

August 26, 1988

Docket Nos.: 50-275
and 50-323

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Mr. J. D. Shiffer, Vice President
Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, California 94106

Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 68034 AND 68035)

The Commission has issued the enclosed Amendment No. 30 to Facility Operating License No. DPR-80 and Amendment No. 29 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 18, 1988.

These amendments allow the initiation value of a reactor trip on turbine trip to be changed from a thermal power greater than 10 percent to a thermal power greater than 50 percent.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

original signed by

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 30 to DPR-80
2. Amendment No. 29 to DPR-82
3. Safety Evaluation

cc w/enclosures:
See next page

DRSP/PD5 JLee 8/18/88	DRSP/PD5 HRood:dr 8/10/88	DEST/SRXB MUMS 8/17/88	OGC rob 8/23/88	DRSP/D:PD5 GKnighton 8/26/88	DRSP/PD5 SJurgens 8/29/88
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 26, 1988

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Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
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Sincerely,

A handwritten signature in cursive script that reads "Harry Rood".

Harry Rood, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 30 to DPR-80
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cc w/enclosures:
See next page

Mr. J. D. Shiffer
Pacific Gas and Electric Company

Diablo Canyon

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-275

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated April 18, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 30, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 26, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2
DOCKET NO. 50-323
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas & Electric Company (the licensee) dated April 18, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 29 , are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


for

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 26, 1988

ATTACHMENT TO LICENSE AMENDMENT NOS.30 AND 29
FACILITY OPERATING LICENSE NOS. DPR-80 AND DPR-82
DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
2-6	2-6
B 2-8	B 2-8
B 2-9	B 2-9
3/4 3-4	3/4 3-4
3/4 3-12	3/4 3-12

TABLE 2.2-1 (Continued)
REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
22. Reactor Trip System Interlocks		
a. Intermediate Range Neutron Flux, P-6	$\geq 1 \times 10^{-10}$ amps	$\geq 6 \times 10^{-11}$ amps
b. Low Power Reactor Trips Block, P-7		
1) P-10 Input	10% of RATED THERMAL POWER	$> 9\%$, $< 11\%$ of RATED THERMAL POWER
2) P-13 Input	$< 10\%$ RTP Turbine Impulse Pressure Equivalent	$< 11\%$ RTP Turbine Impulse Pressure Equivalent
c. Power Range Neutron Flux, P-8	$< 35\%$ of RATED THERMAL POWER	$< 36\%$ of RATED THERMAL POWER
d. Power Range Neutron Flux, P-9	$< 50\%$ of RATED THERMAL POWER	$< 52.1\%$ of RATED THERMAL POWER
e. Power Range Neutron Flux, P-10	10% of RATED THERMAL POWER	$> 9\%$, $< 11\%$ of RATED THERMAL POWER
f. Turbine Impulse Chamber Pressure, P-13	$< 10\%$ RTP Turbine Impulse Pressure Equivalent	$< 11\%$ RTP Turbine Impulse Pressure Equivalent
23. Seismic Trip	≤ 0.35 g	≤ 0.40 g

LIMITING SAFETY SYSTEM SETTINGS

BASES

Turbine Trip

A Turbine trip initiates a Reactor trip. On decreasing power, the Turbine trip is automatically blocked by P-9 (a power level of approximately 50% of RATED THERMAL POWER); and on increasing power, reinstated automatically by P-9.

Safety Injection Input from ESF

If a Reactor trip has not already been generated by the Reactor Trip System instrumentation, the ESF automatic actuation logic channels will initiate a Reactor trip upon any signal which initiates a Safety Injection. The ESF instrumentation channels which initiate a Safety Injection signal are shown in Table 3.3-3.

Reactor Coolant Pump Breaker Position Trip

The Reactor Coolant Pump Breaker Position trip is an anticipatory trip which provides score protection against DNB. The Open/Close Position trip assures a reactor trip signal is generated before the Low Flow Trip Setpoint is reached. No credit was taken in the safety analyses for operation of this trip. The functional capability at the open/close position settings is required to enhance the overall reliability of the Reactor Trip System. Above P-7 (a power level of approximately 10% of RATED THERMAL POWER or a turbine impulse chamber pressure at approximately 10% of full power equivalent) an automatic reactor trip will occur if more than one reactor coolant pump breaker is opened. Below P-7 the trip function is automatically blocked.

Reactor Trip System Interlocks

The Reactor Trip System Interlocks perform the following functions:

- P-6 On increasing power, P-6 allows the manual block of the Source Range trip and de-energizing of the high voltage to the detectors. On decreasing power, Source Range Level trips are automatically reactivated and high voltage restored.
- P-7 On increasing power, P-7 automatically enables Reactor trips on low flow in more than one reactor coolant loop, more than one reactor coolant pump breaker open, reactor coolant pump bus undervoltage and underfrequency, pressurizer low pressure and pressurizer high level. On decreasing power, the above listed trips are automatically blocked.

LIMITING SAFETY SYSTEM SETTINGS

BASES

Reactor Trip System Interlocks (Continued)

- P-8 On increasing power, P-8 automatically enables a reactor trip on low flow in one or more reactor coolant loops. On decreasing power the P-8 automatically blocks the above trip.
- P-9 On increasing power, P-9 automatically enables a reactor trip on turbine trip. On decreasing power, P-9 automatically blocks reactor trip on turbine trip.
- P-10 On increasing power, P-10 allows the manual block of the Intermediate Range reactor trip and the Low Setpoint Power Range trip; and automatically blocks the Source Range trip and deenergizes the Source Range high voltage power. On decreasing power, the Intermediate Range trip and the Low Setpoint Power Range trip are automatically reactivated. Provides input to P-7.
- P-13 Provides input to P-7.

Seismic Trip

The Seismic trip is provided to automatically shutdown the reactor in the event of a seismic occurrence which corresponds in magnitude to the Double Design Earthquake. No credit was taken for operation of the Seismic trip in the safety analysis; however, its functional capability at the specified trip settings is required to enhance the overall reliability of the Reactor Trip System.

DIABLO CANYON - UNITS 1 & 2
 3/4 3-4
 Amendment Nos. 30 and 29

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
18. Safety Injection Input from ESF	2	1	2	1, 2	10
19. Reactor Coolant Pump Breaker Position Trip above P-7	1/breaker	2	1/breaker	1	9
20. Reactor Trip Breakers	2	1	2	1, 2	10
	2	1	2	3*, 4*, 5*	11
21. Automatic Trip and Interlock Logic	2	1	2	1, 2	10
	2	1	2	3*, 4*, 5*	11
22. Reactor Trip System Interlocks					
a. Intermediate Range Neutron Flux, P-6	2	1	2	2##	8
b. Low Power Reactor Trips Block, P-7	P-10 Input 4	2	3	1	8#
	P-13 Input 2	1	2	1	8#
c. Power Range Neutron Flux, P-8	4	2	3	1	8#
d. Power Range Neutron Flux, P-9	4	2	3	1	8#
e. Power Range Neutron Flux, P-10	4	2	3	1, 2	8#
f. Turbine Impulse Chamber Pressure, P-13 (Input to P-7)	2	1	2	1	8#
23. Seismic Trip	3 direc- tions (x,y,z) in 3 locations	2/3 loca- tions one direction	2/3 loca- tions all directions	1, 2	6#

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
20. Reactor Trip System Interlocks (Continued)						
d. Power Range Neutron Flux, P-9	N.A.	R(4)	S/U(1)	N.A.	N.A.	1
e. Low Setpoint Power Range Neutron Flux, P-10	N.A.	R(4)	S/U(1)	N.A.	N.A.	1, 2
f. Turbine Impulse Chamber Pressure, P-13	N.A.	R	S/U(1, 8)	N.A.	N.A.	1
21. Reactor Trip Breaker	N.A.	N.A.	N.A.	M(7, 11)	N.A.	1, 2, 3*, 4*, 5*
22. Automatic Trip and Interlock Logic	N.A.	N.A.	N.A.	N.A.	M(7)	1, 2, 3*, 4*, 5*
23. Seismic Trip	N.A.	R	N.A.	SA	R	1, 2



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated April 18, 1988, Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the Technical Specifications appended to Facility Operating License Nos. DPR-80 and DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2. The proposed amendments would change the DCNPP Combined Technical Specifications to allow a turbine trip without a reactor trip below 50 percent thermal power. Specifically, the proposed change would amend Technical Specification (TS) 2.2.1, "Reactor Trip System Instrumentation Trip Setpoints," and TS 3.3.1 "Reactor Trip System Instrumentation," to allow the initiation value of a reactor trip on turbine trip to be changed from a thermal power greater than 10 percent to a thermal power greater than 50 percent. TS 2.2.1 and 3.3.1 will be revised to incorporate the Power Range Neutron Flux reactor trip system interlock P-9 having a trip setpoint of no greater than 50 percent of rated thermal power with 2 out of 4 logic and to require a minimum of 3 channels operable in Mode 1. The associated bases will be modified to indicate that with a setpoint of 50 percent of rated thermal power, interlock P-9 automatically blocks a reactor trip on turbine trip below the setpoint and enables reactor trip on turbine trip above the setpoint. Also, TS 4.3.1 "Reactor Trip System Instrumentation Surveillance Requirements" will be amended to include the applicable surveillance requirements for interlock P-9 of the power range neutron flux reactor trip system. The licensee states that it is proposing these changes as a part of its Trip Reduction Program to reduce the number of unnecessary challenges to the reactor protection system and impose fewer thermal transients on the plant.

The modification to eliminate the reactor trip on turbine trip with thermal power below 50 percent is accomplished in the Solid State Protection System (SSPS). The licensee will replace the existing reactor trip following turbine trip interlock P-7 output with interlock P-9 output in the logic card rack.

2.0 EVALUATION

The NRC staff has evaluated the proposed changes and has concluded that they are acceptable. The staff's evaluation is given below.

Technical Specification 2.2.1, 3.3.1, 4.3.1 and associated bases currently require a reactor trip on turbine trip to be initiated when the thermal power is greater than 10 percent. This reactor trip is provided to sense a mismatch between turbine load and reactor power. A turbine trip is sensed by a decrease in turbine auto-stop oil pressure or by a limit switch on the full closure of the turbine stop valves. The reactor trip on turbine trip is an anticipatory trip and is not taken credit for in the licensee's FSAR Update Chapter 15 accident analysis. This trip is defeated at some nominal thermal power level to facilitate startups and controlled shut-downs. By raising the thermal level at which the trip is reinstated from the present value of 10 percent to 50 percent, continued reactor operation will be permitted following a turbine trip below 50 percent thermal power. This will enable continued plant operation and restart of the turbine for trips that are readily correctable, or allow commencement of an orderly reactor shutdown. As a result, the number of thermal transients due to unnecessary reactor trips will be reduced.

In support of its requested TS change, the licensee has provided a summary of the evaluations performed by Westinghouse provided in a letter dated October 30, 1987 from J.C. Hobel, Westinghouse, to J.D. Shiffer, PG&E, "Deletion of Reactor Trip on Turbine Trip Below 50 Percent Power for Diablo Canyon Units 1 and 2." The licensee's evaluation considered the applicable accident analyses contained in the FSAR Update for Diablo Canyon and included a best-estimate analysis to show that the pressurizer pressure will not reach the point of power operated relief valve (PORV) activation which is 2350 psia for Diablo Canyon.

(a) Primary Pressure Response

NUREG-0737, Section II.K.3.10 "Proposed Anticipatory Trip Modifications," requires that a change to confine the range of use to high power levels for a reactor trip on turbine trip not be made until it has been shown that the probability of a small break LOCA resulting from a stuck open PORV is substantially unaffected by the modification. The licensee provided the summary and results of analysis performed using the LOFTRAN (reference WCAP-7878) computer code model which simulated the overall thermal/hydraulic/nuclear response of the NSSS as well as the various control and protection systems. In addition, the licensee performed a sensitivity study to determine the effect of certain control system failures on the potential for pressurizer PORV challenges. The results of both analyses based on conservative assumptions showed that the proposed modification will not result in opening a pressurizer PORV. Based

on this best-estimate analysis, the NRC staff concludes that the proposed modification to permit a turbine trip without actuating a direct reactor trip below 50 percent thermal power is acceptable from the standpoint of primary pressure response.

(b) FSAR Update Analysis

The licensee has provided an evaluation of the loss of load transient initiated from a nominal 50 percent power coincident with a loss of flow event 30 seconds after the turbine trip. The licensee's evaluation demonstrates that the consequences of a loss of load/turbine trip event below 50 percent power without a subsequent reactor trip are bounded by the analysis present in the licensee's FSAR Update for loss of load/turbine trip events with respect to peak pressure considerations. The consequences are also bounded by the information provided for a complete loss of flow event with respect to the minimum departure from nuclear boiling ratio (DNBR). If a load decrease from less than 50 percent of thermal power occurs, the control rods will insert and account for a step change of 10 percent thermal power and the turbine steam dump system will operate and control up to 40 percent of the remaining load decrease. Based on the above, the NRC staff concludes that the proposed modification to eliminate the reactor trip on turbine trip with thermal power below 50 percent does not have an adverse impact on plant safety for Diablo Canyon.

In summary, the NRC staff has reviewed the proposed changes to TS 2.2.1, 3.3.1, and 4.3.1 for Diablo Canyon Nuclear Power Plants Units 1 and 2, and the supporting analyses as provided by Pacific Gas and Electric Company. The proposed changes would allow the initiation value of a reactor trip on turbine trip to be changed from a thermal power greater than 10 percent to a thermal power greater than 50 percent. As discussed in the preceding paragraphs, we find that the proposed changes are acceptable, and meet the applicable regulatory requirements.

3.0 CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Chief of the Radiological Health Branch, State Department of Health Service, State of California, of the proposed determination of no significant hazards consideration. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to surveillance requirements. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a

proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Sheri Juergens

Dated: August 26, 1988