

Docket Nos. 50-275
and 50-323

June 22, 1989

Mr. J. D. Shiffer, Vice President
Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, California 94106

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Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 72706 AND 72707)

The Commission has issued the enclosed Amendment No. 41 to Facility Operating License No. DPR-80 and Amendment No. 40 to Facility Operating License No. DPR-82 for the Diablo Canyon Power Plant (DCPP), Units 1 and 2, respectively. The amendments change the combined DCPP Technical Specifications (TS) in response to your application dated February 28, 1989, as supplemented by letter dated April 27, 1989 (Reference LAR 89-01).

The amendments revise Section 6, Administrative Controls, of the Diablo Canyon TS to change the description of the Plant Staff Review Committee membership, quorum, qualifications, and the review and approval of procedures, tests, and experiments.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/s/

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 41 to DPR-80
2. Amendment No. 40 to DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

DRSP/PD5	DRSP/PD	OGC	DRSP/D:PD5
JLee*	HRood*	*	GKnighton
06/09/89	06/08/89	06/13/89	06/22/89

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*See previous concurrence

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cc



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 22, 1989

Docket Nos. 50-275
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Mr. J. D. Shiffer, Vice President
Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
Pacific Gas and Electric Company
77 Beale Street, Room 1451
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A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Harry Rood".

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 41 to DPR-80
2. Amendment No. 40 to DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. J. D. Shiffer
Pacific Gas and Electric Company

Diablo Canyon

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-275
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated February 28, 1989, as supplemented by letter dated April 27, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 41, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1989



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2
DOCKET NO. 50-323
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated February 28, 1989, as supplemented by letter dated April 27, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 40, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1989

ATTACHMENT TO LICENSE AMENDMENT NOS. 41 AND 40
FACILITY OPERATING LICENSE NOS. DPR-80 and DPR-82
DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. Overleaf pages are also included, as appropriate.

<u>Remove Page</u>	<u>Insert Page</u>
6-7	6-7
---	6-7a
6-8	6-8
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-14	6-14

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4 A retraining and replacement training program for the plant staff shall be maintained under the direction of a designated member of the facility staff and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures, tests, and experiments that affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/group. Each such procedure, test, and experiment, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, test, or experiment, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, test, or experiment, or changes thereto.

6.5.1.2 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1 shall be previously designated by the Plant Manager to perform such reviews. Such designation shall include the disciplines or procedure categories for which each individual is qualified. Each individual designated to perform these reviews shall have at least 3 years of related experience and a baccalaureate degree in engineering or a related field, or shall have at least 8 years of related experience.

6.5.2 PLANT STAFF REVIEW COMMITTEE (PSRC)

FUNCTION

6.5.2.1 The Plant Staff Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.2.2 The PSRC shall be chaired by the Plant Manager and shall be composed of a minimum of 8 senior plant management individuals whose responsibilities include the functional areas of: operations, maintenance, radiation protection, support services, technical services, and quality control. All members shall be appointed in writing by the PSRC Chairman. The qualifications of each PSRC member shall meet or exceed the requirements and recommendations of Section 4.7 of ANSI/ANS 3.1-1978.

ALTERNATES

6.5.2.3 The Chairman may designate in writing other regular members who may serve as the Acting Chairman of PSRC meetings. All alternate members shall be appointed in writing by the PSRC Chairman. Alternates may be designated for specific PSRC members and shall have expertise in the same general area as the regular PSRC member they represent. No more than two alternates shall participate as voting members in PSRC activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.2.4 The PSRC shall meet at least once per calendar month and as convened by the PSRC Chairman or his designated alternate.

QUORUM

6.5.2.5 The minimum quorum of the PSRC necessary for the performance of the PSRC responsibility and authority provisions of these Technical Specifications shall be a majority (more than one-half) of the members of the PSRC. For purposes of the quorum, this majority shall include the Chairman or his designated alternate and no more than two alternate members.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES

6.5.2.6 The Plant Staff Review Committee shall be responsible for:

- a. Review of: (1) Administrative procedures, Security Plan implementing procedures, Emergency Plan implementing procedures, and changes thereto; (2) the safety evaluations for: (a) changes to procedures and (b) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question and (3) proposed procedures or changes thereto that have been initially determined to constitute an unreviewed safety question or require a change to the Technical Specifications;
- b. Review of all proposed changes to Appendix "A" Technical Specifications;
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
- d. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President, Nuclear Power Generation and to the Chairman of the General Office Nuclear Plant Review and Audit Committee; the investigation shall include an assessment of the safety significance of each violation.
- e. Review of all REPORTABLE EVENTS;
- f. Review of plant operations to detect potential nuclear safety hazards;
- g. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the General Office Nuclear Plant Review and Audit Committee;
- h. Review of the Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the General Office Nuclear Plant Review and Audit Committee or the Plant Manager, as appropriate;
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the General Office Nuclear Plant Review and Audit Committee or the Plant Manager, as appropriate;
- j. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence to the Vice President, Nuclear Power Generation and to GONPRAC; and
- k. Review of changes to the PROCESS CONTROL PROGRAM, ODCP, ERMP, and the Radwaste Treatment Systems.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

6.5.2.7 The Plant Staff Review Committee shall:

- a. Recommend to the Plant Manager written approval or disapproval of items considered under Specification 6.5.2.6a. through d. above;
- b. Render determinations in writing with regard to whether or not each item considered under Specification 6.5.2.6a. through e. above constitutes an unreviewed safety question; and
- c. Provide written notification within 24 hours to the Vice President, Nuclear Power Generation and the General Office Nuclear Plant Review and Audit Committee of disagreement between the PSRC and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.2.8 The Plant Staff Review Committee shall maintain written minutes of each PSRC meeting that, at a minimum, document the results of all PSRC activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Vice President, Nuclear Power Generation and the General Office Nuclear Plant Review and Audit Committee.

6.5.3 GENERAL OFFICE NUCLEAR PLANT REVIEW AND AUDIT COMMITTEE (GONPRAC)

FUNCTION

6.5.3.1 The General Office Nuclear Plant Review and Audit Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Quality assurance practices.

GONPRAC shall report to and advise the President on those areas of responsibility specified in Specifications 6.5.3.7 and 6.5.3.8.

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.3.2 GONPRAC shall be composed of the following:

Chairman:	Vice President, Nuclear Power Generation
Vice Chairman:	Assistant to the Vice President, Nuclear Power Generation
Member:	Manager, Nuclear Engineering and Construction Services
Member:	Manager, Quality Assurance
Member:	Manager, Nuclear Operations Support
Member:	Chief Mechanical and Nuclear Engineer
Member:	Manager, Station Construction
Member:	Director, Nuclear Regulatory Affairs
Member:	Director, Nuclear Administration and Support Services

ALTERNATES

6.5.3.3 All alternate members shall be appointed in writing by the GONPRAC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in GONPRAC activities at any one time.

CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the GONPRAC Chairman to provide expert advice to GONPRAC.

MEETING FREQUENCY

6.5.3.5 GONPRAC shall meet at least once per calendar quarter during the initial year of plant operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.3.6 A quorum of GONPRAC necessary for the performance of the GONPRAC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least four GONPRAC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the plant.

REVIEW

6.5.3.7 GONPRAC shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment or systems, and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;

ADMINISTRATIVE CONTROLS

REVIEW (Continued)

- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components that could affect nuclear safety; and
- i. Reports and meetings minutes of the Plant Staff Review Committee and the Onsite Safety Review Group.

AUDITS

- 6.5.3.8 Audits of plant activities shall be performed under the cognizance of GONPRAC. These audits shall encompass:
- a. The conformance of plant operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months;
 - b. The performance, training and qualifications of the entire plant staff at least once per 12 months;
 - c. The results of actions taken to correct deficiencies occurring in plant equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months;
 - d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months;
 - e. The fire protection program and implementing procedures at least once per 24 months by qualified personnel;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- f. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- g. Any other area of plant operation considered appropriate by GONPRAC or the President;
- h. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- i. The ODCP and ERMP and implementing procedures at least once per 24 months;
- j. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months; and
- k. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

RECORDS

6.5.3.9 Records of GONPRAC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each GONPRAC meeting shall be prepared, approved and forwarded to the President within 14 working days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.3.7 above, shall be prepared, approved and forwarded to the President within 14 working days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.3.8 above, shall be forwarded to the President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE EVENT ACTION

6.6. The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of 10 CFR 50.73; and

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, and all proposed tests and experiments that affect nuclear safety shall be approved prior to implementation by the Plant Manager or by a technically qualified manager who reports directly to the Plant Manager, as previously designated by the Plant Manager. Prior to approval, the Plant Manager or designated manager shall ensure that all necessary reviews and cross-disciplinary reviews, if appropriate, have been completed. If deemed necessary, such cross-disciplinary reviews shall be performed by the appropriate designated plant review personnel. Administrative procedures, procedures implementing the Security Plan, Emergency Plan, and Process Control Program, the ODCP and ERMP, and changes thereto, shall be reviewed by the PSRC and approved by the Plant Manager prior to implementation. Each procedure of Specification 6.8.1 above shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and
- c. The change is documented, reviewed and approved by the Plant Manager, or by a technically qualified manager who reports directly to the Plant Manager as previously designated by the Plant Manager, within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include portions of the Recirculation Spray System, Safety Injection System, Chemical and Volume Control System, Residual Heat Removal System, RCS Sample System, and Liquid and Gaseous Radwaste Treatment Systems. The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NO. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated February 28, 1989, as supplemented by letter dated April 27, 1989 (Reference LAR 89-01), Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the combined Technical Specifications (TS) appended to Facility Operating License Nos. DPR-80 and DPR-82 for the Diablo Canyon Power Plant (DCPP), Unit Nos. 1 and 2, respectively. The amendments revise Section 6, Administrative Controls, of the DCPP TS to change the requirements for Plant Staff Review Committee (PSRC) membership, quorum, qualifications, and for the review and approval of procedures, tests, and experiments.

2.0 EVALUATION

The NRC staff has reviewed the TS changes proposed by the licensee and finds them acceptable, based on the following evaluation:

- (a) TS Section 6.5.1.2 - The licensee requested that the composition of the PSRC be redefined by deleting the specific titles of members and stating that the composition shall be chaired by the Plant Manager, composed of a minimum of eight senior plant management individuals with responsibility in the areas of operations, maintenance, radiation protection, support services, technical services, and quality assurance; and whose qualifications meet or exceed those described in Section 4.7 of ANSI/ANS 3.1-1978.

The NRC staff finds this requested change acceptable because it meets the appropriate acceptance criteria of Section 13.4 of NUREG 0800, the Standard Review Plan.

- (b) TS Section 6.5.2.3 - The licensee requested that the provisions for PSRC alternates be revised such that no more than two alternates shall participate as voting members, that alternates may be designated for specific PSRC members and shall have expertise in the same area as the regular PSRC member, and that the Chairman may designate in writing other regular members to serve as Acting Chairman.

The NRC staff finds this requested change acceptable because it is consistent with the provisions of the Standard Technical Specifications.

- (c) TS Section 6.5.1.6 - The licensee requested that the responsibilities of the PSRC be revised to delete the review of all procedures and procedure changes except administrative procedures, and proposed tests and experiments, and add the review of safety evaluations for those subjects.

The NRC staff finds this requested change acceptable because adequate provisions have been made for the review of procedures, and proposed tests and experiments (see item (d), below).

- (d) TS Section 6.5.1 Technical Review and Control - A new Section 6.5.1 has been added that describes the provisions for the review of procedures and procedure changes (except administrative procedures), and proposed tests and experiments. The review will be performed by a qualified individual/group other than the preparer who has at least three years of related experience and a baccalaureate degree in engineering or a related field or at least eight years of related experience. Provisions for multidisciplinary review, if necessary, are included.

The NRC staff finds this change acceptable because it constitutes additional and necessary restrictions on the review of procedures, tests, and experiments.

- (e) TS Section 6.8.2 - The licensee requested that this section (Procedures and Programs) be revised such that all procedures of TS 6.8.1 and all proposed tests and experiments that affect nuclear safety be approved prior to implementation by the Plant Manager or by a technically qualified manager who reports directly to the Plant Manager.

The NRC staff finds this requested change acceptable as it meets the appropriate acceptance criteria of Section 13.4 of NUREG 0800, the Standard Review Plan.

- (f) TS Section 6.8.3 - The licensee requested that the provisions for the review of temporary changes to procedures be revised such that they can be approved by a technically qualified manager who reports to the Plant Manager. Formerly all temporary procedures had to be approved by the Plant Manager.

The NRC staff finds requested change acceptable as it provides for the approval of temporary procedures by an adequately high level of plant management.

- (g) The licensee requested that several editorial changes be made that include the renumbering of some sections because of the addition of the new Section 6.5.1.

The NRC staff has reviewed these changes and found them acceptable because the changes are editorial.

Based on the above, the NRC staff finds acceptable these amendments authorizing revision of Section 6, Administrative Controls, of the DCPD TS to change the requirements for Plant Staff Review Committee membership, quorum, qualifications, and for the review and approval of procedures, tests, and experiments.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in administrative requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The NRC staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of these amendments will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributors: Frederick R. Allenspach
Harry Rood

Dated: June 22, 1989