Docket Nos.: 50-361
and 50-362

APR 18 1986

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2244 Walnut Grove Avenue
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Rosemead, California 91770

Mr. James C. Holcombe
Vice President - Power Supply
San Diego Gas & Electric Company
101 Ash Street
Post Office Box 1831
San Diego, California 92112

Gentlemen:

Subject: Issuance of Amendment No. 45 to Facility Operating License NPF-10 and Amendment No. 34 to Facility Operating License NPF-15 San Onofre Nuclear Generating Station, Units 2 and 3

The Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 45 to Facility Operating License No. NPF-10 and Amendment No. 34 to Facility Operating License No. NPF-15 for the San Onofre Nuclear Generating Station, Units 2 and 3, located in San Diego County, California. The amendments revise Technical Specification 3/4.9.8.1, "Shutdown Cooling and Coolant Circulation - High Water Level," and Technical Specification 3/4.9.8.2, "Shutdown Cooling and Coolant Circulation - Low Water Level." The change will reduce the Mode 6 shutdown cooling system flow requirements from 4000 gpm to 2200 gpm.

These amendments were requested by your letter of January 24, 1986, and are covered by Proposed Change Number PCN-3.

A copy of the Safety Evaluation supporting the amendments is also enclosed.

Sincerely,

Original signed by:
George W. Knighton

George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosures:
1. Amendment No. 45 to NPF-10
2. Amendment No. 34 to NPF-15
3. Safety Evaluation

cc: See next page

DPWR-B:PBD7	DPWR-B:PBD7	OELD	DPWR-B:PBD7
JLee	HRood/yt		GWKnighton
2/14/86	4/17/86	2/18/86	2/18/86

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Walnut Creek, California 94596

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Marine Review Committee, Inc.
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Encinitas, California 92024

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c/o U. S. Nuclear Regulatory Commission
Post Office Box 4329
San Clemente, California 92672

cc:
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Government Publications Section
Library & Courts Building
Sacramento, CA 95841
ATTN: Ms. Mary Schnell

Mayor, City of San Clemente
San Clemente, CA 92672

Chairman, Board Supervisors
San Diego County
1600 Pacific Highway, Room 335
San Diego, CA 92101

California Department of Health
ATTN: Chief, Environmental
Radiation Control Unit
Radiological Health Section
714 P Street, Room 498
Sacramento, CA 95814

Mr. Joseph O. Ward, Chief
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State Department of Health Services
714 P Street, Building #8
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APR 18 1986

ISSUANCE OF AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NPF-10
AND AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NPF-15
SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 AND 3

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the license for San Onofre Nuclear Generating Station, Unit 2 (the facility) filed by the Southern California Edison Company on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and The City of Anaheim, California (licensees) dated January 24, 1986, as supplemented by letters dated April 11, and April 16, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and Paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 45, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective immediately and is to be fully implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: APR 18 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contains vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page	Overleaf Page
3/4 9-8	3/4 9-7
3/4 9-9	3/4 9-10

3/4.9.7 FUEL HANDLING MACHINE - SPENT FUEL STORAGE POOL BUILDING

LIMITING CONDITION FOR OPERATION

3.9.7 Loads in excess of 2000 pounds shall be prohibited from travel over fuel assemblies in the storage pool.

APPLICABILITY: With fuel assemblies in the storage pool.

ACTION:

With the requirements of the above specification not satisfied, place the fuel handling machine in a safe condition.

SURVEILLANCE REQUIREMENTS

4.9.7 Fuel handling machine interlocks and physical stops which prevent fuel handling machine travel with loads in excess of 2000 pounds over fuel assemblies shall be demonstrated OPERABLE within 7 days prior to fuel handling machine use and at least once per 7 days thereafter during fuel handling machine operation.

REFUELING OPERATIONS

3/4.9.8 SHUTDOWN COOLING AND COOLANT CIRCULATION

HIGH WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.8.1 At least one shutdown cooling train shall be OPERABLE and in operation.#

APPLICABILITY: MODE 6 when the water level above the top of the reactor pressure vessel flange is greater than or equal to 23 feet.

ACTION:

With no shutdown cooling train OPERABLE and in operation, suspend all operations involving an increase in the reactor decay heat load or a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required shutdown cooling train to OPERABLE and operating status as soon as possible. Close all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere within 4 hours.

SURVEILLANCE REQUIREMENTS

4.9.8.1 At least one shutdown cooling train shall be verified to be in operation and circulating reactor coolant at a flow rate of greater than or equal to 2200 gpm at least once per 12 hours.

#The shutdown cooling train may be removed from operation for up to 1 hour per 8 hour period during the performance of CORE ALTERATIONS in the vicinity of the reactor pressure vessel hot legs.

REFUELING OPERATIONS

LOW WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.8.2 Two independent shutdown cooling trains shall be OPERABLE and at least one shutdown cooling train shall be in operation.

APPLICABILITY: MODE 6 when the water level above the top of the reactor pressure vessel flange is less than 23 feet.

ACTION:

a. With less than the required shutdown cooling trains OPERABLE, immediately initiate corrective action to return the required shutdown cooling trains to OPERABLE status, or to establish greater than or equal to 23 feet of water above the reactor pressure vessel flange as soon as possible.

b. With no shutdown cooling train in operation, suspend all operations involving a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required shutdown cooling train to operation. Close all containment penetrations providing direct access from the containment atmophere to the outside atmosphere within 4 hours.

SURVEILLANCE REQUIREMENTS

4.9.8.2 At least one shutdown cooling train shall be verified to be in operation and circulating reactor coolant at a flow rate of greater than or equal to 2200gpm at least once per 12 hours.

3/4.9.9 CONTAINMENT PURGE ISOLATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.9.9 The containment purge isolation system shall be OPERABLE.

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the containment purge isolation system inoperable, close each of the containment purge penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.9 The containment purge isolation system shall be demonstrated OPERABLE within 72 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment purge valve isolation occurs on manual initiation and on a high radiation test signal.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the license for San Onofre Nuclear Generating Station, Unit 2 (the facility) filed by the Southern California Edison Company on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and The City of Anaheim, California (licensees) dated January 24, 1986, as supplemented by letters dated April 11, and April 16, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this amendment and Paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 34, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective immediately and is to be fully implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: APR 18 1986

APR 18 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page	Overleaf Page
3/4 9-8	3/4 9-7
3/4 9-9	3/4 9-10

3/4.9.7 FUEL HANDLING MACHINE - SPENT FUEL STORAGE POOL BUILDING

LIMITING CONDITION FOR OPERATION

3.9.7 Loads in excess of 2000 pounds shall be prohibited from travel over fuel assemblies in the storage pool.

APPLICABILITY: With fuel assemblies in the storage pool.

ACTION:

With the requirements of the above specification not satisfied, place the fuel handling machine in a safe condition.

SURVEILLANCE REQUIREMENTS

4.9.7 Fuel handling machine interlocks and physical stops which prevent fuel handling machine travel with loads in excess of 2000 pounds over fuel assemblies shall be demonstrated OPERABLE within 7 days prior to fuel handling machine use and at least once per 7 days thereafter during fuel handling machine operation.

REFUELING OPERATIONS

3/4.9.8 SHUTDOWN COOLING AND COOLANT CIRCULATION

HIGH WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.8.1 At least one shutdown cooling train shall be OPERABLE and in operation.#

APPLICABILITY: MODE 6 when the water level above the top of the reactor pressure vessel flange is greater than or equal to 23 feet.

ACTION:

With no shutdown cooling train OPERABLE and in operation, suspend all operations involving an increase in the reactor decay heat load or a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required shutdown cooling train to OPERABLE and operating status as soon as possible. Close all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere within 4 hours.

SURVEILLANCE REQUIREMENTS

4.9.8.1 At least one shutdown cooling train shall be verified to be in operation and circulating reactor coolant at a flow rate of greater than or equal to 2200 gpm at least once per 12 hours.

#The shutdown cooling train may be removed from operation for up to 1 hour per 8 hour period during the performance of CORE ALTERATIONS in the vicinity of the reactor pressure vessel hot legs.

REFUELING OPERATIONS

LOW WATER LEVEL

LIMITING CONDITION FOR OPERATION

3.9.8.2 Two independent shutdown cooling trains shall be OPERABLE and at least one shutdown cooling train shall be in operation.#

APPLICABILITY: MODE 6 when the water level above the top of the reactor pressure vessel flange is less than 23 feet.

ACTION:

a. With less than the required shutdown cooling trains OPERABLE, immediately initiate corrective action to return the required shutdown cooling trains to OPERABLE status, or to establish greater than or equal to 23 feet of water above the reactor pressure vessel flange as soon as possible.

b. With no shutdown cooling train in operation, suspend all operations involving a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required shutdown cooling train to operation. Close all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere within 4 hours.

SURVEILLANCE REQUIREMENTS

4.9.8.2 At least one shutdown cooling train shall be verified to be in operation and circulating reactor coolant at a flow rate of greater than or equal to 2200 gpm at least once per 12 hours.

Both shutdown cooling trains may be removed from operation for up to 1 hour per 8-hour period during the performance of CORE ALTERATIONS in the vicinity of the reactor pressure vessel hot legs provided all operations involving a reduction in boron concentration of the RCS are suspended.

3/4.9.9 CONTAINMENT PURGE ISOLATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.9.9 The containment purge isolation system shall be OPERABLE.

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the containment purge isolation system inoperable, close each of the containment purge penetrations providing direct access from the containment atmosphere to the outside atmosphere. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.9 The containment purge isolation system shall be demonstrated OPERABLE within 72 hours prior to the start of and at least once per 7 days during CORE ALTERATIONS by verifying that containment purge valve isolation occurs on manual initiation and on a high radiation test signal.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION SUPPORTING ISSUANCE OF

AMENDMENT NO. 45 TO NPF-10 AND AMENDMENT NO. 34 TO NPF-15

SAN ONOFRE NUCLEAR GENERATING STATION, UNITS 2 & 3

DOCKET NOS. 50-361 AND 50-362

1.0 INTRODUCTION

Southern California Edison Company (SCE), on behalf of itself and the other licensees, San Diego Gas and Electric Company, The City of Riverside, California, and The City of Anaheim, California, has submitted several applications for license amendments for San Onofre Nuclear Generating Station (SONGS), Units 2 and 3. One such request, Proposed Change PCN-3, is evaluated herein. This change revises Technical Specification (TS) 3/4.9.8.1 and 3/4.9.8.2 to allow the shutdown cooling system (SDCS) flow rate to be as low as 2200 gpm, rather that the previous minimum of 4000 gpm.

2.0 EVALUATION OF CHANGE

The NRC staff has evaluated the proposed changes and has concluded that they are acceptable. The staff's evaluation is given below.

Technical Specifications 4.9.8.1 and 4.9.8.2, which pertain to Mode 6, currently require, among other things that the licensee verify at least once per 12 hours that at least one shutdown cooling train is in operation and circulating reactor coolant at a flow rate of greater than or equal to 4000 gpm during refueling operations. Vortexing has been observed in the suction of the SDCS when the water level in the reactor coolant system was reduced to mid-loop. The requirement of a flow rate of greater than or equal to 4000 gpm during refueling operations can cause vortexing in the suction to the SDCS when the water level in the reactor coolant system is reduced to mid-loop. SCE states that the minimum flow requirement of 4000 gpm for Mode 6 can delay inspecting and/or repairing the steam generators and reactor coolant pumps since these activities require a reduced water level in the reactor coolant system, and it is more efficient to perform these activities during Mode 6. The purpose of the minimum flow requirements in TS 3/4.9.8 are given in the bases for this section, which states that the purpose is to ensure "that (1) sufficient cooling capacity is available to remove decay heat and maintain the water in the reactor pressure vessel below 140°F as required during the REFUELING MODE, and (2) sufficient coolant circulation is maintained through the reactor core to minimize the effects of a boron dilution incident and prevent boron stratification."

In regards to cooling capacity, SCE stated in letters dated January 24, 1986 and April 11, 1986 that an analysis had been performed that shows that a minimum SDCS flow rate of 2200 gpm is sufficient to maintain RCS temperature below 140°F while in the refueling mode. In a subsequent meeting on April 16, 1986, SCE provided the staff with more detailed information regarding the bases for the proposed minimum SDCS flow rate of 2200 gpm. This information is contained in a letter from M.O. Medford to G.W. Knighton, dated April 16, 1986. SCE's analysis includes estimates of the SDCS flow required to maintain the water in the reactor pressure vessel below 140°F for three different periods of time after shutdown. The analysis is based on the heat generation rates from radioactive decay of fission products in a Proposed American Nuclear Society Standard - "Decay Energy Release Rates Following Shutdown of Uranium-Fueled Thermal Reactors," and on standard heat transfer models. The assumptions used in the analysis lead to conservative estimates of the minimum flow requirements. The staff has reviewed SCE's analysis and concludes that the proposed minimum flow requirement for operation of the SDCS during refueling operations will ensure that there is sufficient cooling capacity to remove decay heat and maintain the water in the reactor pressure vessel below 140°F as required during the refueling mode. The lower minimum SDCS flow rate of 2200 gpm will also reduce the potential for vortexing in the SDCS.

In regards to the effects of a boron dilution incident, SCE has analyzed the effects of a boron dilution incident for a partially drained reactor coolant system during refueling operation. This analysis is contained in chapter 7.4.4, "CVCS Malfunction (Inadvertent Boron Dilution)," of the SONGS Unit 2, Cycle 3 Reload Analysis Report. The analysis is based on the models described in Chapter 15.4.1.4 of the Final Safety Analysis Report (FSAR) which contains a detailed analysis of a boron dilution event during cold shutdown. The NRC staff's Standard Review Plan (Chapter 15.4.6) lists the acceptance criteria for analysis of a boron dilution incident. This criteria includes a minimum time interval of 30 minutes for operator action to terminate the transient. With the reduced minimum flow rate, the estimated time to lose the minimum shutdown margin still meets the criterion to terminate the event. With respect to boron stratification, the FSAR analysis of a boron dilution incident during cold shutdown was based on an SDCS flow of 2200 gpm with only one charging pump available (44 gpm) for Mode 6. This results in an SDCS flow to charging pump flow ratio of about 50. In regards to boron stratification, the FSAR analysis still bounds the present analysis for a minimum SDCS flow of 2200 gpm for Mode 6. We conclude that with the proposed changes to the TS, San Onofre 2 and 3 will continue to meet the regulatory guidance and requirements of Chapter 15.4.6 of the Standard Review Plan.

In summary, we have reviewed the proposed changes to TS 3/4.9.8 for San Onofre Units 2 and 3, and SCE's supporting analysis. The proposed changes would reduce the minimum flow requirement for operation of the shutdown cooling system during refueling operations from 4000 gpm to 2200 gpm. As discussed in the preceding paragraphs, we find that the proposed changes are acceptable, and meet the applicable regulatory guidance and requirements.

3.0 CONTACT WITH STATE OFFICIAL

The NRC staff has advised the Chief of the Radiological Health Branch, State Department of Health Services, State of California, of the proposed determination of no significant hazards consideration. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation or use of facility components located within the restricted area. The staff has determined that the amendments involve no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued proposed findings that the amendments involve no significant hazards consideration, and there has been no public comment on such findings. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec. 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

Based upon our evaluation of the proposed changes to the San Onofre Units 2 and 3 Technical Specifications, we have concluded that: there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the proposed changes are acceptable, and are hereby incorporated into the San Onofre 2 and 3 Technical Specifications.

Dated: APR 18 1986