

Docket No.: 50-275

JAN 6 1983

Mr. Malcolm H. Furbush
Vice President - General Counsel
Pacific Gas & Electric Company
Post Office Box 7442
San Francisco, California 94120

Dear Mr. Furbush:

Subject: Issuance of Amendment No. 3 to Facility Operating License No. DPR-76
Diablo Canyon Nuclear Power Plant, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 3 to Facility Operating License No. DPR-76 for the Diablo Canyon Nuclear Power Plant Unit 1, located in San Luis Obispo County, California.

This amendment is in response to your letter of December 21, 1982. Amendment No. 3 authorizes relief from Section 3/4.9.7 of the Diablo Canyon Technical Specifications for a period of 120 days in order that you may make certain modifications in the Fuel Handling Building.

A copy of the related safety evaluation supporting Amendment No. 3 to Facility Operating License No. DPR-76 is enclosed. Also enclosed is a copy of the related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

Enclosures:

- 1. Amendment No. 3 to DPR-76
- 2. Safety Evaluation
- 3. Federal Register Notice

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We would like to see subject to additional findings & review

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Mr. Malcolm H. Furbush

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San Luis Obispo County Board of Supervisors
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PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON NUCLEAR POWER PLANT, UNIT 1
DOCKET NO. 50-275
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3
License No. DPR-76

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated December 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-76 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 3, are hereby incorporated in this license. The Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

Date of Issuance: JAN 6 1983

OFFICE ▶	DL: LB#3	DL LB#3	OELD	DL LB#3	DL: ADL		
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DATE ▶	1/5/83	1/5/83	1/4/83	1/6/83	1/6/83		

*DL - subject to
existing finding
no legal
objection*

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ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. DPR-76

DOCKET NO. 50-275

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. Also to be replaced is the following overleaf page to the amended page.

Amendment Page

3/4 9-7

Overleaf Page

3/4 9-8

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REFUELING OPERATIONS

3/4.9.7 CRANE TRAVEL - FUEL HANDLING BUILDING

LIMITING CONDITION FOR OPERATION

3.9.7 Loads in excess of 2500 pounds* shall be prohibited from travel over fuel assemblies in the spent fuel pool.**

APPLICABILITY: With fuel assemblies in the spent fuel pool.

ACTION:

With the requirements of the above specification not satisfied, place the crane load in a safe condition.

SURVEILLANCE REQUIREMENTS

4.9.7 Loads shall be verified to be less than 2500 pounds prior to movement over fuel assemblies in the spent fuel pool.

* The movable fuel handling building walls may travel over fuel assemblies in the spent fuel pool.

**During the period of January 06, 1983 through May 6, 1983, relief from this specification is granted to permit the installation and removal of temporary spent fuel pool steel plate covers. During this period, loads in excess of 500 pounds shall be prohibited from travel over the spent fuel steel plate covers.

REFUELING OPERATIONS

3/4.9.8 RESIDUAL HEAT REMOVAL AND COOLANT CIRCULATION

HIGH WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.8.1 At least one residual heat removal (RHR) train shall be OPERABLE and in operation.** *

APPLICABILITY: MODE 6 when the water level above the top of the reactor vessel flange is at least 23 feet.

ACTION:

- a. With no residual heat removal train OPERABLE and in operation, suspend all operations involving an increase in the reactor decay heat load or a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required RHR train to OPERABLE and operating status as soon as possible. Close all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere within 4 hours.

SURVEILLANCE REQUIREMENTS

4.9.8.1.1 The required RHR train shall be demonstrated OPERABLE pursuant to Specification 4.0.5.

4.9.8.1.2 At least one residual heat removal train shall be verified to be in operation and circulating reactor coolant at a flow rate of greater than or equal to 3000 gpm at least once per 12 hours.

*The residual heat removal train may be removed from operation for up to 1 hour per 8 hour period during the performance of CORE ALTERATIONS in the vicinity of the reactor pressure vessel hot legs.

**The residual heat removal train may be removed from operation and OPERABLE status for up to 2 hours per 8 hour period for the performance of leak testing the RHR suction isolation valves.

DIABLO CANYON, UNIT 1
SAFETY EVALUATION REPORT

By letter dated December 21, 1982 the Pacific Gas and Electric Company (PG&E) requested relief from Section 3/4.9.7 of the Diablo Canyon, Unit 1 Technical Specifications for a period of 120 days in order to make certain modifications in the Fuel Handling Building. Section 3/4.9.7 of the Technical Specifications prohibits the movement of loads in excess of 2,500 pounds over the spent fuel pool while fuel assemblies are being stored in the pool.

EVALUATION

As a result of the independent design verification program, certain modifications are required to be made to the Fuel Handling Building superstructure in order to assure that the building is designed in accordance with the FSAR commitments. In performing these modifications, there is the potential for dropping objects into the dry Unit 1 spent fuel pool which presently contains unirradiated fuel assemblies which have been stored there since 1976. To preclude an object from falling into the dry spent fuel pool and possibly damaging the unirradiated fuel assemblies, PG&E has proposed to install steel covers over the Unit 1 spent fuel pool until the necessary modifications have been completed. The covers are comprised of two 1/2" thick steel plates each of which weighs approximately 11 tons and one smaller steel plate cover which weighs approximately 2 1/2 tons. Since each of the steel plate covers weigh in excess of the 2500 pounds maximum permissible load to be moved over the spent fuel pool by Section 3/4.9.7 of the Technical Specifications, PG&E has requested relief from this technical specification for a period of 120 days. The sizing of the steel plate cover was based on preventing the heaviest identified object, which weighs approximately 500 pounds, from penetrating the proposed cover.

The Fuel Handling Building Crane, which has a design rating of 100 tons, will be used to transfer the steel plate covers from the temporary storage at a height of 1" above the spent fuel pool side rail tracks as they are moved into position. The same procedure will be used when removing the covers. Established maintenance procedures for testing and maintaining the Fuel Handling Building Crane will be adhered to. The two larger steel plate covers contain angle restraints along the sides, in the direction of travel along the pool side rail tracks, to further preclude the potential for dropping (sliding) these covers into the pool while they are being positioned over the pool. The requirements of CAL OSHA in regard to the rigging and lifting of loads will be complied with, as appropriate, and operations of the crane will be done by qualified and experienced personnel. The rigging that will be used to move the covers will have a safety factor of 10. The foregoing measures and procedures provide assurance against the inadvertent dropping of the steel plate covers into the pool.

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The introduction of borated or unborated water into the spent fuel pool with the fuel assemblies in their present geometry would not result in criticality. The dropping of an object on the fuel assemblies in the dry spent fuel pool will not cause criticality. Criticality could only occur if the fuel racks were deformed in a manner that would decrease the distance between the fuel assemblies to form a critical geometry in the presence of water. To further preclude this possibility, the three systems that provide water to the fuel storage area (Primary Water, Fuel Pool, Makeup Water, and Fire Water to the hose reel stations) will be valved off and tagged inoperable. Following the installation of the covers, the Fire Water valves will be returned to their original valve position and tags removed. Prior to removing the covers the Fire Water will again be temporarily valved off and tagged inoperable. We find that above precautionary measures to prevent damage to the fuel assemblies and consequent criticality are acceptable.

ENVIRONMENTAL CONSIDERATIONS

The NRC staff has determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact and pursuant 10 CFR Section 51.5 (d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

Based upon our evaluation of the proposed changes to the Diablo Canyon, Unit 1 Technical Specifications, we have concluded that: (1) because this amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, this amendment does not involve a significant safety hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the proposed change is acceptable.

Dated: JAN 6 1983

*See previous page for concurrence/initials

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Dated:

New level objects

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-275

PACIFIC GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY

OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 3 to Facility Operating License No. DPR-76 issued to the Pacific Gas and Electric Company (the licensee), for Diablo Canyon Nuclear Power Plant, Unit No. 1 (the facility) located in San Luis Obispo County, California. The amendment is effective as of the date of issuance.

The amendment authorizes relief from specific technical specifications for a period of 120 days to allow for the movement of loads in excess of 2500 pounds in order to make certain modifications in the Fuel Handling Building.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5 (d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) the application for amendment dated December 21, 1982, (2) Amendment No. 3 to Facility Operating License No. DPR-76, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. 20555, and at the California Polytechnic State University Library, Documents and Maps Department, San Luis Obispo, California 93407. A copy of items (2) and (3) may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing, Office of Nuclear Reactor Regulation.

Dated at Bethesda, Maryland this ^{6th} day of January, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

Bob
will change
F.R. NOTICE
MAND.

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DATE	1/5/83	1/5/83	1/6/83	1/6/83	1/ /83		

DIABLO CANYON AMENDMENT NO. 3

Document Control (50-275)

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