

March 6, 1991

Docket Nos. 50-275
and 50-323

Mr. J. D. Shiffer
Nuclear Power Generation
Senior Vice President
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, CA 94106

Dear Mr. Shiffer:

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SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 77904 AND 77905)

The Commission has issued the enclosed Amendment No. 59 to Facility Operating License No. DPR-80 and Amendment No. 58 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated September 11, 1990.

These amendments revise the combined Technical Specifications (TS) for the Diablo Canyon Power Plant (DCPP) Unit Nos. 1 and 2 to incorporate changes to management titles, clarify the roles of Shift Supervisor and Shift Foreman, provide additional flexibility for the Shift Technical Advisor (STA) position, and correct an erroneous value of the minimum contained borated water volume of the Refueling Water Storage Tank (RWST).

A copy of the related Safety Evaluation is enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

original signed by

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000275
P PDR

Enclosures:

1. Amendment No. 59 to License No. DPR-80
2. Amendment No. 58 to License No. DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

OFC	:LA/PD5/DRPW	:PM/PD5/DRPW	:BC/LPEB/DLPQ	:OGC	:D/PD5/DRPW	:
NAME	:DFoster <i>DFoster</i>	:HRood: <i>HR</i>	:WBrach	: <i>DF</i>	:JDyer <i>JWC</i>	:
DATE	:02/14/91	:02/14/91	:02/14/91	:02/14/91	: 3/1/91	:

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Mr. J. D. Shiffer
Pacific Gas and Electric Company

Diablo Canyon

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1
DOCKET NO. 50-275
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-80

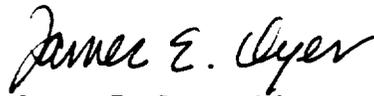
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated September 11, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 59 , are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 6, 1991



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2
DOCKET NO. 50-323
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated September 11, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 58 , are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 6, 1991

ATTACHMENT TO LICENSE AMENDMENT NOS. 59 AND 58

FACILITY OPERATING LICENSE NOS. DPR-80 AND DPR-82

DOCKET NOS. 50-273 AND 50-323

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages are also included, as appropriate.

REMOVE PAGE

3/4 1-13

6-1

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INSERT PAGE

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REACTIVITY CONTROL SYSTEMS

BORATED WATER SOURCES - OPERATING

LIMITING CONDITION FOR OPERATION

3.1.2.6 Each of the following borated water source(s) shall be OPERABLE:

For Unit 1 and 2, Cycle 4:

- a. A Boric Acid Storage System and at least one associated heat tracing channel with:
 - 1) A minimum contained borated water volume of 5106 gallons,
 - 2) A boron concentration between 20,000 and 22,500 ppm, and
 - 3) A minimum solution temperature of 145°F.
- b. The Refueling Water Storage Tank (RWST) with:
 - 1) A contained borated water volume of greater than or equal to 400,000 gallons,
 - 2) A boron concentration between 2300 and 2500 ppm, and
 - 3) A minimum solution temperature of 35°F, or

For Unit 1 and 2, Cycle 5 and after:

- a. A Boric Acid Storage System with:
 - 1) A minimum contained borated water volume of 14,042 gallons,
 - 2) A boron concentration between 7,000 and 7,700 ppm, and
 - 3) A minimum solution temperature of 65°F.
- b. The Refueling Water Storage Tank (RWST) with:
 - 1) A contained borated water volume of greater than or equal to 400,000 gallons,
 - 2) A boron concentration between 2300 and 2500 ppm, and
 - 3) A minimum solution temperature of 35°F.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With the Boric Acid Storage System inoperable, restore the system to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and borated to a SHUTDOWN MARGIN equivalent to at least 1% $\Delta k/k$ at 200°F; restore the Boric Acid Storage System to OPERABLE status within the next 7 days or be in COLD SHUTDOWN within the next 30 hours.
- b. With the RWST inoperable, restore the tank to OPERABLE status within 1 hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

REACTIVITY CONTROL SYSTEMS

SURVEILLANCE REQUIREMENTS

- 4.1.2.6 Each borated water source shall be demonstrated OPERABLE:
- a. At least once per 7 days by:
 - 1) Verifying the boron concentration in the water,
 - 2) Verifying the contained borated water volume of the water source, and
 - 3) Verifying the Boric Acid Storage System solution temperature.
 - b. At least once per 24 hours by verifying the RWST temperature when the outside air temperature is less than 35°F.

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Vice President, Diablo Canyon Operations and Plant Manager, hereinafter called Plant Manager, shall be responsible for overall plant operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Foreman (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room Command function. A management directive to this effect signed by the Senior Vice President and General Manager, Nuclear Power Generation shall be reissued to all plant personnel on an annual basis.

6.2 ORGANIZATION

6.2.1 OFFSITE AND ONSITE ORGANIZATIONS

Onsite and offsite organizations shall be established for plant operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR Update.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Senior Vice President and General Manager - Nuclear Power Generation shall have corporate responsibilities for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 PLANT STAFF

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODE 1, 2, 3 or 4, at least one licensed Senior Operator shall be in the Control Room;
- c. A Health Physics Technician* shall be on site when fuel is in the reactor;

*The Health Physics Technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

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TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION		
	BOTH UNITS IN MODE 1, 2, 3, OR 4	BOTH UNITS IN MODE 5 OR 6 OR DEFUELED	ONE UNIT IN MODE 1, 2, 3 OR 4 AND ONE UNIT IN MODE 5 OR 6 OR DEFUELED
SFM	1	1	1
SOL	1	none ^b	1
OL	3 ^a	2 ^a	3 ^a
AO	3 ^a	3 ^a	3 ^a
STA	1*	none	1*

- SFM - Shift Foreman with a Senior Operator License
- SOL - Individual with a Senior Operator License
- OL - Individual with an Operator License
- AO - Auxiliary Operator
- STA - Shift Technical Advisor

The Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Foreman from the control room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid Senior Operator license shall be designated to assume the control room command function. During any absence of the Shift Foreman from the control room while the unit is in MODE 5 or 6, an individual with a valid Senior Operator or Operator license shall be designated to assume the control room command function.

^{a/} At least one of the required individuals must be assigned to the designated position for each unit.

^{b/} At least one licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling must be present during Core Alterations on either unit, who has no other concurrent responsibilities.

*The STA position shall be manned in MODES 1, 2, 3, and 4 unless an individual with a Senior Operator license meets the Commission Policy Statement on Engineering Expertise on Shift.

ADMINISTRATIVE CONTROLS

PLANT STAFF (Continued)

- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation;
- e. A site Fire Brigade of at least five members* shall be maintained onsite at all times. The Fire Brigade shall not include the Shift Foreman and the two other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency;
- f. Administrative procedures shall be developed and implemented to limit the working hours of plant staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, Health Physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a nominal 40-hour week while the unit is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modifications, on a temporary basis, the following guidelines shall be followed:

- 1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time;
- 2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time;
- 3) A break of at least 8 hours should be allowed between work periods, including shift turnover time; and
- 4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or his designee, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized; and

- g. The Operations Manager shall hold a senior reactor operator license.

*The Health Physics Technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

6.2.3 ONSITE SAFETY REVIEW GROUP (OSRG)

FUNCTION

6.2.3.1 The OSRG shall function to examine plant operating characteristics, NRC issuances, industry advisories, REPORTABLE EVENTS and other sources of plant design and operating experience information, including plants of similar design, which may indicate areas for improving plant safety. The OSRG shall make detailed recommendations for revised procedures, equipment modifications, maintenance activities, operations activities or other means of improving plant safety to the Manager, Nuclear Safety Assessment and Regulatory Affairs.

COMPOSITION

6.2.3.2 The OSRG shall be composed of at least five engineers located on site.

RESPONSIBILITIES

6.2.3.3 The OSRG shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4 The Shift Technical Advisor shall provide advisory technical support to the Shift Foreman in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the plant. |

6.3 PLANT STAFF QUALIFICATIONS

6.3 Each member of the plant staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions, except for the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 2, April 1987 for Radiation Protection Manager. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of 10 CFR Part 55 and the supplemental requirements specified in Section A of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

6.5.2.6 The Plant Staff Review Committee shall be responsible for:

- a. Review of: (1) Administrative procedures, Security Plan implementing procedures, Emergency Plan implementing procedures, and changes thereto; (2) the safety evaluations for: (a) changes to procedures and (b) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question and (3) proposed procedures or changes thereto that have been initially determined to constitute an unreviewed safety question or require a change to the Technical Specifications;
- b. Review of all proposed changes to Appendix "A" Technical Specifications;
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
- d. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Senior Vice President and General Manager, Nuclear Power Generation and to the Chairman of the General Office Nuclear Plant Review and Audit Committee; the investigation shall include an assessment of the safety significance of each violation.
- e. Review of all REPORTABLE EVENTS;
- f. Review of plant operations to detect potential nuclear safety hazards;
- g. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the General Office Nuclear Plant Review and Audit Committee;
- h. Review of the Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the General Office Nuclear Plant Review and Audit Committee or the Plant Manager, as appropriate;
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the General Office Nuclear Plant Review and Audit Committee or the Plant Manager, as appropriate;
- j. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence to the Senior Vice President and General Manager, Nuclear Power Generation and to GONPRAC; and
- k. Review of changes to the PROCESS CONTROL PROGRAM, ODCP, ERMP, and the Radwaste Treatment Systems.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

6.5.2.7 The Plant Staff Review Committee shall:

- a. Recommend to the Plant Manager written approval or disapproval of items considered under Specification 6.5.2.6a. through d. above;
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.2.6a. through e. above constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Senior Vice President and General Manager, Nuclear Power Generation and the General Office Nuclear Plant Review and Audit Committee of disagreement between the PSRC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.2.8 The Plant Staff Review Committee shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Senior Vice President and General Manager, Nuclear Power Generation and the General Office Nuclear Plant Review and Audit Committee.

6.5.3 GENERAL OFFICE NUCLEAR PLANT REVIEW AND AUDIT COMMITTEE (GONPRAC)

FUNCTION

6.5.3.1 The General Office Nuclear Plant Review and Audit Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

GONPRAC shall report to and advise the Senior Vice President and General Manager, Nuclear Power Generation on those areas of responsibility specified in Specifications 6.5.3.7 and 6.5.3.8.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.3.2 GONPRAC shall be composed of the following:

Chairman:	Manager, Nuclear Safety Assessment and Regulatory Affairs
Vice Chairman:	Manager, Nuclear Operations Support
Member:	Assistant to the Senior Vice President and General Manager, Nuclear Power Generation
Member:	Manager, Nuclear Engineering and Construction Services
Member:	Manager, Quality Assurance
Member:	Chief Mechanical and Nuclear Engineer
Member:	Manager, Station Construction
Member:	Director, Nuclear Regulatory Affairs
Member:	Plant Manager, Diablo Canyon Power Plant

ALTERNATES

6.5.3.3 All alternate members shall be appointed in writing by the GONPRAC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in GONPRAC activities at any one time.

CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the GONPRAC Chairman to provide expert advice to GONPRAC.

MEETING FREQUENCY

6.5.3.5 GONPRAC shall meet at least once per calendar quarter during the initial year of plant operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.3.6 A quorum of GONPRAC necessary for the performance of the GONPRAC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least four GONPRAC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the plant.

REVIEW

6.5.3.7 GONPRAC shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components that could affect nuclear safety; and
- i. Reports and meetings minutes of the Plant Staff Review Committee and the Onsite Safety Review Group.

AUDITS

- 6.5.3.8 Audits of plant activities shall be performed under the cognizance of GONPRAC. These audits shall encompass:
- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
 - b. The performance, training and qualifications of the entire plant staff at least once per 12 months;
 - c. The results of actions taken to correct deficiencies occurring in plant equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months;
 - d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
 - e. The fire protection program and implementing procedures at least once per 24 months by qualified personnel;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. Any other area of plant operation considered appropriate by GONPRAC or the Senior Vice President and General Manager, Nuclear Power Generation;
- h. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- i. The ODCP and ERMP and implementing procedures at least once per 24 months;
- j. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months; and
- k. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

RECORDS

6.5.3.9 Records of GONPRAC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each GONPRAC meeting shall be prepared, approved and forwarded to the Senior Vice President and General Manager, Nuclear Power Generation within 14 working days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.3.7 above, shall be prepared, approved and forwarded to the Senior Vice President and General Manager, Nuclear Power Generation within 14 working days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.3.8 above, shall be forwarded to the Senior Vice President and General Manager, Nuclear Power Generation and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6. The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.73; and

ADMINISTRATIVE CONTROLS

REPORTABLE EVENT ACTION (Continued)

- b. Each REPORTABLE EVENT shall be reviewed by the PSRC and the results of this review submitted to GONPRAC and the Senior Vice President and General Manager, Nuclear Power Generation.

6.7 SAFETY LIMIT VIOLATION

6.7 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Senior Vice President and General Manager, Nuclear Power Generation and GONPRAC shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PSRC. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon unit components, systems or structures, and (3) corrective action taken to prevent recurrence;
- c. The Safety Limit Violation Report shall be submitted to the Commission, GONPRAC and the Senior Vice President and General Manager, Nuclear Power Generation within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The emergency operating procedures required to implement the requirements of NUREG-0737 and Supplement 1 to NUREG-0737 as stated in Generic Letter No. 82-33;
- c. Security Plan implementation;
- d. Emergency Plan implementation;
- e. PROCESS CONTROL PROGRAM implementation;
- f. ODCP and ERMP implementation; and
- g. Quality Assurance Program for effluent and environmental monitoring.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, and all proposed tests and experiments that affect nuclear safety shall be approved prior to implementation by the Plant Manager or by a technically qualified manager who reports directly to the Plant Manager, as previously designated by the Plant Manager. Prior to approval, the Plant Manager or designated manager shall ensure that all necessary reviews and cross-disciplinary reviews, if appropriate, have been completed. If deemed necessary, such cross-disciplinary reviews shall be performed by the appropriate designated plant review personnel. Administrative procedures, procedures implementing the Security Plan, Emergency Plan, and Process Control Program, the ODCP and ERMP, and changes thereto, shall be reviewed by the PSRC and approved by the Plant Manager prior to implementation. Each procedure of Specification 6.8.1 above shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and
- c. The change is documented, reviewed and approved by the Plant Manager, or by a technically qualified manager who reports directly to the Plant Manager as previously designated by the Plant Manager, within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include portions of the Recirculation Spray System, Safety Injection System, Chemical and Volume Control System, Residual Heat Removal System, RCS Sample System, and Liquid and Gaseous Radwaste Treatment Systems. The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated September 11, 1990, Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the Technical Specifications appended to Facility Operating License Nos. DPR-80 and DPR-82 for operation of Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2. The amendments revise the combined Technical Specifications (TS) for the Diablo Canyon Power Plant (DCPP) Unit Nos. 1 and 2 to incorporate changes to management titles, clarify the roles of Shift Supervisor and Shift Foreman, provide additional flexibility for the Shift Technical Advisor (STA) position, and correct an erroneous value of the minimum contained borated water volume of the Refueling Water Storage Tank (RWST). Specifically, the amendments make the following changes in the TS:

- (1) TS 6.1, "Responsibility," TS 6.2, "Organization," TS 6.5, "Review and Audit," TS 6.6, "Reportable Event Action," and TS 6.7, "Safety Limit Violation," are revised to change all references from Vice President, Nuclear Power Generation, to Senior Vice President and General Manager, Nuclear Power Generation.
- (2) TS 6.1, "Responsibility," is revised to change the initial reference from Plant Manager to Vice President, Diablo Canyon Operations and Plant Manager. All subsequent references to Plant Manager shall be understood to mean Vice President, Diablo Canyon Operations and Plant Manager.
- (3) TS 6.1, "Responsibility," and TS 6.2, "Organization," is revised to change all references from Shift Supervisor to Shift Foreman.
- (4) TS 6.2, "Organization," Table 6.2-1, "Minimum Shift Crew Composition," footnote designated *, is changed to state that an individual with a Senior Operator License who meets the staff criteria for engineering expertise on shift may fill the STA position in Operating Modes 1, 2, 3, and 4.
- (5) TS 3.1.2.6, "Borated Water Sources - Operating," is revised to correct an administrative error. Previously, this TS required, for Units 1 and 2, Cycle 5 and after, that the RWST have a minimum contained borated water volume of 50,000 gallons. These amendments change this to the correct value of 400,000 gallons.

2.0 EVALUATION

The licensee states in its September 11, 1990 application that changes (1) and (2), above are administrative in nature, and reflect organizational changes in the licensee's Nuclear Power Generation Business Unit. Specifically, the management title changes (1) from Vice President, Nuclear Power Generation, to Senior Vice President and General Manager, Nuclear Power Generation, and (2) from Plant Manager to Vice President, Diablo Canyon Operations and Plant Manager, are a part of the licensee's continuing effort to improve the effectiveness, efficiency, and safety of Diablo Canyon operations, and are purely administrative. On this basis, the NRC staff finds these changes to be acceptable.

The licensee indicates that change (3), above, in which the title "Shift Supervisor" in TS 6.2.2.a is changed to "Shift Foreman" will clarify the intended roles of each position at DCCP, but will not change intended functions of either position. Based on this, the NRC staff finds this change to be acceptable.

The licensee states that change (4), above, revises the STA manning requirements and allows the licensee additional flexibility in shift manning, while meeting the NRC qualification criteria for the STA position. Because the NRC criteria for maintaining engineering expertise on shift will be met, this change is acceptable to the NRC staff.

Change (5), above, corrects an error in the minimum volume of borated water required to be maintained in the refueling water storage tank (RWST). This change, from 50,000 gallons to 400,000 gallons is acceptable to the NRC staff because it requires the facility to meet the design basis requirements for borated water.

In summary, the NRC staff has reviewed the licensee's basis for the proposed changes to the Combined Technical Specifications for Diablo Canyon Units 1 and 2 and finds them acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve (1) changes with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change a surveillance requirement, and (2) changes in administrative procedures or requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission has proposed to determine that these amendments involve no significant hazards consideration. The proposed determination was published in the Federal Register on November 14, 1990 at 55 FR 47575. No public comments or requests for hearing were received.

The NRC staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Rood

Dated: March 6, 1991