

Docket Nos.: 50-361
and 50-362

JAN 10 1986

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Mr. James C. Holcombe
Vice President - Power Supply
San Diego Gas & Electric Company
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Post Office Box 1831
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Gentlemen:

Subject: Issuance of Notice of Consideration of Issuance of Amendments

Enclosed for your information is a copy of a notice of Consideration of Issuance of Amendments to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing related to your applications dated November 27 and December 10, 1985, to revise the technical specifications relating to the allowable range for the moderator temperature coefficient (Proposed Change No. 213). This Notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

ORIGINAL SIGNED BY

Harry Rood, Project Manager
PWR Project Directorate No. 7
Division of PWR Licensing-B

Enclosure:
Federal Register Notice

cc: See next page

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Southern California Edison Company

San Onofre Nuclear Generating Station
Units 2 and 3

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Southern California Edison Company - 2 - San Onofre 2/3

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Library & Courts Building
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ATTN: Ms. Mary Schnell

Mayor, City of San Clemente
San Clemente, CA 92672

Chairman, Board Supervisors
San Diego County
1600 Pacific Highway, Room 335
San Diego, CA 92101

California Department of Health
ATTN: Chief, Environmental
Radiation Control Unit
Radiological Health Section
714 P Street, Room 498
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Mr. Joseph O. Ward, Chief
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714 P Street, Building #8
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UNITED STATES NUCLEAR REGULATORY COMMISSIONSOUTHERN CALIFORNIA EDISON COMPANY, ET AL.DOCKET NOS. 50-361 AND 50-362NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF-10 and NPF-15, issued to Southern California Edison Company, San Diego Gas and Electric Company, The City of Riverside, California and The City of Anaheim, California (the licensees), for operation of the San Onofre Nuclear Generating Station, Units 2 and 3 located in San Diego County, California.

The amendments would revise the technical specifications relating to the allowable range for the moderator temperature coefficient (reference PCN-213). The proposed amendments were requested by the licensee's letters of November 27, and December 10, 1985.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the request for amendments involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facilities in accordance with the proposed amendments would not (1) involve significant increase in the probability or consequences of an accident previously evaluated; or (2)

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create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The Commission has provided guidance concerning the application of standards for determining whether a significant hazards consideration exists by providing certain examples (48 FR 14870) of amendments that are considered not likely to involve significant hazards considerations. Example (vi) relates to a change which either may result in some increase to the probability or consequences of a previously-analyzed accident or may reduce in some way a safety margin, but where the results of the change are clearly within all acceptable criteria with respect to the system or component specified in the Standard Review Plan; for example, a change resulting from the application of a small refinement of a previously used calculational model or design method. The proposed change is similar to this example of 48 FR 14870.

The proposed change revises Technical Specification 3/4.1.1.3, "Moderator Temperature Coefficient". Technical Specification 3/4.1.1.3 defines limitations on moderator temperature coefficient (MTC) to ensure that the assumptions used in the accident and transient analyses remain valid through each fuel cycle. The surveillance requirements for measurement of the MTC during each fuel cycle are performed to confirm the MTC value since this coefficient changes slowly due principally to the reduction in reactor coolant system (RCS) boron concentration associated with fuel burnup. The confirmation that the measured MTC value is within its limit provides assurances that the coefficient will be maintained within acceptable values throughout each fuel cycle.

Technical Specification 3/4.1.1.3 currently states that the moderator temperature coefficient shall be less negative than -2.5×10^{-4} delta k/k°F at rated thermal power. This limit is based on transient analysis of overcooling events, in which a more negative MTC results in a more severe transient. The MTC normally becomes more negative during an operating cycle, primarily because boron is progressively removed from the moderator to compensate for fuel burnup. However, recent mid-cycle measurements of MTC at San Onofre Unit 2 have indicated that the MTC is becoming more negative faster than predicted. This is predicted to result in the existing technical specification limit being exceeded before the end of the cycle. The licensees have re-examined the transient analysis used to set the lower MTC limit and have found that the analysis was in error. Specifically, the reactivity uncertainty associated with CEA worth was included twice in the analysis, rather than once. Correction of this error results in a change in the lower limit on MTC from -2.5×10^{-4} to -3.0×10^{-4} delta k/k°F at rated power. Consequently, SCE has proposed to change TS 3.4.1.1.3 to reflect the revised lower limit.

The proposed change is similar to Example (vi) of 48 FR 14870 in that the change in limiting Moderator Temperature Coefficient is a reduction in a margin of safety, but the results of the change are within the guidelines of the Standard Review Plan (SRP), Section 4.3, "Nuclear Design." This change does not make changes in analytical methods or results of analyses previously found to be acceptable by the NRC and used to demonstrate conformance with the regulations. Furthermore, a re-evaluation of the most limiting transient

shows that the present safety analyses remain valid and bounding. Thus, the proposed change to relax the Technical Specification MTC limit is compensated for by the removal of an erroneous CEA rod worth uncertainty and does not change the overall conclusion of the present safety analyses.

Because of the proposed change is similar to Example (vi) of 48 FR 14870, the NRC staff proposes to determine that the change does not involve a significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Comments should be addressed to the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attn: Docketing and Service Branch.

By January 27, 1986, the licensees may file a request for a hearing with respect to issuance of the amendment to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Request for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a

request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment requests involves no significant hazards consideration, the Commission may issue the amendments and make them effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendments.

Normally, the Commission will not issue the amendments until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue

the license amendments before the expiration of the 30-day notice period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 1717 H Street, N.W. Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at (800) 325-6000 (in Missouri (800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to George W. Knighton: petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Executive Legal Director, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Charles R. Kocher, Esq., Southern California Edison Company, 2244 Walnut Grove Avenue, P. O. Box 800 Rosemead, California 91770 and Orrick, Herrington & Sutcliffe, Attn: David R. Pigott, Esq., 600 Montgomery Street, San Francisco, California 94111.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board, that the petitioner and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714 (a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the applications for amendments which are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the San Clemente Library, 242 Avenida Del Mar, San Clemente, California 92672.

Dated at Bethesda, Maryland, this 20th day of December 1985.

FOR THE NUCLEAR REGULATORY COMMISSION

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George W. Knighton, Director
PWR Project Directorate No. 7
Division of PWR Licensing-B

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