

Docket Nos. 50-275
and 50-323

February 26, 1990

Mr. J. D. Shiffer, Vice President
Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, California 94106

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HRood	TEssig
OC/LFMB	JWigginton
GPA/PA	LCunningham

Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 75624 AND 75625)

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. DPR-80 and Amendment No. 49 to Facility Operating License No. DPR-82 for the Diablo Canyon Power Plant (DCPP), Units 1 and 2, respectively. The amendments change the Diablo Canyon combined Technical Specifications (TS) in response to your application for license amendments dated December 20, 1989 as supplemented by letter dated January 30, 1990 (Reference LAR 89-15). The amendments revise the TS to increase the alarm/trip setpoint of the spent fuel pool (SFP) storage area radiation monitor (RM-58). Specifically, the amendments revise TS Table 3.3-6, "Radiation Monitoring Instrumentation for Plant Operations," to increase the alarm/trip setpoint of RM-58 from 15 to 75 mR/hr.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Also enclosed is page B 3/4 3-5 of the Technical Specification Bases. This page was revised in Amendment 42 and 41 and, through administrative error, Bases Section 3/4.3.3.10, line 14 was incorrect. It should read " μ Ci/ml are measurable."

Sincerely,

ORIGINAL SIGNED BY HARRY ROOD

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PDR ADOCK 05000275
P PDR

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to DPR-80
2. Amendment No. 49 to DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

DRSP/LA:PD5	DRSP/PM:PD5	OGC	DRSP/D(A):PD5
PShea	HRood*	CPW*	CTramme11*
02/14/90	02/13/90	02/15/90	02/20/90

*See previous concurrence

Handwritten signature and initials

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PShea*	HRood*	CPW*	CTrammell*
02/14/90	02/13/90	02/15/90	02/20/90

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SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 75624 AND 75625)

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. DPR-80 and Amendment No. 48 to Facility Operating License No. DPR-82 for the Diablo Canyon Power Plant (DCPP), Units 1 and 2, respectively. The amendments change the Diablo Canyon combined Technical Specifications (TS) in response to your application for license amendments dated December 20, 1989 as supplemented by letter dated January 30, 1990 (Reference LAR 89-15). The amendments revise the TS to increase the alarm/trip setpoint of the spent fuel pool (SFP) storage area radiation monitor (RM-58). Specifically, the amendments revise TS Table 3.3-6, "Radiation Monitoring Instrumentation for Plant Operations," to increase the alarm/trip setpoint of RM-58 from 15 to 75 mR/hr.

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Harry Rood, Senior Project Manager
Project Directorate V
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Nuclear Reactor Regulation

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2. Amendment No. 48 to DPR-82
3. Safety Evaluation

cc w/enclosures:

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PShea
02/11/90

DRSP/PM:PD5
HRood
02/13/90

OGC
CPW
02/15/90

DRSP/D(A):PD5
CTramme11
02/20/90



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 26, 1990

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Mr. J. D. Shiffer
Pacific Gas and Electric Company

Diablo Canyon

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-275
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated December 20, 1989, as supplemented by letter dated January 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 50, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Harry Rood, Acting Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 26, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2
DOCKET NO. 50-323
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 49
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated December 20, 1989, as supplemented by letter dated January 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 49, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Harry Rood, Acting Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 26, 1990

ATTACHMENT TO LICENSE AMENDMENT NOS. 50 AND 49
FACILITY OPERATING LICENSE NOS. DPR-80 and DPR-82
DOCKET NOS. 50-275 AND 50-323

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change. An overleaf page is also included.

Remove Page

3/4 3-37

Insert Page

3/4 3-37

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION FOR PLANT OPERATIONS

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ALARM/TRIP SETPOINT</u>	<u>ACTION</u>
1. Fuel Storage Area				
a. Spent Fuel Pool	1	*	≤ 75 mR/hr	30 & 32**
b. New Fuel Storage	1	*	≤ 15 mR/hr	30 & 32**
2. Control Room Ventilation Mode Change	2***	A11	≤ 2 mR/hr	34
3. Containment				
a. Gaseous Activity				
1) Containment Ventilation Isolation (RM-14A or 14B)	1	6	Per Specification 3.3.3.10	33
2) RCS Leakage	1	1, 2, 3, 4	N.A.	31
b. Particulate Activity				
RCS Leakage	1	1, 2, 3, 4	N.A.	31

*With fuel in the spent fuel pool or new fuel storage vault.

**With irradiated fuel in the spent fuel pool.

***One channel for each normal intake to the Control Room Ventilation System (common to both units).

TABLE 3.3-6 (Continued)

ACTION STATEMENTS

- ACTION 30 - With less than the Minimum Channels OPERABLE requirement, operation may continue for up to 30 days provided an appropriate portable continuous monitor with the same Alarm Setpoint or an individual qualified in radiation protection procedures with a radiation dose rate monitoring device is provided in the fuel storage pool area. Restore the inoperable monitors to OPERABLE status within 30 days or suspend all operations involving fuel movement in the fuel storage pool areas.
- ACTION 31 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.6.1.
- ACTION 32 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.12.
- ACTION 33 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 34 - With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirement, within 1 hour initiate and maintain operation of the Control Room Ventilation System in a recirculation mode with the HEPA filter and charcoal adsorber bank in operation.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 49 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NO. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated December 20, 1989, as supplemented by letter dated January 30, 1990 (Reference LAR 89-15), Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the combined Technical Specifications (TS) appended to Facility Operating License Nos. DPR-80 and DPR-82 for the Diablo Canyon Power Plant (DCPP), Unit Nos. 1 and 2, respectively. The amendments change the TS to increase the alarm/trip setpoint of the spent fuel pool (SFP) storage area radiation monitor (RM-58). Specifically, the amendments revise TS Table 3.3-6, "Radiation Monitoring Instrumentation for Plant Operations," to increase the alarm/trip setpoint for RM-58 from 15 to 75 mR/hr. The setpoint change is needed because the previous setpoint resulted in unnecessary safety system challenges during offload of the reactor core to the SFP.

The staff evaluation of these changes is given below and is based on the licensee's letters of December 20, 1989 and January 30, 1990. The information contained in the licensee's letter of January 30, 1990 did not change the action noticed in, or alter the staff's proposed determination of no significant hazards consideration published in the Federal Register on January 24, 1990 at 55 FR 2442.

2.0 EVALUATION

The NRC staff has evaluated the proposed change and finds it acceptable, based on the analyses and evaluations provided by the licensee. A discussion of the specific technical specification change made by these amendments and the basis for its acceptability is given below.

In its letter dated December 20, 1989, PG&E requested a change in the Diablo Canyon TS to increase the alarm/trip setpoint of spent fuel pool radiation monitor RM-58, from 15 mR/hr to 75 mR/hr. The 15 mR/hr setpoint of an identical monitor (RM-59) located in the fuel handling building adjacent to the new fuel storage area at the other end of the SFP would not be changed. These monitors are located at the side of the

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pool, about five feet above the surface of the water. They monitor radioactivity levels in the area and activate alarms in the control room and the SFP area if the alarm setpoint is exceeded, and actuate Engineered Safety Features (ESF) grade area ventilation and effluent air cleaning systems if the trip setpoint is exceeded. The fuel handling building ventilation system is shifted to the iodine removal mode if the trip setpoint of either monitor is exceeded.

During previous fuel movements associated with offloading the core, the licensee has experienced numerous false alarms from SFP monitor RM-58, when this monitor's setpoint of 15mR/hr has been exceeded slightly, apparently due to instrument drift and statistical fluctuations. Other instruments in the area, including portable instruments, have confirmed that the alarms were indeed false (see NRC Inspection Report 50-275/89-25 & 50-323/89-25, December 8, 1989).

The licensee requested a TS change to raise the alarm/trip setpoint of RM-58 to 75 mR/hr. PG&E has provided analyses to demonstrate that a design basis fuel handling accident would cause an ESF actuation by this monitor with the revised trip setting, thereby providing protection for the public. The 15 mR/hr trip level for the companion SFP monitor RM-59 would also be exceeded in the event of a fuel handling accident. With the revised RM-58 setpoint, a site boundary dose of only about 1.5 mR would be expected from the fuel handling accident. This is not significantly different from the expected fuel handling accident dose with the previous 15 mR/hr RM-58 setpoint.

In its letter dated January 30, 1990, the licensee also described its radiation protection program to assure that plant workers in the area are adequately protected. In the letter, the licensee committed to administratively control (by procedure) the alarm setpoint for RM-58 at about 20 mR/hr, rather than the 75 mR/hr allowed by the revised TS. This, in conjunction with providing portable, alarming radiation monitors, will adequately protect plant workers in the area. The ESF trip setpoint for RM-58 will be set at 75 mR/hr, as discussed above.

In summary, the staff finds that the licensee's request for a technical specification change to be reasonable, justified and acceptable, based on the analyses and evaluations provided by the licensee.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes to a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. At Diablo Canyon, the restricted area coincides with the site boundary. We have determined that the amendments involve no significant increase in the amounts, and no

significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of these amendments will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributors: James Martin
Harry Rood

Dated: February 26, 1990

INSTRUMENTATION

BASES

3/4.3.3.10 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluents during actual or potential releases of gaseous effluents. The Alarm/Trip Setpoints for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCP to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. This instrumentation also includes provisions for monitoring (and controlling) the concentrations of potentially explosive gas mixtures in the GASEOUS RADWASTE SYSTEM. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63 and 64 of Appendix A to 10 CFR Part 50. The sensitivity of any noble gas activity monitors used to show compliance with the gaseous effluent release requirements of Specification 3.11.2.2 shall be such that concentrations as low as 1×10^{-5} $\mu\text{Ci}/\text{m}^3$ are measurable.

3/4.3.4 TURBINE OVERSPEED PROTECTION

This specification is provided to ensure that the turbine overspeed protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety related components, equipment or structures.

The quarterly valve test frequency required by Specification 4.3.4.1.2a, is based on Diablo Canyon operating experience and the results of an evaluation documented in WCAP-11525, "Probabilistic Evaluation of Reduction in Turbine Valve Test Frequency," June 1987. The evaluation shows that for Diablo Canyon the probability of turbine missile generation is within the NRC acceptance criteria (letter from C. E. Rossi, USNRC, to J. A. Martin, Westinghouse, dated February 2, 1987) for turbine valve test intervals up to seven months.