

February 9, 1989

Docket Nos. 50-275  
and 50-323

Mr. J. D. Shiffer, Vice President  
Nuclear Power Generation  
c/o Nuclear Power Generation, Licensing  
Pacific Gas and Electric Company  
77 Beale Street, Room 1451  
San Francisco, California 94106

Dear Mr. Shiffer:

SUBJECT: CORRECTION TO SAFETY EVALUATION ASSOCIATED WITH AMENDMENTS  
INVOLVING SURVEILLANCE OF SEISMIC TRIP SYSTEM (TAC NO. 71169)

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By letter dated December 29, 1988, the Commission issued Amendment No. 33 to Facility Operating License No. DPR-80 and Amendment No. 32 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments changed the combined Technical Specifications for Diablo Canyon in response to your application dated November 10, 1988 (Reference LAR 88-07).

The purpose of this letter is to transmit a correction to the Safety Evaluation that accompanied the above amendments. Please replace page 1 of the Safety Evaluation with the enclosed page. The revised page states that the licensee is considering installation of circuitry providing channel and logic status indication in Unit 1 at the next refueling outage, and has committed to test the seismic trip devices at the next outage of sufficient length (a minimum of 24 hours). The revision does not affect the conclusions of the Safety Evaluation.

Sincerely,

original signed by

Harry Rood, Senior Project Manager  
Project Directorate V  
Division of Reactor Projects - III/IV/V  
and Special Projects  
Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation page 1

cc w/enclosure: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

February 9, 1989

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and 50-323

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Sincerely,

A handwritten signature in cursive script that reads "Harry Rood".

Harry Rood, Senior Project Manager  
Project Directorate V  
Division of Reactor Projects - III/IV/V  
and Special Projects  
Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation page 1

cc w/enclosure: See next page

Mr. J. D. Shiffer Pacific Gas and Electric Company

Diablo Canyon

cc:

Richard F. Locke, Esq.  
Pacific Gas & Electric Company  
Post Office Box 7442  
San Francisco, California 94120

NRC Resident Inspector  
Diablo Canyon Nuclear Power Plant  
c/o U.S. Nuclear Regulatory Commission  
P. O. Box 369  
Avila Beach, California 93424

Janice E. Kerr, Esq.  
California Public Utilities Commission  
350 McAllister Street  
San Francisco, California 94102

Mr. Dick Blakenburg  
Editor & Co-Publisher  
South County Publishing Company  
P. O. Box 460  
Arroyo Grande, California 93420

Ms. Sandra A. Silver  
660 Granite Creek Road  
Santa Cruz, California 95065

Bruce Norton, Esq.  
c/o Richard F. Locke, Esq.  
Pacific Gas and Electric Company  
Post Office Box 7442  
San Francisco, California 94120

Mr. W. C. Gangloff  
Westinghouse Electric Corporation  
P. O. Box 355  
Pittsburgh, Pennsylvania 15230

Dr. R. B. Ferguson  
Sierra Club - Santa Lucia Chapter  
Rocky Canyon Star Route  
Creston, California 93432

Managing Editor  
San Luis Obispo County Telegram  
Tribune  
1321 Johnson Avenue  
P. O. Box 112  
San Luis Obispo, California 93406

Chairman  
San Luis Obispo County Board of  
Supervisors  
Room 270  
County Government Center  
San Luis Obispo, California 93408

Mr. Leland M. Gustafson, Manager  
Federal Relations  
Pacific Gas and Electric Company  
1726 M Street, N. W.  
Washington, DC 20036-4502

Director  
Energy Facilities Siting Division  
Energy Resources Conservation and  
Development Commission  
1516 9th Street  
Sacramento, California 95814

Dian M. Grueneich  
Marcia Preston  
Law Office of Dian M. Grueneich  
380 Hayes Street, Suite 4  
San Francisco, California 94102

Ms. Jacquelyn Wheeler  
3033 Barranca Court  
San Luis Obispo, California 93401

Pacific Gas & Electric Company

- 2 -

Diablo Canyon

cc:

Ms. Laurie McDermott, Coordinator  
Consumers Organized for Defense  
of Environmental Safety  
731 Pacific Street, Suite 42  
San Luis Obispo, California 93401

Mr. Jack McGurk, Acting Chief  
Radiological Health Branch  
State Department of Health  
Services  
714 P Street, Office Building #8  
Sacramento, California 95814

Regional Administrator, Region V  
U.S. Nuclear Regulatory Commission  
1450 Maria Lane  
Suite 210  
Walnut Creek, California 94596

Ms. Nancy Culver  
192 Luneta Street  
San Luis Obispo, California 93401

President  
California Public Utilities  
Commission  
California State Building  
350 McAllister Street  
San Francisco, California 94102

Michael M. Strumwasser, Esq.  
Special Assistant Attorney General  
State of California  
Department of Justice  
3580 Wilshire Boulevard, Room 800  
Los Angeles, California 90010



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Revised 2/9/89

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 33 TO FACILITY OPERATING LICENSE NO. DPR-80  
AND AMENDMENT NO. 32 TO FACILITY OPERATING LICENSE NO. DPR-82  
PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2  
DOCKET NO. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated November 10, 1988 (Reference LAR 88-07), Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the combined Technical Specifications (TS) appended to Facility Operating License Nos. DPR-80 and DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2. The proposed amendments would exempt Unit 1 from performing operational testing of the seismic trip system until the next refueling outage, currently scheduled for October 1989.

2.0 DISCUSSION

During the last year, Diablo Canyon Unit 2 has tripped twice as a result of surveillance testing of the seismic trip system when one channel was in the tripped condition. The plant operators were not aware that one channel was tripped. Since the seismic trip system trips the reactor on two out of three logic, taking one channel out for testing when another channel is tripped results in a reactor trip. The licensee has implemented a corrective measure on Unit 2 by installing, at the last refueling outage, circuitry providing trip channel and logic status indication. The licensee is evaluating the installation of this circuitry in Unit 1 at the next refueling outage, and has committed to test the seismic trip devices at the next outage of sufficient length (a minimum of 24 hours). In the meantime, the licensee proposes that it be granted an exception to the requirement to test the trip devices every six months. This could result in the Unit 1 trip devices not being tested until the next refueling outage, which is currently scheduled for October 1989. As part of its justification for the proposed change, the licensee stated that the proposed status indication can not be installed at power.

The NRC staff has reviewed the licensee's proposed TS change and finds it acceptable, based on the following:

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