

January 7, 1987

Docket Nos. 50-275
and 50-323

Mr. J. D. Shiffer, Vice President
Nuclear Power Generation
c/o Nuclear Power Generation, Licensing
Pacific Gas and Electric Company
77 Beale Street, Room 1451
San Francisco, California 94106

Dear Mr. Shiffer:

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Docket File

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C. Vogan
OGC
E. Jordan
T. Barnhart(8)
V. Benaroya
N. Thompson
ACRS(10)
E. Butcher
J. Shapaker
C. Moon

The Commission has issued the enclosed Amendment No. 11 to Facility Operating License No. DPR-80 and Amendment No. 9 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated February 14, 1986 (LAR 86-03).

These amendments change (1) Technical Specification 3.6.2.3 to assure that two containment fan cooler units are available assuming a single failure; (2) Technical Specification 3.6.1.4 and its Bases to specify a maximum positive containment internal pressure of 1.2 psig and a maximum positive pressure of 46.65 psig in the event of a loss of coolant accident (LOCA); (3) Basis 3/4.6.1.6 to specify a maximum containment pressure of 46.65 psig in the event of a LOCA.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

151

Hans Schierling, Senior Project Manager
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

Enclosures:

- 1. Amendment No. 11 to DPR-80
- 2. Amendment No. 9 to DPR-82
- 3. Safety Evaluation

cc: w/enclosures
See next page

* SEE PREVIOUS CONCURRENCE

PD#3
CVogan*
12/1/86
12/19/86

PD#3
HSchierling*:mak
12/19/86

FOB
VBenaroya*
12/1/86

OGC
HMcGurren*
12/12/86

PD#3
SVargas
12/17/86

No legal objection
HP

Docket Nos. 50-275
and 50-323

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Hans Schierling, Senior Project Manager
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

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- 2. Amendment No. to DPR-82
- 3. Safety Evaluation

cc: w/enclosures
See next page

PD#3
CVogan
7/16/86

PD#3
HSchierling
7/15/86

FOB
VBenaroya
7/15/86

OELD
7/15/86

PD#3
SVarga
7/15/86

Handwritten notes and signatures:
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E. Butcher
J. Shapaker
C. Moon

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- 2 -

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-275
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment dated February 14, 1986, (LAR 86-03) by Pacific Gas & Electric Company (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the combined Technical Specifications for Units 1 and 2 as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

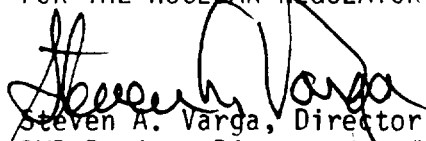
8701140461 870107
PDR ADOCK 05000275
P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 11, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. PG&E shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment becomes effective on the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Director
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 7, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2
DOCKET NO. 50-323
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. DPR-82


1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment dated February 14, 1986, (LAR 86-03) by Pacific Gas & Electric Company (the licensee) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the combined Technical Specifications for Units 1 and 2 as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. PG&E shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment becomes effective on the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Director
PWR Project Directorate #3
Division of PWR Licensing-A, NRR

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 7, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT NOS. 11 AND 9
FACILITY OPERATING LICENSE NOS. DPR-80 AND DPR-82
DOCKET NOS. 50-275 AND 50-323

Revise the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment Number and contain vertical lines indicating the area of change.

Remove Pages

3/4 6-7

3/4 6-13

B 3/4 6-1

B 3/4 6-2

Insert Pages

3/4 6-7

3/4 6-13

B 3/4 6-1

B 3/4 6-2

CONTAINMENT SYSTEMS

INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.4 Containment internal pressure shall be maintained between -1.0 and +1.2 psig.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.4 The containment internal pressure shall be determined to be within the limits at least once per 12 hours.

CONTAINMENT SYSTEMS

CONTAINMENT COOLING SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.3 The Containment Cooling System shall be OPERABLE with either:

- a. At least three electrically independent groups of containment fan cooler units with a minimum of one unit in each group, or
- b. At least two electrically independent groups of containment fan cooler units with a minimum of two units in each group.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

- a. With one group of the above required containment cooling fans inoperable and both Containment Spray Systems OPERABLE, restore the inoperable group of cooling fans to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With two groups of the above required containment cooling fans inoperable, and both Containment Spray Systems OPERABLE, restore at least one group of cooling fans to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore both above required groups of cooling fans to OPERABLE status within 7 days of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With one group of the above required containment cooling fans inoperable and one Containment Spray System inoperable, restore the inoperable Spray System to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore the inoperable group of containment cooling fans to OPERABLE status within 7 days of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.3 Each containment fan cooler unit shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 - 1) Starting each containment fan cooler unit and verifying that each containment fan cooler unit operates for at least 15 minutes,

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 CONTAINMENT

3/4.6.1.1 CONTAINMENT INTEGRITY

CONTAINMENT INTEGRITY ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the safety analyses. This restriction, in conjunction with the leakage rate limitation, will limit the SITE BOUNDARY radiation doses to within the dose guideline values of 10 CFR Part 100 during accident conditions.

3/4.6.1.2 CONTAINMENT LEAKAGE

The limitations on containment leakage rates ensure that the total containment leakage volume will not exceed the value assumed in the safety analyses at the peak accident pressure, P_a . As an added conservatism, the measured overall integrated leakage rate is further limited to less than or equal to $0.75 L_a$ or less than or equal to $0.75 L_t$, as applicable, during performance of the periodic test to account for possible degradation of the containment leakage barriers between leakage tests.

The surveillance testing for measuring leakage rates is consistent with the requirements of Appendix J of 10 CFR Part 50.

3/4.6.1.3 CONTAINMENT AIR LOCKS

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on CONTAINMENT INTEGRITY and containment leak rate. Surveillance testing of the air lock seals provide assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.

3/4.6.1.4 INTERNAL PRESSURE

The limitations on containment internal pressure ensure that: (1) the containment structure is prevented from exceeding its design negative pressure differential with respect to the outside atmosphere of 3.5 psig, and (2) the containment peak pressure does not exceed the design pressure of 47 psig during LOCA conditions.

The maximum peak pressure expected to be obtained from a LOCA event is 46.65 psig. This includes the limit of 1.2 psig for initial positive containment pressure. The total pressure is less than design pressure and is consistent with the safety analyses.

CONTAINMENT SYSTEMS

BASES

3/4.6.1.5 AIR TEMPERATURE

The limitations on containment average air temperature ensure that the overall containment average air temperature does not exceed the initial temperature condition assumed in the safety analysis for a LOCA.

3/4.6.1.6 CONTAINMENT STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the containment will withstand the maximum pressure of 46.65 psig in the event of a LOCA. The visual examination of the concrete, liner, and the Type A leakage test are sufficient to demonstrate this capability.

3/4.6.1.7 CONTAINMENT VENTILATION SYSTEM

Use of the containment purge lines is restricted to two of the three following lines: (1) a supply line, (2) an exhaust line of the purge system, and (3) the vacuum/pressure relief line to ensure that the SITE BOUNDARY dose guidelines of 10 CFR Part 100 would not be exceeded in the event of a loss-of-coolant accident during containment purging operations. The vacuum/pressure relief valves must be blocked to open no more than 50° because these valves have not yet been qualified to close under accident conditions.

Operation will be limited to 200 hours during a calendar year. The 200-hour limit shall not become effective until after initial criticality. The total time the Containment Purge (vent) System isolation valves may be open during MODES 1, 2, 3, and 4 in a calendar year is a function of anticipated need and operating experience.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2
DOCKET NOS. 50-275 AND 50-323

INTRODUCTION

By letter dated February 14, 1986 (Ref. 1), the licensee proposed the following changes to the facility Technical Specifications:

1. Revise Technical Specification 3.6.2.3 to assure that two, instead of the current three, containment fan cooler units are available, assuming a single failure;
2. Revise Technical Specification 3.6.1.4 and its Bases to specify a maximum positive containment internal pressure of 1.2 psig, instead of the current 0.3 psig, and a maximum positive containment internal pressure of 46.65 psig, instead of the current 46.91 psig in the event of a loss of coolant accident (LOCA); and
3. Revise Basis 3/4.6.1.6 to specify a maximum containment pressure of 46.65 psig, instead of the current 46.91 psig in the event of a LOCA.

EVALUATION

The containment fan cooler system (CFCS) for Unit 1 and Unit 2 of the Diablo Canyon Nuclear Power Plant consists of five fan cooler units each, also called fan coolers, cooling fans or fans. The five fan cooler units are divided into three independent groups, each group powered from a separate electrical bus (Buses F, G and H). Two groups (Bus F and Bus G) have two fan units each; the third group (Bus H) has a single fan unit. The current Technical Specifications require all three groups with all five fan cooler units to be operable. A single failure with the loss of one fan places the CFCS into an action statement mode. Thus, three fan coolers would be operable, as a minimum, assuming the single failure to be the loss of a group with two fan cooler units.

The proposed revised Technical Specifications would require either one fan cooler unit in each of the three groups or two fan cooler units in two groups to be operable, before the loss of a fan cooler unit would place the CFCS into an action statement mode. This revision would greatly enhance the operational capability of the CFCS. Two fan coolers would be operable, as a minimum, to provide the required cooling capability as discussed below. The staff concludes that the proposed revision with respect to the three independent electrical power sources is acceptable.

The staff also considered the proposed changes in the Technical Specifications with respect to the necessary redundancy in capability for heat removal from the fan cooler units via the component cooling water (CCW) system and auxiliary saltwater (ASW) system to the ocean as the ultimate heat sink. The CCW system had been evaluated in detail with respect to its redundancy and heat removal capability under accident conditions as discussed in Supplement No. 16 to the Safety Evaluation Report (SSER-16, August 1983). That evaluation determined the system to be adequate in all aspects; it does not change as a result of the proposed changes in the Technical Specifications. (The acceptability of two fan coolers as compared to three fan coolers required for safe shutdown under emergency conditions is discussed below). The ASW system consists of two, 100 percent capacity loops, one each connected to each of the two CCW system heat exchangers. The two ASW loops are cross-connected similar to the cross-connection between the two vital CCW loops. Based on its evaluation of the CCW and ASW system, the staff has determined that adequate cooling capability is provided to the fan coolers under emergency conditions assuming a single failure. The staff concludes that the proposed changes in Technical Specifications are acceptable with respect to heat removal capability from the fan cooler units.

To support the proposed changes to the Technical Specifications, the licensee performed a reanalysis of the containment pressure and temperature response to the design basis LOCA, that is, a double-ended break of the reactor coolant pump suction (cold leg) piping. The licensee's reanalysis, performed by Westinghouse, shows that the peak containment pressure does not exceed the containment design pressure of 47 psig. For the analysis the licensee increased the assumed initial containment pressure from the current 0.3 psig to 1.2 psig, assumed two instead of three containment fan cooler units to be operable following a single failure, and took credit for containment heat sinks that had not been included in the previous analysis. The heat sinks act as a passive heat removal system and, in conjunction with the containment spray system, provide most of the heat removal from the containment atmosphere early in the transient. The additional heat sinks consist of items that have traditionally been ignored in containment analyses as unnecessary, but which have always been present inside the containment. The additional items are the accumulators, reactor coolant pump motors, fan coolers, steam generator snubbers, and large bore pipe supports. Except for these changes, the reanalysis is the same as the analysis previously presented by the licensee (Refs. 2 and 3) and reviewed and approved by the staff (Ref. 4).

Using these assumptions the licensee calculated a peak containment pressure of 46.65 psig. The staff performed a confirmatory analysis using its CONTEMPT-LT/28 computer code, based on two operable fan cooler units, an initial containment pressure of 1.2 psig, and the original heat sinks (i.e., the staff's calculation did not account for the additional heat sinks mentioned above). This resulted in a calculated peak containment pressure of 47 psig, which is slightly higher than the licensee's calculated pressure. However, considering information gained by the earlier confirmatory analysis performed by the staff in Reference 4 and considering the additional heat sinks which the licensee included in its calculation, the staff finds that the licensee's calculated value of 46.65 psig is reasonable. Therefore, the staff concludes that the licensee's analysis is acceptable.

An additional acceptance criterion of the Standard Review Plan (SRP), Section 6.2.1.1.A, is that the calculated containment pressure should decrease to less than 50% of the peak calculated pressure within 24 hours of the onset of an accident. This is based on General Design Criterion 38, which requires that the containment heat removal systems shall be capable of rapidly reducing the containment pressure following a LOCA and maintaining it at an acceptably low level. Although the licensee has not addressed this criterion in the license amendment request (Ref. 1), the licensee's earlier analysis (Refs. 2 and 3) shows that the criterion would be met in less than 3 hours, at which time containment pressure would be less than 20 psig. Although in the current analysis by the licensee the containment heat removal capacity is reduced by one-third (i.e., decreasing the number of operable fan coolers from three to two) and the initial containment pressure is increased by 0.9 psig, the staff concludes that the containment pressure would be reduced to less than 23.3 psig (i.e., 50% of peak calculated pressure) in approximately 6 hours or less, that is, less than the 24 hour maximum set forth in the SRP. Therefore, the staff concludes that General Design Criterion 38 and the cited acceptance criterion of SRP 6.2.1.1.A are satisfied.

Based on the above, the proposed changes to the Technical Specifications are found to be acceptable, as they would not result in the containment design pressure of 47 psig being exceeded in the event of a LOCA, and the requirements of General Design Criteria 38, as provided for in SRP 6.2.1.1.A, are satisfied.

The revised Technical Specifications as approved by the staff are not verbatim the changes proposed by the licensee. In order to maintain greater consistency between the Diablo Canyon Technical Specifications and the Westinghouse Standard Specifications, the staff changed some of the wording proposed by the licensee. These alterations, however, do not change the intent of the changes proposed by the licensee.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the use of the facilities components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously

issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSIONS

The staff concludes, based on its evaluation of the licensee's submittal and its own confirmatory analysis as discussed above, that the proposed changes to the Technical Specifications involve no significant hazards consideration and are acceptable. The staff further concludes, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 7, 1987

PRINCIPAL CONTRIBUTOR:

J. Pulsipher
C. Moon
H. Schierling

REFERENCES:

1. Letter DCL 86-035 from J. D. Shiffer (PG&E) to H. R. Denton (NRC), Subject: LAR 86-03, Containment Internal Pressure and Fan Cooler Operability Technical Specification Changes, dated February 14, 1986.
2. Letter from J. O. Schuyler (PG&E) to H. R. Denton (NRC), Subject: Containment Spray Timing, dated June 23, 1983.
3. Letter from J. O. Schuyler (PG&E) to H. R. Denton (NRC), Subject: Additional Information on Containment Spray Timing - LAR 83-06, dated July 26, 1983.
4. U.S. Nuclear Regulatory Commission, Amendment No. 7 to Facility Operating License No. DPR-76, letter from G. W. Knighton, (NRC) to P. A. Crane (PG&E), dated November 10, 1983.