

July 27, 1987

Docket No. 50-275  
and 50-323

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Mr. J. D. Shiffer, Vice President  
Nuclear Power Generation  
c/o Nuclear Power Generation, Licensing  
Pacific Gas and Electric Company  
77 Beale Street, Room 1451  
San Francisco, California 94106

Dear Mr. Shiffer:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 60829 AND 60830)

The Commission has issued the enclosed Amendment No. 16 to Facility Operating License No. DPR-80 and Amendment No. 15 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated February 13, 1986.

These amendments provide for operability and surveillance tests for certain check valves in the residual heat removal and safety injection systems to ensure adequate pressure isolation between the reactor coolant system and these lower pressure support systems.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

**Original signed by**

Charles M. Trammell, Project Manager  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects

Enclosures:

- 1. Amendment No. 16 to DPR-80
- 2. Amendment No. 15 to DPR-82
- 3. Safety Evaluation

cc w/enclosures:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1  
DOCKET NO. 50-275  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 16  
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated February 13, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 16, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
George W. Knighton, Director  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 27, 1987



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 2  
DOCKET NO. 50-323  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15  
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas & Electric Company (the licensee), dated February 13, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 15, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment becomes effective at the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
George W. Knighton, Director  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 27, 1987

July 27, 1987

ATTACHMENT TO LICENSE AMENDMENT NOS. 16 AND 15  
FACILITY OPERATING LICENSE NOS. DPR-80 AND DPR-82  
DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 4-19

3/4 4-20

3/4 4-21

Insert

3/4 4-19

3/4 4-20

3/4 4-21



REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

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- 3.4.6.2 Reactor Coolant System leakage shall be limited to:
- a. No PRESSURE BOUNDARY LEAKAGE,
  - b. 1 gpm UNIDENTIFIED LEAKAGE,
  - c. 1 gpm total reactor-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,
  - d. 10 gpm IDENTIFIED LEAKAGE from the Reactor Coolant System,
  - e. 40 gpm CONTROLLED LEAKAGE at a Reactor Coolant System pressure of  $2235 \pm 20$  psig, and
  - f. 1 gpm leakage at a Reactor Coolant System pressure of  $2235 \pm 20$  psig for Reactor Coolant System Pressure Isolation Valves as specified in Table 3.4-1.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE and leakage from Reactor Coolant System pressure isolation valves, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System pressure isolation valve leakage greater than the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual and/or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS

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4.4.6.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

- a. Monitoring the containment atmosphere particulate or gaseous radioactivity monitor at least once per 12 hours;
- b. Monitoring the containment structure sump inventory and discharge at least once per 12 hours;
- c. Measurement of the CONTROLLED LEAKAGE to the reactor coolant pump seals at least once per 31 days when the Reactor Coolant System pressure is  $2235 \pm 20$  psig with the modulating valve fully open. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3 or 4;
- d. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours, except when  $T_{avg}$  is being changed by greater than  $5^{\circ}\text{F}/\text{hour}$  or when diverting reactor coolant to the liquid holdup tank, in which cases the required inventory balance shall be performed within 12 hours after completion of the excepted operation; and
- e. Monitoring the Reactor Head Flange Leakoff System at least once per 24 hours.

4.4.6.2.2 As specified in Table 3.4-1, Reactor Coolant System pressure isolation valves shall be demonstrated OPERABLE pursuant to Specification 4.0.5, except that in lieu of any leakage testing required by Specification 4.0.5, each valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:

- a. Every refueling outage during startup,
- b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve, and
- c. Within 24 hours following valve actuation due to automatic or manual action or flow through the valve. After each disturbance of the valve, in lieu of measuring leak rate, leak-tight integrity may be verified by absence of pressure buildup in the test line downstream of the valve.

The provisions of Specification 4.0.4 are not applicable for entry into MODE 3 or 4.

TABLE 3.4-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>
1. 8948 A, B, C, and D	Accumulator, RHR and SIS first off check valves from RCS cold legs
2. 8819 A, B, C, and D	SIS second off check valves from RCS cold legs
3. 8818 A, B, C, and D	RHR second off check valves from RCS cold legs
4. 8956 A, B, C, and D	Accumulator second off check valves from RCS cold legs
5. 8701* and 8702*	RHR suction isolation valves
6. 8949# A, B, C, and D	RHR and SIS first off check valves from RCS hot legs
7. 8905# A, B, C, and D	SIS second off check valves from RCS hot legs
8. 8740# A and B	RHR second off check valves from RCS hot legs
9. 8802*# A and B	SIS to RCS hot legs isolation valves
10. 8703*#	RHR to RCS hot legs isolation valve

\*Testing per Specification 4.4.6.2.2c. not required.

#For flowpaths with 3 pressure isolation valves in series, at least 2 of the 3 valves shall meet the requirements of Specification 3.4.6.2f.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 16 TO FACILITY OPERATING LICENSE NO. DPR-80  
AND AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. DPR-82  
PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By letter dated February 13, 1986, Pacific Gas and Electric Company (PG&E or the licensee) requested amendments to the Technical Specifications appended to Facility Operating License Nos. DPR-80 and DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2. The proposed amendments would provide for operability and surveillance requirements for certain check valves in the residual heat removal system and the safety injection system to ensure adequate pressure isolation between these lower pressure systems and the reactor coolant system.

2.0 EVALUATION

The licensee proposes to add the following check valves to Table 3.4-1, "Reactor Coolant System Pressure Isolation Valves":

8949 A, B, C and D	RHR and SIS first off check valves from RCS hot legs
8905 A, B, C and D	SIS second off check valves from RCS hot legs
8740 A and B	RHR second off check valves from RCS hot legs
8802 A and B	SIS to RCS hot legs isolation valves
8703	RHR to RCS hot legs isolation valve

The licensee states that the initial surveillance for these valves will be performed prior to heatup following the first refueling for each unit. Subsequently, surveillance will be performed every refueling outage during startup and prior to returning the valve to service following maintenance, repair or replacement work on the valve in accordance with the Technical Specifications. An added footnote to Table 3.4-1 states that, for flowpaths with three pressure isolation valves, at least two of three shall meet the requirements of Specification 3.4.6.2.f. Additional

changes reflecting the new footnote were made to the Limiting Condition for Operation and Surveillance Requirements of Specification 3.4.6.2 to maintain consistency. These changes were made by the licensee to comply with a staff request which is detailed in an NRC meeting minutes dated December 9, 1985 and the Diablo Canyon SSER 31.

The staff has evaluated this amendment request in accordance with the guidelines of the Standard Review Plan (SRP) Section 3.9.6 and the ASME Boiler and Pressure Vessel Code Section XI. The staff also checked the request for consistency with the recommendations specified in the NRC meeting minutes dated December 9, 1985 and SSER 31, Section 5.2.8.1, which required the inclusion in Specification 3.4.6.2 of two check valves in series for both the Safety Injection and the Residual Heat Removal Systems to ensure adequate isolation between the Reactor Coolant System and the lower pressure support systems. Because these plants have three valves in series which isolate these systems, the proposed amendment calls for demonstration of operability in any two out of the three valves. The staff review indicates that these changes proposed by the licensee are consistent with applicable guidelines.

On the basis of this review, the staff concludes that the Technical Specification changes are in accordance with guidelines of (1) SRP 3.9.6, (2) the ASME Boiler and Pressure Vessel Code, Section XI, and (3) the staff request contained in the NRC meeting minutes dated December 9, 1985 and SSER 31, Section 5.2.8.1. The staff therefore concludes that the proposed change is acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Dempsey

Dated: July 27, 1987