December 12, 1986

Docket Nos.: 50-361
and 50-362

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Southern California Edison Company
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Post Office Box 800
Rosemead, California 91770

Mr. James C. Holcombe
Vice President - Power Supply
San Diego Gas & Electric Company
101 Ash Street
Post Office Box 1831
San Diego, California 92112

Gentlemen:

Subject: Issuance of Amendment No. 56 to Facility Operating License NPF-10 and Amendment No. 45 to Facility Operating License NPF-15 San Onofre Nuclear Generating Station, Units 2 and 3

The Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 56 to Facility Operating License No. NPF-10 and Amendment No. 45 to Facility Operating License No. NPF-15 for the San Onofre Nuclear Generating Station, Units 2 and 3, located in San Diego County, California. The amendments revise the Technical Specifications concerning (1) the fuel handling area vent gaseous airborne radiation monitor and (2) the minimum capacity of the refueling machine and the corresponding overload cutoff limit.

These amendments were requested by your letters of June 13 and August 28, 1986 as supplemented by your letter dated November 4, 1986, and are covered by Proposed Change Numbers PCN-217 and PCN-222.

A copy of the Safety Evaluation supporting the amendments is also enclosed.

Sincerely,

/s/

Harry Rood, Senior Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosures:
1. Amendment No. 56 to NPF-10
2. Amendment No. 45 to NPF-15
3. Safety Evaluation

cc: See next page

PD7	PD7	OGC	DIR/PD7
JLee/yt	HRood	12-2	GWKnighton
11/26/86	12/01/86	11/ /86	12/ /86

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DEC 12 1986

ISSUANCE OF AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NPF-10
AND AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NPF-15
SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3

DISTRIBUTION

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California State Library
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San Clemente, CA 92672

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment to the license for San Onofre Nuclear Generating Station, Unit 2 (the facility) filed by the Southern California Edison Company on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and the City of Anaheim, California (licensees) dated June 13 and August 28, 1986, as supplemented by letter dated November 4, 1986, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and Paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 56, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Harry Rood

Harry Rood, Senior Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: DEC 12 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Pages	Overleaf Pages
3/4 3-25	-
3/4 3-26	-
3/4 9-6	3/4 9-5

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES

FUNCTIONAL UNIT	TRIP VALUE	ALLOWABLE VALUES
11. FUEL HANDLING ISOLATION (FHIS)		
a. Manual (Trip Buttons)	Not Applicable	Not Applicable
b. Airborne Radiation		
i. Gaseous	(8)	(8)
ii. Particulate/Iodine	$\leq 5.7 \times 10^4$ cpm**	$\leq 6.0 \times 10^4$ cpm**
c. Automatic Actuation Logic	Not Applicable	Not Applicable
12. CONTAINMENT PURGE ISOLATION (CPIS)		
a. Manual (Trip Buttons)	Not Applicable	Not Applicable
b. Airborne Radiation		
i. Gaseous	(6)(7)	(6)(7)
ii. Particulate	(6)(7)	(6)(7)
iii. Iodine	(6)(7)	(6)(7)
c. Containment Area Radiation (Gamma)	$\leq$ 325 mR/hr (MODES 1-4) $\leq$ 2.4 mR/hr (MODE 6)	$\leq$ 340 mR/hr (MODES 1-4) $\leq$ 2.5 mR/hr (MODE 6)
d. Automatic Actuation Logic	Not Applicable	Not Applicable

TABLE 3.3-4 (Continued)

TABLE NOTATION

(1) Value may be decreased manually, to a minimum of greater than or equal to 300 psia, as pressurizer pressure is reduced, provided the margin between the pressurizer and this value is maintained at less than or equal to 400 psia;* the setpoint shall be increased automatically as pressurizer pressure is increased until the trip setpoint is reached. Trip may be manually bypassed below 400 psia; bypass shall be automatically removed whenever pressurizer is greater than or equal to 400 psia.

(2) Value may be decreased manually as steam generator pressure is reduced, provided the margin between the steam generator pressure and this value is maintained at less than or equal to 200 psi;* the setpoint shall be increased automatically as steam generator pressure is increased until the trip setpoint is reached.

(3) % of the distance between steam generator upper and lower level instrument nozzles.

(4) Inverse time relay set value 3165V, trip will occur within the tolerances specified in Figure 3.3-1 for the range of bus voltages.

(5) Actuated equipment only; does not result in CIAS.

(6) The trip setpoint shall be set sufficiently high to prevent spurious alarms/trips yet sufficiently low to assure an alarm/trip should an inadvertent release occur.

(7) Prior to the completion of DCP 53N, the setpoints for Containment Airborne Radiation Monitor 2RT-7804-1 shall be determined by the ODCM.

(8) The trip setpoint shall be set sufficiently high to prevent spurious alarm/trips yet sufficiently low to assure an alarm/trip should a fuel handling accident occur.

*Variable setpoints are for use only during normal, controlled plant heatups and cooldowns.

**Above normal background.

REFUELING OPERATIONS

3/4.9.5 COMMUNICATIONS

LIMITING CONDITION FOR OPERATION

3.9.5 Direct communications shall be maintained between the control room and personnel at the refueling station.

APPLICABILITY: During CORE ALTERATIONS.

ACTION:

When direct communications between the control room and personnel at the refueling station cannot be maintained, suspend all CORE ALTERATIONS.

SURVEILLANCE REQUIREMENTS

4.9.5 Direct communications between the control room and personnel at the refueling station shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.

REFUELING OPERATIONS

3/4.9.6 REFUELING MACHINE

LIMITING CONDITION FOR OPERATION

3.9.6 The refueling machine shall be used for movement of fuel assemblies with or without CEAs and shall be OPERABLE with:

a. A minimum capacity of 3200 pounds, and

b. An overload cut off limit of less than or equal to 3550 pounds.

The refueling machine auxiliary hoist may be used for the movement of CEAs without fuel bundles and shall be OPERABLE with an overload cut off limit of less than or equal to 1000 pounds.

APPLICABILITY: During movement of CEAs and/or fuel assemblies within the reactor pressure vessel utilizing the refueling machine auxiliary hoist or refueling machine.

ACTION: With the requirements for the refueling machine OPERABILITY not satisfied, suspend all refueling machine operations involving the movement of fuel assemblies with or without CEAs within the reactor pressure vessel. With the requirements for the refueling machine auxiliary hoist not satisfied, suspend all refueling machine auxiliary hoist operations involving the movement of CEAs within the reactor pressure vessel.

SURVEILLANCE REQUIREMENTS

4.9.6 The refueling machine used for movement of fuel assemblies with or without CEAs within the reactor pressure vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by performing a load test of at least 3200 pounds and demonstrating an automatic load cut off when the refueling machine load exceeds 3550 pounds. The refueling machine auxiliary hoist used for movement of CEAs within the reactor pressure vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by demonstrating an automatic load cut off when the auxiliary hoist load exceeds 1000 pounds.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment to the license for San Onofre Nuclear Generating Station, Unit 3 (the facility) filed by the Southern California Edison Company on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and the City of Anaheim, California (licensees) dated June 13 and August 28, 1986, as supplemented by letter dated November 4, 1986, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and Paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 45, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The changes in the Technical Specifications are to become effective within 30 days of issuance of the amendment. In the period between issuance of the amendment and the effective date of the new Technical Specifications, the licensees shall adhere to the Technical Specifications existing at the time. The period of time during changeover shall be minimized.

4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Harry Rood

Harry Rood, Senior Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: DEC 12 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page	Overleaf Page
3/4 3-25	-
3/4 3-26	-
3/4 9-6	3/4 9-5

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP VALUES

FUNCTIONAL UNIT	TRIP VALUE	ALLOWABLE VALUES
11. FUEL HANDLING ISOLATION (FHIS)		
a. Manual (Trip Buttons)	Not Applicable	Not Applicable
b. Airborne Radiation		
i. Gaseous	(8)	(8)
ii. Particulate/Iodine	$\leq 5.7 \times 10^4$ cpm**	$\leq 6.0 \times 10^4$ cpm**
c. Automatic Actuation Logic	Not Applicable	Not Applicable
12. CONTAINMENT PURGE ISOLATION (CPIS)		
a. Manual (Trip Buttons)	Not Applicable	Not Applicable
b. Airborne Radiation		
i. Gaseous	(6)(7)	(6)(7)
ii. Particulate	(6)(7)	(6)(7)
iii. Iodine	(6)(7)	(6)(7)
c. Containment Area Radiation (Gamma)	$\leq$ 325 mR/hr (MODES 1-4) $\leq$ 2.4 mR/hr (Mode 6)	$\leq$ 340 mR/hr (MODES 1-4) $\leq$ 2.5 mR/hr (MODE 6)
d. Automatic Actuation Logic	Not Applicable	Not Applicable

TABLE 3.3-4 (Continued)

TABLE NOTATION

(1) Value may be decreased manually, to a minimum of greater than or equal to 300 psia, as pressurizer pressure is reduced, provided the margin between the pressurizer and this value is maintained at less than or equal to 400 psia;* the setpoint shall be increased automatically as pressurizer pressure is increased until the trip setpoint is reached. Trip may be manually bypassed below 400 psia; bypass shall be automatically removed whenever pressurizer is greater than or equal to 400 psia.

(2) Value may be decreased manually as steam generator pressure is reduced, provided the margin between the steam generator pressure and this value is maintained at less than or equal to 200 psi;* the setpoint shall be increased automatically as steam generator pressure is increased until the trip setpoint is reached.

(3) % of the distance between steam generator upper and lower level instrument nozzles.

(4) Inverse time relay set value 3165V, trip will occur within the tolerances specified in Figure 3.3-1 for the range of bus voltages.

(5) Actuated equipment only; does not result in CIAS.

(6) The trip setpoint shall be set sufficiently high to prevent spurious alarms/trips yet sufficiently low to assure an alarm/trip should an inadvertent release occur.

(7) Prior to the completion of DCP 53N, the setpoints for Containment Airborne Radiation Monitor 3RT-7804-1 shall be determined by the ODCM.

(8) The trip setpoint shall be set sufficiently high to prevent spurious alarm/trips yet sufficiently low to assure an alarm/trip should a fuel handling accident occur.

*Variable setpoints are for use only during normal, controlled plant heatups and cooldowns.

**Above normal background.

REFUELING OPERATIONS

3/4.9.5 COMMUNICATIONS

LIMITING CONDITION FOR OPERATION

3.9.5 Direct communications shall be maintained between the control room and personnel at the refueling station.

APPLICABILITY: During CORE ALTERATIONS.

ACTION:

When direct communications between the control room and personnel at the refueling station cannot be maintained, suspend all CORE ALTERATIONS.

=

SURVEILLANCE REQUIREMENTS

4.9.5 Direct communications between the control room and personnel at the refueling station shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.

3/4.9.6 REFUELING MACHINE

LIMITING CONDITION FOR OPERATION

3.9.6 The refueling machine shall be used for movement of fuel assemblies with or without CEAs and shall be OPERABLE with:

a. A minimum capacity of 3200 pounds, and

b. An overload cut off limit of less than or equal to 3550 pounds.

The refueling machine auxiliary hoist may be used for the movement of CEAs without fuel bundles and shall be OPERABLE with an overload cut off limit of less than or equal to 1000 pounds.

APPLICABILITY: During movement of CEAs and/or fuel assemblies within the reactor pressure vessel utilizing the refueling machine auxiliary hoist or refueling machine.

ACTION: With the requirements for the refueling machine OPERABILITY not satisfied, suspend all refueling machine operations involving the movement of fuel assemblies with or without CEAs within the reactor pressure vessel. With the requirements for the refueling machine auxiliary hoist not satisfied, suspend all refueling machine auxiliary hoist operations involving the movement of CEAs within the reactor pressure vessel.

SURVEILLANCE REQUIREMENTS

4.9.6 The refueling machine used for movement of fuel assemblies with or without CEAs within the reactor pressure vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by performing a load test of at least 3200 pounds and demonstrating an automatic load cut off when the refueling machine load exceeds 3550 pounds. The refueling machine auxiliary hoist used for movement of CEAs within the reactor pressure vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by demonstrating an automatic load cut off when the auxiliary hoist load exceeds 1000 pounds.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 56 TO NPF-10 AND AMENDMENT NO. 45 TO NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL.

SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 & 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

Southern California Edison Company (SCE, or the licensee), on behalf of itself and the other licensees, San Diego Gas and Electric Company, The City of Riverside, California, and The City of Anaheim, California, has submitted a number of applications for license amendments for San Onofre Nuclear Generating Station (SONGS), Units 2 and 3. The NRC staff's evaluation of two of these applications is described below.

2.0 DISCUSSION

Proposed Change PCN-217

By letter dated June 13, 1986, SCE requested that the SONGS 2 and 3 technical specifications concerning the fuel handling area vent (FHAV) gaseous airborne radiation monitor be changed. This monitor provides an alarm to the control room in the event of detection of high radiation levels in the fuel handling building (FHB) such as would occur from a postulated fuel handling accident, and sends a signal to the fuel handling isolation system which trips (isolates) the normal ventilation system and activates the emergency (post-accident cleanup) ventilation system for the FHB. These actions are intended to preclude unacceptable radioactivity releases from a postulated fuel handling accident. The proposed technical specification change would delete the noble gas allowable alarm setpoint for the FHAV gaseous airborne radiation monitor, and substitute a requirement to set the alarm at a value sufficiently high to prevent spurious alarm/trips yet sufficiently low to assure an alarm/trip should a fuel handling accident occur.

The staff has evaluated the licensees' proposal to revise the technical specification for the FHAV gaseous airborne radiation monitor. The bases for our conclusion are the requirements of General Design Criterion (GDC) 64 and the guidelines of Standard Review Plan (SRP), NUREG-0800, Sections 11.5 and 15.7.4.

The current setpoint of 140 cpm above background for the FHAV monitor does not provide adequate margin above background during refueling. San Onofre Units 2 and 3 have experienced several spurious alarms and fuel handling

isolation system actuations during their refueling outages. These nuisance alarms and actuations are attributable to movement of irradiated fuel and fuel reconstitution activities. The proposed change will permit the trip setpoint to be set sufficiently high to prevent spurious alarm/trips yet sufficiently low to assure an alarm/trip in the event a fuel handling accident should occur. Further, the licensees have indicated that they have performed a study which confirms adequate monitor response to radiation releases from a postulated design basis fuel handling accident (60 broken fuel rods) and thereby ensure that any releases will be a small fraction of 10 CFR Part 100 limits with the proposed change. A similar technical specification change was previously incorporated in the San Onofre, Units 2 and 3 technical specifications for the containment purge isolation radiation monitor.

Based on the above, the staff concludes that the licensees' proposed technical specification change for the FHAV gaseous airborne radiation monitor (PCN-217) meets the requirements of GDC-64 for monitoring of potential radiation releases during normal and accident conditions in order to ensure that 10 CFR Part 100 limits are not exceeded. We, therefore, find the proposed change acceptable.

Proposed Change PCN-222

By letter dated August 28, 1986, SCE proposed changing Technical Specification 3/4.9.6, "Refueling Machine" to increase the minimum capacity of the refueling machine from 3000 pounds to 3200 pounds and the corresponding overload cutoff limit from 3350 pounds to 3550 pounds, in order to incorporate the added weight associated with the installation of a new removable TV camera assembly. At the staff's request, the licensees provided additional information supporting PCN-222 by letter dated November 4, 1986.

PCN-222 was proposed because SCE has decided to install a removable TV camera unit on the refueling machine as a replacement for the existing fixed TV camera in order to facilitate camera maintenance. The removable TV camera unit is mounted on a stainless steel track installed alongside the refueling machine hoist box and is thus an integral part of the hoist box. The new camera/track assembly adds 200 pounds additional weight to the hoist box. Because the refueling machine is used for movement of fuel assemblies, this additional weight must be accounted for in order to ensure its proper operation and to confirm that the design of the machine is not affected.

The staff has evaluated the proposed change and has concluded that the refueling machine operational functions will not be affected by the installation of the new TV camera unit as follows:

a. The load carrying capacity of the refueling machine is not challenged by 200 lbs. of additional weight. The load carrying capacity of the hoist is 125% of the maximum existing service load of 3000 lbs., i.e. 3750 lbs., as verified by the vendor. Thus, the new service load of 3200 lbs. maximum which includes the hoist box, TV camera and fuel assembly, is within the vendor specified limits.

b. NUREG-0612, "Control of Heavy Loads" guidelines do not apply to the refueling machine since the new TV camera unit does not add sufficient weight for the machine load to be considered a heavy load as defined in NUREG-0612.

c. The margin between the actual weight (incorporating the new camera unit) and the new technical specification setpoints remains identical to the existing limiting condition for operation (LCO) requirements. Thus, the new setpoints provide the same safety margin inherent in the currently existing technical specification.

d. The LCO load limit requirement (3550 lbs) would result in interruption of the fuel assembly hoisting should the overload setpoint be exceeded. Thus, loads will remain below the 3750 lbs. maximum recommended by the vendor and a fuel assembly drop accident is precluded.

e. Repair of the camera unit will only be undertaken with an empty hoist box (without fuel assembly in the refueling machine), thereby eliminating such actions as a possible means for causing a fuel assembly drop accident.

f. The safety features provided by the existing interlock system associated with the refueling machine for preventing a fuel handling accident (fuel assembly drop) as previously identified in the San Onofre, Units 2 and 3 FSAR are unchanged.

Based on our evaluation, we conclude that the proposed 200 lb. increase in the minimum capacity of the refueling machine and corresponding increase in the overload cutoff limit meets the fuel handling requirements of GDC 61 as it will not affect the safe operation of the refueling machine and thereby will not increase the probability or consequences of a postulated design basis fuel handling accident (fuel assembly drop). Therefore, we find the proposed change to Technical Specification 3/4.9.6, "Refueling Machine" to be acceptable.

3.0 CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Chief of the Radiological Health Branch, State Department of Health Services, State of California, of the proposed determinations of no significant hazards consideration. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation or use of facility components located within the restricted area. The staff has determined that the amendments involve no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The commission has previously issued proposed findings that the amendments involve no significant hazards consideration, and there has been no public comment on such findings. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

Based upon our evaluation of the proposed changes to the San Onofre Units 2 and 3 Technical Specifications, we have concluded that: there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the proposed changes are acceptable, and are hereby incorporated into the San Onofre 2 and 3 technical specifications.

Dated: DEC 12 1986

Principal Contributors: J. Minns, J. Raval