April 14, 1989

Docket Nos. 50-275
and 50-323

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Mr. J. D. Shiffer, Vice President
Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, California 94106

Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 64903 and 64904)

The Commission has issued the enclosed Amendment No. 35 to Facility Operating License No. DPR-80 and Amendment No. 34 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the combined Technical Specifications (TS) in response to your application dated March 13, 1987 (Reference LAR 87-03).

The amendments change the TS in three areas. These are:

1. TS 4.7.5.1, "Control Room Ventilation System," is revised to require that once per month, for each of the two control room ventilation system (CRVS) trains, one of the two redundant sets of booster and pressurization supply fans must be tested for 10 hours with the heaters operating. The present TS requires both sets of fans in each CRVS train to be tested monthly. The revised TS also requires that once per month, each main supply fan, booster fan, and pressurization fan must be tested for one hour to demonstrate fan operability. The present TS requires only the main supply fans to be tested for 15 minutes.

2. Table 3.3-1 of TS 3/4.3.1, "Reactor Trip System Instrumentation," is revised to clearly require a Quadrant Power Tilt Ratio determination to be made only if the plant is above 50 percent power. The previous wording of the TS could have, under some circumstances, required plant shutdown in situations when reduction to 50 percent power would be sufficient to maintain the departure from nucleate boiling (DNB) and linear heat generation within their limits.

3. Table 4.3-1 of TS 3/4.3.1 is revised to allow a plant heat balance to be performed, during startup, between 15 and 30 percent power. The previous TS could be interpreted to require the heat balance to be performed immediately upon reaching 15 percent power. This could result in a less accurate heat balance, and would unnecessarily constrain plant operators during startup.

DF01
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CP1
cc

A copy of the related Safety Evaluation is enclosed.

A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/S/

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III/IV/V
and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 35 to DPR-80
2. Amendment No. 34 to DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

DRSP/PD5	DRSP/PD5	SRXB	OGC	DRSP/PD5
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03/30/89	03/29/89	03/31/89	4/7/89	4/11/89

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-275

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated March 13, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 35, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III/IV/V
and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1989

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2

DOCKET NO. 50-323

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated March 13, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 34, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III/IV/V
and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1989

ATTACHMENT TO LICENSE AMENDMENT NOS. 35 AND 34

FACILITY OPERATING LICENSE NOS. DPR-80 and DPR-82

DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. Overleaf pages are provided for the inserted pages, as appropriate.

Remove Page	Insert Page
3/4 3-5	3/4 3-5
3/4 3-13	3/4 3-13
3/4 7-14	3/4 7-14

TABLE 3.3-1 (Continued)

TABLE NOTATIONS

*When the Reactor Trip System breakers are in the closed position and the Control Rod Drive System is capable of rod withdrawal.

#The provisions of Specification 3.0.4 are not applicable.

##Below the P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.

###Below the P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.

(1)These channels also provide inputs to ESFAS. Comply with the applicable MODES and ACTION statements of Specification 3.3.2 for any portion of the channel required to be OPERABLE by Specification 3.3.2.

ACTION STATEMENTS

ACTION 1 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours.

ACTION 2 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

a. The inoperable channel is placed in the tripped condition within 6 hours,

b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing per Specification 4.3.1.1, and

c. Either, THERMAL POWER is restricted to less than or equal to 75% of RATED THERMAL POWER and the Power Range Neutron Flux Trip Setpoint is reduced to less than or equal to 85% of RATED THERMAL POWER within 4 hours; or, the QUADRANT POWER TILT RATIO is monitored per Specification 4.2.4.2 when THERMAL POWER is greater than or equal to 50% of RATED THERMAL POWER.

TABLE 3.3-1 (Continued)

ACTION STATEMENTS (Continued)

ACTION 3 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement and with the THERMAL POWER level:

a. Below the P-6 (Intermediate Range Neutron Flux Interlock) Setpoint, restore the inoperable channel to OPERABLE status prior to increasing THERMAL POWER above the P-6 Setpoint, and

b. Above the P-6 Setpoint, but below 10% of RATED THERMAL POWER, restore the inoperable channel to OPERABLE status prior to increasing THERMAL POWER above 10% of RATED THERMAL POWER.

ACTION 4 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement suspend all operations involving positive reactivity changes.

ACTION 5 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement, verify compliance with the SHUTDOWN MARGIN requirements of Specification 3.1.1.1 or 3.1.1.2, as applicable, within 1 hour and at least once per 12 hours thereafter.

ACTION 6 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

a. The inoperable channel is placed in the tripped condition within 6 hours, and

b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.1.1.

ACTION 7 - With the number of OPERABLE channels less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the inoperable channel(s) is (are) placed in the tripped condition within 6 hours.

ACTION 8 - With less than the Minimum Number of Channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.

TABLE 4.3-1 (Continued)

TABLE NOTATIONS

* - When the Reactor Trip System breakers are closed and the Control Rod Drive System is capable of rod withdrawal.

- Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.

- Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.

(1) - If not performed in previous 31 days.

(2) - Heat balance only, above 15% of RATED THERMAL POWER. During startup in MODE 1 above 15% of RATED THERMAL POWER, the required heat balance shall be performed prior to exceeding 30% of RATED THERMAL POWER, or within 24 hours, whichever occurs first. Adjust channel if absolute difference greater than 2%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.

(3) - Compare incore to excore axial flux difference above 15% of RATED THERMAL POWER at least once per 31 Effective Full Power days. Recalibrate if the absolute difference is greater than or equal to 3%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.

(4) - Neutron detectors may be excluded from CHANNEL CALIBRATION.

(5) - Detector plateau curves shall be obtained and evaluated. For the Intermediate Range and Power Range Neutron Flux Channels the provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.

(6) - Incore - Excore Calibration, above 75% of RATED THERMAL POWER at least once per 92 Effective Full Power days. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.

(7) - Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.

(8) - The surveillance requirement is not applicable for a reactor startup from MODE 2 or 3.

(9) - Quarterly Surveillance in MODES 3*, 4* and 5* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.

(10) - Setpoint verification is not applicable.

(11) - At least once per 18 months and following maintenance or adjustment of the Reactor trip breakers, the TRIP ACTUATING DEVICE OPERATIONAL TEST shall include verification of the independence of the Undervoltage trip and the Shunt trip.

(12) - CHANNEL CALIBRATION shall include the RTD bypass loops flow rate.

(13) - Each channel shall be tested at least every 92 days on a STAGGERED TEST BASIS.

(14) - These channels also provide inputs to ESFAS. Comply with the applicable MODES and surveillance frequencies of Specification 4.3.2.1 for any portion of the channel required to be OPERABLE by Specification 3.3.2.

3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.2 The Engineered Safety Features Actuation System (ESFAS) instrumentation channels and interlocks shown in Table 3.3-3 shall be OPERABLE with their Trip Setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3-4 and with RESPONSE TIMES as shown in Table 3.3-5.

APPLICABILITY: As shown in Table 3.3-3.

ACTION:

a. With an ESFAS Instrumentation Channel or Interlock Trip Setpoint less conservative than the value shown in the Allowable Values column of Table 3.3-4, declare the channel inoperable and apply the applicable ACTION requirement of Table 3.3-3 until the channel is restored to OPERABLE status with the Trip Setpoint adjusted consistent with the Trip Setpoint value.

b. With an ESFAS instrumentation channel or interlock inoperable, take the ACTION shown in Table 3.3-3.

SURVEILLANCE REQUIREMENTS

4.3.2.1 Each ESFAS instrumentation channel and interlock and the automatic actuation logic and relays shall be demonstrated OPERABLE by the performance of the Engineered Safety Feature Actuation System Instrumentation Surveillance Requirements specified in Table 4.3-2.

4.3.2.2 The ENGINEERED SAFETY FEATURES RESPONSE TIME of each ESFAS function shall be demonstrated to be within the limit at least once per 18 months. Each test shall include at least one train such that both trains are tested at least once per 36 months and one channel per function such that all channels are tested at least once per N times 18 months where N is the total number of redundant channels in a specific ESFAS function as shown in the "Total No. of Channels" column of Table 3.3-3.

3/4.7.5 CONTROL ROOM VENTILATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.5.1 The Control Room Ventilation System* shall be OPERABLE** with two separate trains with each train consisting of one main supply fan, one filter booster fan, one pressurization supply fan and one HEPA Filter and Charcoal Adsorber System.

APPLICABILITY: All MODES.

ACTION:

MODES 1, 2, 3, and 4:

With one Control Room Ventilation System train inoperable, restore the inoperable train to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

MODES 5 and 6:

a. With one Control Room Ventilation System train inoperable, restore the inoperable train to OPERABLE status within 7 days or initiate and maintain operation of the OPERABLE Control Room Ventilation System train in the recirculation mode.

b. With both Control Room Ventilation System trains inoperable, or with the OPERABLE Control Room Ventilation System required to be in the recirculation mode by ACTION a. not capable of being powered by an OPERABLE emergency power source, suspend all operations involving CORE ALTERATIONS or positive reactivity changes.

SURVEILLANCE REQUIREMENTS

4.7.5.1 Each Control Room Ventilation System train shall be demonstrated OPERABLE:

a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 120°F;

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*The Control Room Ventilation System is common to both units.
**The system may be considered OPERABLE with no chlorine monitors, provided no bulk chlorine gas is stored within the SITE BOUNDARY.

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 31 days by:

1) Initiating flow through the HEPA Filter And Charcoal Adsorber System and verifying that either redundant set of booster and pressurization supply fans operate for at least 10 continuous hours with the heaters operating,

2) Verifying that each Ventilation System redundant fan is aligned to receive electrical power from a separate OPERABLE vital bus, and

3) Starting (unless already operating) each main supply fan, booster fan, and pressurization supply fan, and verifying that it operates for 1 hour.

c. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire, or chemical release in any ventilation zone communicating with the system by:

1) Verifying that the cleanup system satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 1% and uses the test procedure guidance in Regulatory Positions C.5.a, C.5.c, and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978, and the system flow rate is 2100 cfm $\pm$ 10%;

2) Verifying within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978, for a methyl iodide penetration of less than 1%; and

3) Verifying a system flow rate of 2100 cfm + 10% during system operation when tested in accordance with ANSI N510-1980.

d. After 720 hours of charcoal adsorber operation, by verifying, within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978, for a methyl iodide penetration of less than 1%;

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 35 TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NO. DPR-82

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NO. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated March 13, 1987, (Reference LAR 87-03), Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2. The proposed amendments make the following changes to the TS:

A. TS 4.7.5.1, "Control Room Ventilation System," is revised to require that once per month, for each of the two control room ventilation system (CRVS) trains, one of the two redundant sets of booster and pressurization supply fans must be tested for 10 hours with the heaters operating. The present TS requires both sets of fans in each CRVS train to be tested monthly. The revised TS also requires that once per month, each main supply fan, booster fan, and pressurization fan must be tested for one hour to demonstrate fan operability. The present TS requires only the main supply fans to be tested for 15 minutes.

B. Table 3.3-1 of TS 3/4.3.1, "Reactor Trip System Instrumentation," is revised to clearly require a Quadrant Power Tilt Ratio determination to be made only if the plant is above 50 percent power. The previous wording of the TS could have, under some circumstances, required plant shutdown in situations when reduction to 50 percent power would be sufficient to maintain the departure from nucleate boiling (DNB) and linear heat generation within their limits.

C. Table 4.3-1 of TS 3/4.3.1 is revised to allow a plant heat balance to be performed, during startup, between 15 and 30 percent power. The previous TS could be interpreted to require the heat balance to be performed immediately upon reaching 15 percent power. This could result in a less accurate heat balance, and would unnecessarily constrain plant operators during startup.

2.0 DISCUSSION

The NRC staff has reviewed each of the three proposed changes and finds them to be acceptable as follows:

A. TS 4.7.5.1, "Control Room Ventilation System."

The 10-hour test specified in TS 4.7.5.1.b.1 derives from the guidelines of Regulatory Guide 1.52, which states that each engineered safety features (ESF) atmosphere cleanup train should be operated at least 10 hours per month, with the heaters on, to reduce the buildup of moisture on the adsorbers and high-efficiency particulate air (HEPA) filters. Each CRVS train has two redundant booster fans, pressurization fans, and main supply fans. However, each train has only one set of HEPA filters, charcoal adsorbers, and heaters. Thus, it is necessary to run only one booster fan, pressurization fan, and main supply fan on each train for 10 hours with the heaters on to dry out the adsorbers and filters. Therefore, changing this TS to allow the test to be conducted with either redundant set of fans, rather than both redundant sets of fans is acceptable, because the objective of the TS will still be met.

The test specified in TS 4.7.5.1.b.2 derives from the requirement to verify the operability of all fans every 31 days. The change requires all main supply fans, booster fans, and pressurization supply fans to be tested. The present TS requires only the main supply fans to be tested. In discussions with the licensee, it was agreed that testing the fans for one hour provided a higher degree of assurance that the fans are operable than the previously specified 15 minute test. Because this change requires all fans to be tested for a longer period of time, it is more conservative than the present TS and therefore is acceptable to the staff.

B. TS 3/4.3.1, Table 3.3-1, "Reactor Trip System Instrumentation," Action Statement 2.c

This TS is revised to clearly require a Quadrant Power Tilt Ratio (QPTR) determination to be made only if the plant is above 50 percent power. The previous wording of the TS could have, under some circumstances, required plant shutdown in situations when reduction to 50 percent power would be sufficient to maintain the departure from nucleate boiling (DNB) and linear heat generation within their limits. This change is acceptable to the staff because reducing power is an acceptable way of maintaining the plant within its DNB and linear heat generation rate limits.

Also, the frequency of once per 12 hours is deleted from this part of the TS because it is already required by TS 4.2.4.2.

C. TS 3/4.3.1, Table 4.3-1, "Reactor Trip System Instrumentation Surveillance Requirements," Note (2).

This change allows a plant heat balance to be performed, during startup, between 15 and 30 percent power. The previous TS could be interpreted to require the heat balance to be performed immediately upon reaching 15 percent power. This could result in a less accurate heat balance, and would unnecessarily constrain plant operators during startup. Because this change will not have an adverse effect on plant safety, the staff finds it to be acceptable.

For the reasons given above, the NRC staff finds the above changes to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of these amendments will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributors: H. Rood
T. Quay
J. Hayes

Dated: April 14, 1989