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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NOV 2 1984

Docket No.: 50-275

Mr. J. O. Shiffer, Vice President  
Nuclear Power Generation  
c/o Nuclear Power Generation, Licensing  
Pacific Gas and Electric Company  
77 Beale Street, Room 1451  
San Francisco, California 94106

Dear Mr. Shiffer:

Subject: Diablo Canyon Nuclear Power Plant, Unit 1 - Issuance of Facility  
Operating License DPR-80

The U. S. Nuclear Regulatory Commission has issued the enclosed Facility  
Operating License No. DPR-80 to the Pacific Gas & Electric Company for the  
Diablo Canyon Nuclear Power Plant, Unit 1, located in San Luis Obispo County,  
California. License No. DPR-80 authorizes operation of the Diablo Canyon  
Nuclear Power Plant, Unit 1, at reactor core power levels not in excess of  
3338 megawatts thermal (100% rated power). This license supersedes Facility  
Operating License No. DPR-76, as amended.

Also enclosed are a copy of (1) a related Federal Register notice which has  
been forwarded to the Office of the Federal Register for publication, and (2)  
a Safety Evaluation pertaining to the issuance of the full power license.

Two signed originals of Amendment No. 7 to Indemnity Agreement Iic. B-75  
which covers the activities authorized under License No. DPR-80 are also  
enclosed. Please sign both copies and return one copy to this office.

Sincerely,

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*Frank M. ...*  
Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures:

1. Facility Operating License No. DPR-80
2. Federal Register Notice
3. Safety Evaluation
4. Amendment 7 to Indemnity Agreement B-75

*See license booklet*

cc: See next page

Diablo Canyon

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PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1  
DOCKET NO. 50-275  
FACILITY OPERATING LICENSE

License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for licenses by Pacific Gas and Electric Company complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Diablo Canyon Nuclear Power Plant, Unit 1 (the facility), has been substantially completed in conformity with Provisional Construction Permit No. CPPR-39 and the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission, except as exempted from compliance in Section 2.D below;
  - D. There is reasonable assurance (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the regulations of the Commission set forth in 10 CFR Chapter I, except as exempted from compliance in Section 2.D below;
  - E. The Pacific Gas and Electric Company is technically qualified to engage in the activities authorized by this operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
  - F. The Pacific Gas and Electric Company has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;
  - G. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;

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- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Facility Operating License No. DPR-80, subject to the conditions for protection of the environment set forth herein, is in accordance with applicable Commission regulations governing environmental reviews (10 CFR Part 50, Appendix D and 10 CFR Part 51) and all applicable requirements have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Pursuant to Commission's Memorandum and Order CLI-84-13, dated August 10, 1984, Facility Operating License No. DPR-76 issued September 22, 1981, as subsequently amended, is superseded by Facility Operating License No. DPR-80, hereby issued to Pacific Gas and Electric Company to read as follows:
- A. This License applies to the Diablo Canyon Nuclear Power Plant, Unit 1, a pressurized water nuclear reactor and associated equipment (the facility), owned by the Pacific Gas and Electric Company (PG&E). The facility is located in San Luis Obispo County, California, and is described in PG&E's Final Safety Analysis Report as supplemented and amended, and the Environmental Report as supplemented and amended.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the Pacific Gas and Electric Company:
    - (1) Pursuant to Section 104(b) of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities", to possess, use, and operate the facility at the designated location in San Luis Obispo County, California, in accordance with the procedures and limitations set forth in this license;
    - (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
    - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This License shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Pacific Gas and Electric Company is authorized to operate the facility at reactor core power levels not in excess of 3338 megawatts thermal (100% rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in this license. The Pacific Gas and Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Initial Test Program

The Pacific Gas and Electric Company shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Pacific Gas and Electric Company's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of PG&E's Final Safety Analysis Report as amended as being essential;

- b. Modification of test objectives, methods, or acceptance criteria for any test identified in Section 14 of PG&E's Final Safety Analysis Report, as amended, as being essential;
- c. Performance of any test at a power level different from that described in the program; and
- d. Failure to complete any test included in the described program (planned or scheduled for power levels up to the authorized power level).

(4) Special Tests

PG&E is authorized to perform steam generator moisture carryover studies and turbine performance tests at the Diablo Canyon Nuclear Power Plant, Unit 1. These studies involve the use of an aqueous tracer solution of three (3) curies of sodium-24. PG&E's personnel shall be in charge of conducting these studies and be knowledgeable in the procedures. PG&E shall impose personnel exposure limits, posting, and survey requirements in conformance with those in 10 CFR Part 20 to minimize personnel exposure and contamination during the studies. Radiological controls shall be established in the areas of the chemical feed, feedwater, steam, condensate and sampling systems where the presence of the radioactive tracer is expected to warrant such controls. PG&E shall take special precautions to minimize radiation exposure and contamination during both the handling of the radioactive tracer prior to injection and the taking of system samples following injection of the tracer. PG&E shall ensure that all regulatory requirements for liquid discharge are met during disposal of all sampling effluents and when re-establishing continuous blowdown from the steam generators after completion of the studies.

(5) Fire Protection

- a. PG&E shall maintain in effect and fully implement all provisions of the approved fire protection plan and of the NRC staff's Fire Protection Evaluation in Supplements 8, 9, 13, 23, and 27 to the Diablo Canyon Safety Evaluation Report, subject to provisions b and c below.
- b. The licensee shall make no change to features of the approved fire protection program which would decrease the level of fire protection in the plant without prior approval of the Commission. To make such a change the licensee must submit an application for license amendment pursuant to 10 CFR 50.90.



- c. The licensee may make changes to features of the approved fire protection program which do not decrease the level of fire protection without prior Commission approval after such features have been installed as approved, provided such changes do not otherwise involve a change in a license condition or technical specification or result in an unreviewed safety question (see 10 CFR 50.59). However, the licensee shall maintain, in an auditable form, a current record of all such changes including an evaluation of the effects of the change on the fire protection program and shall make such records available to NRC inspectors upon request. All changes to the approved program made without prior Commission approval shall be reported to the Director of the Office of Nuclear Reactor Regulation, together with supporting analyses, annually.

(6) NUREG-0737 Conditions

Each of the following conditions shall be completed to the satisfaction of the NRC as indicated below. Each of the following conditions references the appropriate Section in SER Supplements No. 10 and/or No. 12.

a. Shift Technical Advisor (Section I.A.1.1)

PG&E shall provide a fully-trained, on-shift technical advisor to the Shift Foreman.

b. Shift Staffing (Section I.A.1.3)

Until the plant has completed its startup test program, licensed personnel who are not regularly assigned members of the shift staff, including but not limited to the Operations Supervisor, shall not be assigned shift duties to satisfy the minimum staffing requirements for operation in Modes 1, 2, 3 and 4 except for cases of emergencies such as unexpected illness. Exceptions to this requirement may be made only after prior consultation with and approval by the NRC.

c. Management of Operations (Section I.B.1)

The Pacific Gas and Electric Company shall augment the plant staff to provide on each shift an individual experienced in comparable size pressurized water reactor operation. These individuals shall have at least one year of experience in operation of large pressurized

water reactors or shall have participated in the startup of at least three pressurized water reactors. At least one such experienced individual shall be on duty on each shift through the startup test program whenever the reactor is not in a cold shutdown condition for at least the first year of operation or until the plant has attained a nominal 100% power level, whichever occurs first.

d. Procedures for Verifying Correct Performance of Operating Activities (Section I.C.6)

Procedures shall be available to verify the adequacy of the operating activities.

e. Post Accident Sampling (Section II.B.3)

PG&E shall provide the capability to promptly obtain and perform radioisotopic and chemical analyses of reactor coolant and containment atmosphere samples under degraded core conditions without excessive exposure.

f. Relief and Safety Valve Test Requirements (Section II.D.1)

PG&E shall implement the results of the EPRI test program.

g. Containment Isolation Dependability (Section II.E.4.2)

PG&E shall limit the 12-inch vacuum/overpressure relief valve opening to less than or equal to 50 degrees.

h. Calculations for Small-Break LOCAs (Sections II.K.3.30 and II.K.3.31)

PG&E is participating in the Westinghouse Owners Group effort for this item and shall conform to the results of this effort. Within one year of staff approval of the Westinghouse generic methodology for calculating small break LOCAs (II.K.3.30), PG&E shall submit a plant specific calculation (II.K.3.31) for staff review and approval.

i. Long-Term Emergency Preparedness (Section III.A.2)

(1) PG&E shall submit a detailed control room design review summary report by December 31, 1984.

(2) PG&E shall complete operator training on the Safety Parameter Display System and emergency operating procedures by March 28, 1985.

- (3) PG&E shall implement emergency operating procedures based upon Westinghouse Owners Group guidelines by March 28, 1985.

(7) Seismic Design Bases Reevaluation Program (SSER 27 Section IV.5)

PG&E shall develop and implement a program to reevaluate the seismic design bases used for the Diablo Canyon Nuclear Power Plant.

The program shall include the following Elements:

- (1) PG&E shall identify, examine, and evaluate all relevant geologic and seismic data, information, and interpretations that have become available since the 1979 ASLB hearing in order to update the geology, seismology and tectonics in the region of the Diablo Canyon Nuclear Power Plant. If needed to define the earthquake potential of the region as it affects the Diablo Canyon Plant, PG&E will also reevaluate the earlier information and acquire additional new data.
- (2) PG&E shall reevaluate the magnitude of the earthquake used to determine the seismic basis of the Diablo Canyon Nuclear Plant using the information from Element 1.
- (3) PG&E shall reevaluate the ground motion at the site based on the results obtained from Element 2 with full consideration of site and other relevant effects.
- (4) PG&E shall assess the significance of conclusions drawn from the seismic reevaluation studies in Elements 1, 2 and 3, utilizing a probabilistic risk analysis and deterministic studies, as necessary, to assure adequacy of seismic margins.

PG&E shall submit for NRC staff review and approval a proposed program plan and proposed schedule for implementation by January 30, 1985. The program shall be completed and a final report submitted to the NRC three years following the approval of the program by the NRC staff.

PG&E shall keep the staff informed on the progress of the reevaluation program as necessary, but as a minimum will submit quarterly progress reports and arrange for semi-annual meetings with the staff. PG&E will also keep the ACRS informed on the progress of the reevaluation program as necessary, but not less frequently than once a year.

(8) Control of Heavy Loads (SSER 27, Section IV.6)

Prior to startup following the first refueling outage, the licensee shall submit commitments necessary to implement changes and modifications as required to satisfy the guidelines of Section 5.1.2 through 5.1.6 of NUREG-0612 (Phase II: 9-month responses to the NRC Generic Letter dated December 22, 1980).

(9) Emergency Preparedness (SSER 27, Section IV.3)

In the event that the NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

(10) Masonry Walls (SSER-27, Section IV.4; Safety Evaluation of November 2, 1984)

Prior to start-up following the first refueling outage, the licensee shall (1) evaluate the differences in margins between the staff criteria as set forth in the Standard Review Plan and the criteria used by the licensee, and (2) provide justification acceptable to the staff for those cases where differences exist between the staff's and the licensee's criteria.

D. Exemption

Exemption from certain requirements of Appendix J to 10 CFR Part 50 is described in the Office of Nuclear Reactor Regulation's Safety Evaluation Report, Supplement No. 9. This exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest. Therefore, this exemption, previously granted in Facility Operating License No. DPR-76, is hereby reaffirmed. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission.

E. Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission approved physical security, guard training and qualification, and safeguards contingency plans, including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plans, which contain safeguards information as described in 10 CFR 73.21 are entitled "Diablo Canyon

"Nuclear Plant Physical Security Plan," Revision 11 dated May 27, 1982, as revised July 19, August 12, September 17, 1982; February 4, August 3, 1983; January 11, February 6, March 19, April 19, August 29, 1984; "Diablo Canyon Nuclear Plant Guard Training and Qualification Plan," Revision 2 dated February 4, 1983 as revised August 29, 1984; "Diablo Canyon Nuclear Plant Safeguards Contingency Plan," Revision 2 dated February 4, 1983 as revised August 3, 1983, August 29, 1984.

F. Antitrust

Pacific Gas and Electric Company shall comply with the antitrust conditions in Appendix C to this license.

G. Reporting

PG&E shall report any violations of the requirements contained in Sections 2.C(3) through 2.C(10), 2.E and 2.F, of this License within 24 hours. Initial notification shall be made in accordance with the provisions of 10 CFR 50.72 with written follow-up in accordance with the procedures described in 10 CFR 50.73 (b), (c), (d) and (e).

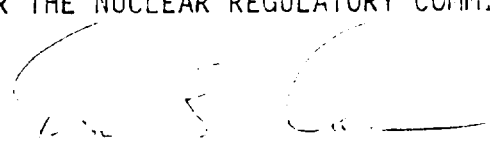
H. Financial Protection

PG&E shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

I. Term of License

This License is effective as of the date of issuance and shall expire at midnight on April 23, 2008.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

Attachments:

1. Appendix A - Technical Specifications
2. Appendix B - Environmental Protection Plan
3. Appendix C - Antitrust Conditions

Date of Issuance: November 2, 1984

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1

PACIFIC GAS ELECTRIC COMPANY

NOTICE OF ISSUANCE OF FACILITY OPERATING LICENSE DPR-80

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission), has issued Facility Operating License No. DPR-80 (the License), to Pacific Gas and Electric Company (PG&E or the licensee) which authorizes operation of the Diablo Canyon Nuclear Power Plant, Unit 1 (the facility or Diablo Canyon Unit 1) at reactor core power levels not in excess of 3338 megawatts thermal (100% rated power) in accordance with the provisions of the license, the Technical Specifications and the Environmental Protection Plan. Diablo Canyon, Unit 1 is a pressurized water reactor located in San Luis Obispo County, California.

On September 22, 1981, the Commission issued to Pacific Gas and Electric Company Facility Operating License No. DPR-76, which authorized fuel loading and operation up to 5% of rated power. On November 19, 1981, the Commission suspended Facility Operating License DPR-76 following PG&E's discovery of errors in seismic design. After substantial effort and review by the licensee and the staff, the Commission reinstated the license on November 8, 1983, CLI-83-27, to the extent of authorizing fuel loading and cold system testing. Hot system testing was subsequently authorized on January 25, 1984, CLI-84-2. Full reinstatement of the License to permit criticality and low power testing (up to 5% of rated power) was authorized on April 13, 1984, CLI-84-5. Following

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additional review, the Commission, on August 10, 1984, authorized issuance of a full-power license, CLI-84-13. On August 17, 1984, the U. S. Court of Appeals, responding to a petition of the Joint Intervenors, granted a stay of issuance of a full-power license pending the Court's review of certain issues. On October 31, 1984 the U. S. Court of Appeals lifted the stay.

Facility Operating License No. DPR-80 incorporates changes to the technical specification that were made subsequent to the issuance of Facility Operating License No. DPR-76, updates and amends the license conditions in Facility Operating License No. DPR-76 in accordance with the NRC evaluation as contained in Supplement 27 to the Safety Evaluation Report and in the Safety Evaluation dated November 2, 1984, and supersedes Facility Operating License No. DPR-76, as amended.

The application for license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the License. Prior public notice of the overall action involving the proposed issuance of an operating license authorizing full power operation was published in the FEDERAL REGISTER on October 19, 1973 (38 F. R. 29105).

The Commission has determined that the issuance of this License will not result in any environmental impacts other than those evaluated in the Final Environmental Statement (issued in May 1973, 38 F.R. 14183) and its Addendum (issued in May 1976, 41 F.R. 22895), the NRC Flood Plain Review (dated September 9, 1981) and the NRC Discussion of Environmental Effects of the

Uranium Fuel Cycle (dated September 9, 1981) since the activity authorized by this License is encompassed by the overall action evaluated in those documents.

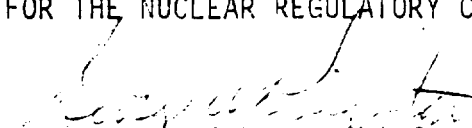
For further details with respect to this action, see (1) the Commission's Memorandum and Order (CLI-83-27) dated November 8, 1983, Commission Memorandum and Order (CLI-84-2), dated January 25, 1984, Commission Memorandum and Order (CLI-84-5) dated April 13, 1984 and Commission Memorandum and Order (CLI-84-13) dated August 10, 1984; (2) Facility Operating License No. DPR-76 for fuel load and 5% power dated September 22, 1981; (3) Facility Operating License No. DPR-80 with Technical Specifications (NUREG-1102) and the Environmental Protection Plan; (4) the reports of the Advisory Committee on Reactor Safeguards dated June 12, 1975, August 19, 1977, July 14, 1978, November 12, 1980, February 14, 1984, April 9, 1984, June 20, 1984 and July 16, 1984; (5) the Commission's Safety Evaluation Report (NUREG-0675, Supplements 1 through No. 27); (6) the Final Environmental Statement dated May 1973 and its Addendum dated May 1976; (7) NRC Flood Plain Review of Diablo Canyon Nuclear Power Plant Site dated September 9, 1981; (8) Discussion of the Environmental Effects of Uranium Fuel Cycle dated September 9, 1981; and (9) Safety Evaluation dated November 2, 1984. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and the California Polytechnic State University Library, Documents and Maps Department, San Luis Obispo, California 93407. A copy of the Facility Operating License No. DPR-80 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing. Copies of NUREG-0675 and the Final Environmental



Statement and its Addendum may be purchased by calling (301) 492-9530 or by writing to the Publications Service Section, Division of Technical Information and Document Control, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555 or purchased from the National Technical Information Service, Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.

Dated at Bethesda, Maryland, the 2nd day of November 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

  
George W. Knighton, Chief  
Licensing Branch No. 3  
Division of Licensing

NOV 2 1984

Safety Evaluation  
Related to Issuance of  
Facility Operating License DPR-80  
Diablo Canyon Nuclear Power Plant, Unit 1  
Docket No. 50-275

1. Introduction

This safety evaluation pertains to the issuance of Facility Operating License DPR-80 for Unit 1 of the Diablo Canyon Nuclear Power Plant authorizing facility operation at 100 percent of rated power. On September 22, 1981 the NRC issued Facility Operating License DPR-76 authorizing facility operation not in excess of 5 percent of rated power. Changes were made to the Technical Specifications and further license conditions were added to DPR-76 in Amendments 1 through 9 and in an Order to modify the License.

The staff has reviewed all provisions of license DPR-76, including those conditions previously proposed to be added to the license by an amendment authorizing full power operation, with respect to their applicability to the full power license DPR-80. A number of license conditions previously included in DPR-76 under Section 2.C have been satisfied and need not be reinstated or have been revised because the required action has been partially or totally completed, or a regulation has since been issued which encompasses the requirements of the license condition. The inclusion of new license conditions and issuance of full power Technical Specifications was previously addressed by the staff in Supplement 27 to the Safety Evaluation Report (SSER-27) dated July 1984.

Presented below is the staff evaluation for a revision to a previously proposed license condition regarding masonry walls (SSER-27, Section II.4 and IV.4) and the deletion of certain exemptions regarding fracture toughness, previously included in Section 2.D of DPR-76.

2. Masonry Walls

In SSER-27, Sections II.4 and IV.4, the staff indicated that there continued to be a need for a license condition regarding certain additional information comparing the licensee's criteria with staff criteria for evaluation of masonry walls. Since that time, the staff has conducted a site visit which included a number of discussions with the licensee, and review of additional documentation including test results. This has permitted a more detailed understanding of the licensee's criteria. As a result, the staff concludes that there is reasonable assurance that these walls will remain functional in the event of a design earthquake and that applicable regulations are met.

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However, it is still necessary for the licensee to document its analysis of the differences in margins when comparing its criteria to staff criteria even though it is not likely that structural change will result from this evaluation. The proposed license condition as discussed in SSER-27 has therefore been revised accordingly and is included under Section 2.C(10).

### 3. Compliance with Appendices G and H to 10 CFR Part 50 (Fracture Toughness)

In Section 2.D of the low power license DPR-76 exemptions were granted from certain requirements, among others, of Appendices G and H as related to fracture toughness. These exemptions have been deleted from the full power license DPR-80 as discussed below.

In previous safety evaluations (Supplement Nos. 9 and 13) the staff determined that exemptions to Sections III.C.2 and IV.A.4 of Appendix G to 10 CFR 50 and Section II.B of Appendix H to 10 CFR 50 would be required and were justified. Since those evaluations were published, Appendices G and H have been revised. The revised Appendices G and H were published in the Federal Register on May 27, 1983 and became effective on July 26, 1983. The exemptions to Appendices G and H, which were discussed in our previous safety evaluations, are no longer required, because the Diablo Canyon, Unit 1 materials and surveillance program complies with the revised Appendices G and H requirements. A discussion of these requirements follows.

Section III.C.2 and IV.A.4 in previous versions of Appendix G had specific requirements for preparation of reactor vessel beltline weld metal test specimens and minimum fracture toughness requirements for reactor coolant pressure boundary ferritic bolting, respectively. In lieu of these specific requirements, the current provisions of Appendix G require that reactor vessel beltline weld metal test specimens and reactor coolant pressure boundary ferritic bolting comply with the requirements in ASME Code edition and addenda permitted by section 50.55a of 10 CFR 50. In a previous safety evaluation we determined that the reactor vessel for Diablo Canyon, Unit 1 was fabricated to ASME Code edition and addenda as provided by the requirements of section 50.55a. Hence, the Diablo Canyon, Unit 1 materials comply with the revised Appendix G requirements and exemptions to Appendix G are no longer required.

Section II.B in previous versions of Appendix H required that the surveillance program conducted prior to the first capsule withdrawal comply with the 1973 edition of ASTM E-185. The current provision of Appendix H requires that the surveillance program conducted prior to the first capsule withdrawal comply with the requirements of the edition of ASTM E-185 that is current on the issue date of the ASME Code to which the reactor vessel was purchased. The Diablo Canyon, Unit 1 surveillance program complies with these requirements. Hence, the Diablo Canyon, Unit 1 surveillance program complies with the revised Appendix H requirements and an exemption to Appendix H is no longer required.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket Nos. 50-275  
50-323

AMENDMENT TO INDEMNITY AGREEMENT NO. B-75  
Amendment No. 7

Effective November 2, 1984, Indemnity Agreement No. B-75, between Pacific Gas and Electric Company, and the Nuclear Regulatory Commission, dated December 31, 1975, as amended, is hereby further amended as follows:

Item 3 of the Attachment to the indemnity agreement is deleted in its entirety and the following substituted therefor:

Item 3 - License number or numbers

SNM-1503 (From 12:01 a.m., December 31, 1975 to 12 midnight, September 21, 1981 inclusive)

SNM-1667 (From 12:01 a.m., October 15, 1976)

DPR-76 (From 12:01 a.m., September 22, 1981 to 12 midnight, November 1, 1984 inclusive)

DPR-80 (From November 2, 1984

FOR THE UNITED STATES NUCLEAR REGULATORY COMMISSION

Jerome Saltzman, Assistant Director  
State and Licensee Relations  
Office of State Programs

Accepted \_\_\_\_\_, 1984

By \_\_\_\_\_  
PACIFIC GAS AND ELECTRIC COMPANY

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