater regulating valve, non-conservative steam generator setpoints and contributing causes, and other licensee actions relating to these events.

NRC INFORMATION NOTICE 2002-10, SUPPLEMENT 1

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1. <u>Reporting Categories:</u>	<u>1.3.1</u>	_____	_____
2. <u>Plant Status Prior to the Event:</u>	<u>2.1.1</u>	_____	_____
3. <u>Failed/Affected Systems:</u>	<u>3.IN</u>	_____	_____
4. <u>Failed/Affected Components:</u>	<u>4.1.3</u>	_____	_____
5. <u>Cause of the Event:</u>	<u>5.1.10.2</u>	<u>5.6.2</u>	<u>5.7.1</u>
6. <u>Effects on Operation:</u>	<u>6.1.2</u>	_____	_____
7. <u>Characteristics of the Incident:</u>	<u>7.13</u>	_____	_____
8. <u>Nature of Failure or Error:</u>	<u>8.2</u>	<u>8.4</u>	_____
9. <u>Nature of Recovery Actions:</u>	<u>9.1</u>	_____	_____

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>11/11/2001</b>	<b>DATE RECEIVED</b>
NRC INFORMATION NOTICE 2002-12:		<b>EVENT TITLE</b> SUBMERGED SAFETY-RELATED ELECTRICAL CABLES	
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> Oyster Creek Nuclear Power Plant	<b>REACTOR TYPE</b> BWR/2	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b> N/A		
<b>DESIGNER</b> GE	<b>1st COMMERCIAL OPERATION</b> December 1, 1969		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of observed protracted submergence in water of electrical cables that feed safety related equipment. The licensee concluded that the cable failure resulted from a localized delamination of the cable jacket aggravated by water intrusion into the underground cable conduit, subsequent cable drying, and corona degradation of the insulation.

NRC INFORMATION NOTICE 2002-12

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |  |                |       |       |
|--|----------------|-------|-------|
| 1. <u>Reporting Categories:</u>            | <u>1.4</u>     | _____ | _____ |
| 2. <u>Plant Status Prior to the Event:</u> | <u>2.1.1</u>   | _____ | _____ |
| 3. <u>Failed/Affected Systems:</u>         | <u>3.EB</u>    | _____ | _____ |
| 4. <u>Failed/Affected Components:</u>      | <u>4.3.8</u>   | _____ | _____ |
| 5. <u>Cause of the Event:</u>              | <u>5.1.2.8</u> | _____ | _____ |
| 6. <u>Effects on Operation:</u>            | <u>6.2</u>     | _____ | _____ |
| 7. <u>Characteristics of the Incident:</u> | <u>7.5</u>     | _____ | _____ |
| 8. <u>Nature of Failure or Error:</u>      | <u>8.3</u>     | _____ | _____ |
| 9. <u>Nature of Recovery Actions:</u>      | <u>9.0</u>     | _____ | _____ |

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>6/6/2002</b>	<b>DATE RECEIVED</b>
NRC INFORMATION NOTICE 2002-18:		<b>EVENT TITLE</b> EFFECT OF ADDING GAS INTO WATER STORAGE TANKS ON THE NET POSITIVE SUCTION HEAD FOR PUMPS	
<b>COUNTRY</b> USA	<b>PLANT AND UNIT</b> Callaway Unit 1	<b>REACTOR TYPE</b> PWR	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b> N/A		
<b>DESIGNER</b> Westinghouse	<b>1st COMMERCIAL OPERATION</b> December 19, 1984		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to alert addressees to the potential impact of adding gas into water storage tanks on the available net positive suction head (NPSH) for pumps. The NRC recently performed an inspection at the Union Electric Company's Callaway Nuclear Power Plant in response to the identification of a degraded condensate storage tank (CST) floating diaphragm seal. The degradation resulted in a failure of a motor-driven auxiliary feedwater (AFW) pump. It also had the potential to induce a common-cause failure of multiple AFW pumps. The inspection included a review of several calculations related to the available NPSH for AFW pumps.

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |  |                |                   |                   |
|--|----------------|-------------------|-------------------|
| 1. <u>Reporting Categories:</u>            | <u>1.3.1</u>   | <u>1.4</u>        | <u>          </u> |
| 2. <u>Plant Status Prior to the Event:</u> | <u>2.0</u>     | <u>          </u> | <u>          </u> |
| 3. <u>Failed/Affected Systems:</u>         | <u>3.B.B</u>   | <u>          </u> | <u>          </u> |
| 4. <u>Failed/Affected Components:</u>      | <u>4.2.1</u>   | <u>          </u> | <u>          </u> |
| 5. <u>Cause of the Event:</u>              | <u>5.1.4.1</u> | <u>          </u> | <u>          </u> |
| 6. <u>Effects on Operation:</u>            | <u>6.0</u>     | <u>          </u> | <u>          </u> |
| 7. <u>Characteristics of the Incident:</u> | <u>7.5</u>     | <u>          </u> | <u>          </u> |
| 8. <u>Nature of Failure or Error:</u>      | <u>8.3</u>     | <u>          </u> | <u>          </u> |
| 9. <u>Nature of Recovery Actions:</u>      | <u>9.0</u>     | <u>          </u> | <u>          </u> |

# INCIDENT REPORTING SYSTEM

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**IRS NO.**      **EVENT DATE**      **5/1/2002**      **DATE RECEIVED**

**EVENT TITLE**

NRC INFORMATION NOTICE 2002-21: AXIAL OUTSIDE-DIAMETER CRACKING AFFECTING  
THERMALLY TREATED ALLOY 600 STEAM  
GENERATOR TUBING

**COUNTRY**

USA

**PLANT AND UNIT**

Seabrook Unit 1

**REACTOR TYPE**

PWR

**INITIAL STATUS**

**RATED POWER (MWe NET)**

N/A

**DESIGNER**

Westinghouse

**1st COMMERCIAL OPERATION**

August 17, 1990

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**ABSTRACT**

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of preliminary indications of axial outside-diameter (OD) cracking of thermally treated Alloy 600 steam generator (SG) tubing at Seabrook.

NRC INFORMATION NOTICE 2002-21

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1. <u>Reporting Categories:</u>	<u>1.2.5</u>	<u>1.4</u>	<u>          </u>
2. <u>Plant Status Prior to the Event:</u>	<u>2.1</u>	<u>2.3</u>	<u>          </u>
3. <u>Failed/Affected Systems:</u>	<u>3.AH</u>	<u>          </u>	<u>          </u>
4. <u>Failed/Affected Components:</u>	<u>4.2.4</u>	<u>          </u>	<u>          </u>
5. <u>Cause of the Event:</u>	<u>5.1.1</u>	<u>          </u>	<u>          </u>
6. <u>Effects on Operation:</u>	<u>6.0</u>	<u>          </u>	<u>          </u>
7. <u>Characteristics of the Incident:</u>	<u>7.5</u>	<u>          </u>	<u>          </u>
8. <u>Nature of Failure or Error:</u>	<u>8.3</u>	<u>          </u>	<u>          </u>
9. <u>Nature of Recovery Actions:</u>	<u>9.0</u>	<u>          </u>	<u>          </u>

# INCIDENT REPORTING SYSTEM

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**IRS NO.**            **EVENT DATE**    **4/23/2001**    **DATE RECEIVED**

**EVENT TITLE**  
NRC INFORMATION NOTICE 2002-22: DEGRADED BEARING SURFACES IN GM/EMD  
EMERGENCY DIESEL GENERATORS

**COUNTRY**

USA

**PLANT AND UNIT**

Surry Power Station, Unit 1

**REACTOR TYPE**

PWR

**INITIAL STATUS**

**RATED POWER (MWe NET)**

N/A

**DESIGNER**

Westinghouse

**1st COMMERCIAL OPERATION**

December 22, 1972

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**ABSTRACT**

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice to inform addressees of the discovery of degraded bearing surfaces on the piston bearings in General Motors/Electromotive Division (GM/EMD) emergency diesel generator (EDG) engines.

NRC INFORMATION NOTICE 2002-22

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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- |    |   |                |       |       |
|----|---|----------------|-------|-------|
| 1. | <u>Reporting Categories:</u>            | <u>1.4</u>     | _____ | _____ |
| 2. | <u>Plant Status Prior to the Event:</u> | <u>2.0</u>     | _____ | _____ |
| 3. | <u>Failed/Affected Systems:</u>         | <u>3.EF</u>    | _____ | _____ |
| 4. | <u>Failed/Affected Components:</u>      | <u>4.2.7</u>   | _____ | _____ |
| 5. | <u>Cause of the Event:</u>              | <u>5.1.1.2</u> | _____ | _____ |
| 6. | <u>Effects on Operation:</u>            | <u>6.0</u>     | _____ | _____ |
| 7. | <u>Characteristics of the Incident:</u> | <u>7.5</u>     | _____ | _____ |
| 8. | <u>Nature of Failure or Error:</u>      | <u>8.3</u>     | _____ | _____ |
| 9. | <u>Nature of Recovery Actions:</u>      | <u>9.0</u>     | _____ | _____ |

# INCIDENT REPORTING SYSTEM

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<b>IRS NO.</b>	<b>EVENT DATE</b>	<b>7/19/2002</b>	<b>DATE RECEIVED</b>
		<b>EVENT TITLE</b>	
NRC INFORMATION NOTICE 2002-24:		POTENTIAL PROBLEMS WITH HEAT COLLECTORS ON FIRE PROTECTION SPRINKLERS	
<b>COUNTRY</b>	<b>PLANT AND UNIT</b>	<b>REACTOR TYPE</b>	
USA	N/A	N/A	
<b>INITIAL STATUS</b>	<b>RATED POWER (MWe NET)</b>		
	N/A		
<b>DESIGNER</b>	<b>1st COMMERCIAL OPERATION</b>		
N/A	N/A		

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## ABSTRACT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this information notice (IN) to alert licensees to potential problems with using heat collectors on sprinklers and fire detectors installed to satisfy NRC fire protection requirements.

NRC INFORMATION NOTICE 2002-24

Please refer to the dictionary of codes corresponding to each of the sections below and to the coding guidelines manual.

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1. <u>Reporting Categories:</u>	<u>1.3.1</u>	<u>1.4</u>	<u>                    </u>
2. <u>Plant Status Prior to the Event:</u>	<u>2.0</u>	<u>                    </u>	<u>                    </u>
3. <u>Failed/Affected Systems:</u>	<u>3.KH</u>	<u>                    </u>	<u>                    </u>
4. <u>Failed/Affected Components:</u>	<u>4.1.11</u>	<u>                    </u>	<u>                    </u>
5. <u>Cause of the Event:</u>	<u>5.1.5.2</u>	<u>                    </u>	<u>                    </u>
6. <u>Effects on Operation:</u>	<u>6.0</u>	<u>                    </u>	<u>                    </u>
7. <u>Characteristics of the Incident:</u>	<u>7.5</u>	<u>                    </u>	<u>                    </u>
8. <u>Nature of Failure or Error:</u>	<u>8.3</u>	<u>                    </u>	<u>                    </u>
9. <u>Nature of Recovery Actions:</u>	<u>9.0</u>	<u>                    </u>	<u>                    </u>