

Docket No.: 50-361

DEC 23 1982

Mr. Robert Dietch
Vice President
Southern California Edison Company
2244 Walnut Grove Avenue
Post Office Box 800
Rosemead, California 91770

Mr. Gary D. Cotton
Mr. Louis Bernath
San Diego Gas & Electric Company
101 Ash Street
Post Office Box 1831
San Diego, California 92112

Gentlemen:

Subject: Issuance of Amendment No. 12 to Facility Operating License NPF-10
San Onofre Nuclear Generating Station, Unit 2

The Nuclear Regulatory Commission has issued Amendment No. 12 to Facility Operating License NPF-10 for the San Onofre Nuclear Station, Unit 2, located in San Diego County, California.

This amendment is in response to your letters dated December 20, 21, and 22, 1982. The amendment modifies the technical specifications to allow the non-critical component cooling water loop to circulate water to the reactor coolant pumps after safety injection initiation. The non-critical CCM loop will still be required to isolate on high containment pressure.

A copy of the related safety evaluation supporting Amendment No. 12 to Facility Operating License NPF-10 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original Signed By:

Victor Aersia
Joe George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

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PDR ADOCK 05000361
P PDR

Enclosures:

- 1. Amendment No. 12
- 2. Safety Evaluation
- 3. Federal Register Notice

cc w/enclosures:
See next page

*See previous concurrences/initials *V.N. 12/23/82*

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This amendment is in response to your letters dated December 20 and 21, 1982. The amendment modifies the technical specifications to allow the non-critical component cooling water loop to circulate water to the reactor coolant pumps after safety injection initiation. The non-critical CCM loop will still be required to isolate on high containment pressure.

A copy of the related safety evaluation supporting Amendment No. 12 to Facility Operating License NPF-10 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

George W. Knighton, Chief
Licensing Branch No. 3
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cc: California Department of Health
ATTN: Chief, Environmental Radiation
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Radiological Health Section
714 P Street, Room 498
Sacramento, California 95814

Chairman, Board Supervisors
San Diego County
San Diego, California 92412

Mayor, City of San Clemente
San Clemente, California 92672

U. S. Environmental Protection Agency
ATTN: EIS Coordinator
Region IX Office
215 Fremont Street
San Francisco, California 94111

Director, Energy Facilities Siting Division
Energy Resources Conservation &
Development Commission
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Sacramento, California 95825

California State Library
Government Publications Section
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Sacramento, California 95841
ATTN: Ms. Mary Schell

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
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SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment for the San Onofre Nuclear Generating Station, Unit 2 (the facility) filed by the Southern California Edison Company on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and The City of Anaheim, California (licensees) dated December 20, 1982, as supplemented by letters dated December 21 and 22, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 12, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

for Victor Arnes
George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

Date of Issuance: DEC 23 1982

*no legal objection
subject to
clearly noted
p. 1*

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ATTACHMENT TO LICENSE AMENDMENT NO. 12

FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Also to be replaced are the following overleaf pages to the amended pages.

Amendment Page

Overleaf Page

3/4 3-28

3/4 3-27

3/4 3-30

3/4 3-29

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TABLE 3.3-5

ENGINEERED SAFETY FEATURES RESPONSE TIMES

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME (SEC)</u>
1. <u>Manual</u>	
a. SIAS	
Safety Injection	Not Applicable
Control Room Isolation	Not Applicable
Containment Isolation (3)	Not Applicable
Containment Emergency Cooling	Not Applicable
b. CSAS	
Containment Spray	Not Applicable
c. CIAS	
Containment Isolation	Not Applicable
d. MSIS	
Main Steam Isolation	Not Applicable
e. RAS	
Containment Sump Recirculation	Not Applicable
f. CCAS	
Containment Emergency Cooling	Not Applicable
g. EFAS	
Auxiliary Feedwater	Not Applicable
h. CRIS	
Control Room Isolation	Not Applicable
i. TGIS	
Toxic Gas Isolation	Not Applicable
j. FHIS	
Fuel Handling Building Isolation	Not Applicable
k. CPIS	
Containment Purge Isolation	Not Applicable

Table 3.3-5 (continued)

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME (SEC)</u>
2. <u>Pressurizer Pressure-Low</u>	
a. SIAS	
(1) Safety Injection	
(a) High Pressure Safety Injection	31.2*
(b) Low Pressure Safety Injection	41.2*
(2) Control Room Isolation	Not Applicable
(3) Containment Isolation (NOTE 3)	11.2* (NOTE 2)
(4) Containment Spray (Pumps)	25.6*
(5) Containment Emergency Cooling	
(a) CCW Pumps	31.2*
(b) CCW Valves (Note 4b)	23.2*
(c) Emergency Cooling Fans	21.2*
3. <u>Containment Pressure-High</u>	
a. SIAS	
(1) Safety Injection	
(a) High Pressure Safety Injection	41.0*
(b) Low Pressure Safety Injection	41.0*
(2) Control Room Isolation	Not Applicable
(3) Containment Spray (Pumps)	25.4*
(4) Containment Emergency Cooling	
(a) CCW Pumps	31.0*
(b) CCW Valves (Note 4b)	23.0*
(c) Emergency Cooling Fans	21.0*
b. CIAS	
(1) Containment Isolation	10.9* (NOTE 2)
(2) CCW Valves (Note 4a)	20.9
4. <u>Containment Pressure - High-High</u>	
CSAS	
Containment Spray	21.0*

Table 3.3-5 (Continued)

INITIATING SIGNAL AND FUNCTION	RESPONSE TIME (SEC)
5. <u>Steam Generator Pressure - Low</u>	
MSIS	
(1) Main Steam Isolation	20.9
(2) Main Feedwater Isolation	10.9
6. <u>Refueling Water Storage Tank - Low</u>	
RAS	
(1) Containment Sump Valves Open	50.7*
(2) ECCS Miniflow Valves Shut	50.7*
7. <u>4.16 kv Emergency Bus Undervoltage</u>	
LOV (loss of voltage and degraded voltage)	Figure 3.3-1
8. <u>Steam Generator Level - Low (and No Pressure-Low Trip)</u>	
EFAS	
(1) Auxiliary Feedwater (AC trains)	50.9*/40.9**
(2) Auxiliary Feedwater (steam/DC train)	30.9 (NOTE 6)
9. <u>Steam Generator Level - Low (and ΔP - High)</u>	
EFAS	
(1) Auxiliary Feedwater (AC trains)	50.9*/40.9**
(2) Auxiliary Feedwater (Steam/DC train)	30.9 (NOTE 6)
10. <u>Control Room Ventilation Airborne Radiation</u>	
CRIS	
(1) Control Room Ventilation - Emergency Mode	Not Applicable
11. <u>Control Room Toxic Gas (Chlorine)</u>	
TGIS	
(1) Control Room Ventilation - Isolation Mode	16 (NOTE 5)
12. <u>Control Room Toxic Gas (Ammonia)</u>	
TGIS	
Control Room Ventilation - Isolation Mode	36 (NOTE 5)

Table 3.3-5 (Continued)

<u>INITIATING SIGNAL AND FUNCTION</u>	<u>RESPONSE TIME (SEC)</u>
13. <u>Control Room Toxic Gas (Butane/Propane)</u>	
TGIS	
Control Room Ventilation - Isolation Mode	36 (NOTE 5)
14. <u>Control Room Toxic Gas (Carbon Dioxide)</u>	
TGIS	
Control Room Ventilation - Isolation Mode	36 (NOTE 5)
15. <u>Fuel Handling Building Airborne Radiation</u>	
FHIS	
Fuel Handling Building Post-Accident Cleanup Filter System	Not Applicable
16. <u>Containment Airborne Radiation</u>	
CPIS	
Containment Purge Isolation	2 (NOTE 2)
17. <u>Containment Area Radiation</u>	
CPIS	
Containment Purge Isolation	2 (NOTE 2)

NOTES:

1. Response times include movement of valves and attainment of pump or blower discharge pressure as applicable.
 2. Response time includes emergency diesel generator starting delay (applicable to A.C. motor-operated valves other than containment purge valves), instrumentation and logic response only. Refer to Table 3.6-1 for containment isolation valve closure times.
 3. All CIAS-actuated valves except MSIVs, MFIVs, and CCW Valves 2HV-6211 and 2HV-6216.
 - 4a. CCW noncritical loop isolation Valves 2HV-6212, 2HV-6213, 2HV-6218, and 2HV-6219 close.
 - 4b. Containment emergency cooler CCW isolation Valves 2HV-6366, 2HV-6367, 2HV-6368, 2HV-6369, 2HV-6370, 2HV-6371, 2HV-6372, and 2HV-6373 open.
 5. Response time includes instrumentation, logic, and isolation damper closure times only.
 6. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.
- * Emergency diesel generator starting delay (10 sec.) and sequence loading delays for SIAS are included.
- ** Emergency diesel generator starting delay (10 sec.) is included.

SAFETY EVALUATION
 AMENDMENT NO. 12 TO NPF-10
 SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2
 DOCKET NO. 50-361

Introduction

By letters dated December 20, 21, and 22, 1982, Southern California Edison Company (SCE) on behalf of itself and the other licensees (San Diego Gas and Electric Company, the City of Anaheim and the City of Riverside), requested a revision to Technical Specification 3/4.3.2, Table 3.3-5. The proposed change will delete the requirement that the component cooling water (CCW) non-critical loop containment isolation valves and the CCW critical/non-critical loop isolation valves isolate on low pressurizer pressure. All these valves will continue to isolate on high containment pressure (about 3 psig). The CCW critical/non-critical loop isolation valves will continue to isolate on low-low CCW surge tank level.

SCE states that removal of the Pressurizer Pressure-Low isolation signal from these valves will permit the CCW system to continue cooling non-critical loop loads such as the Reactor Coolant Pump (RCP) motors and seals and the Control Element Drive Mechanism (CEDM) windings during certain transient events. Under the previous Technical Specifications, these transient events would unnecessarily require that cooling to the RCP motor and seal and CEDM winding be terminated. SCE states that the continued cooling allowed by the proposed change to the Technical Specification will minimize cumulative damage to the RCP pumps seals caused by unnecessary interruptions of CCW. Minimization of such cumulative damage to the RCP seals will increase the availability of the RCPs and reduce the probability of RCP seal failures.

Evaluation of the Proposed Change

As indicated by SCE in their letters, the proposed change to the Technical Specifications will only affect the response of the CCW system for those transient events which result in Pressurizer Pressure-Low but not Containment Pressure-High. Such events are pressurizer pressure control system failures, main steam or feedwater system control system or piping failures outside containment, and small steam, feedwater and reactor coolant system piping failures inside containment. For these events, the following analyses and conclusions are presented by SCE:

- (1) The CCW System design has been reviewed and it has been verified that flow and heat capacity are adequate to simultaneously serve all essential and non-essential loads with the exception of the Shutdown Cooling Heat Exchangers (SDCHX). The SDCHX's are isolated until they receive the Containment Spray Actuation Signal at approximately 16 psig containment pressure. Because the proposed signal to isolate the essential from the non-essential loads occurs at approximately 3 psig containment pressure (on Containment Isolation Actuation Signal), the CCW system capacity will not be exceeded.

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- (2) Isolation of the critical CCW loops from the non-critical loop occurs on low-low CCW surge tank level, thereby protecting the critical loops from non-critical loop failures. SCE states that each of the safety analyses of Section 15 of the FSAR has been examined, and the proposed change will not impact the analyses.
- (3) While the proposed change will reduce the diversity of the actuation signal to the non-critical loop CCW containment isolation valves and the critical/non-critical CCW isolation valves, the overall result of the change will be to allow operation of the RCPs during a wider range of transient events. Since RCP operation can mitigate the consequences of many accident sequences, SCE argues that the proposed change results in increased plant safety, on balance.

The NRC staff has reviewed the SCE letters, and has discussed the issue with SCE in a meeting on December 21, 1982. The staff concurs with SCE's conclusions, for the reasons given above. Consequently, the staff finds the proposed changes to Technical Specification 3/4.3.2 to be acceptable.

Evaluation of Related Issues

During the course of the staff's review of the proposed change, it became clear that the non-critical CCW containment penetrations do not meet the applicable staff criteria or the criteria defined in the FSAR. Specifically, the non-critical CCW loop can not be shown to meet the criteria specified in the FSAR for systems which meet General Design Criterion (GDC) 57, since the loop inside containment is not missile and pipe-whip protected, and the components served are not seismic Category I. Therefore, the isolation provisions must meet GDC 56, which requires two automatic isolation valves for each line penetrating containment. The present design has two isolation valves per line, but only one of the valves isolates automatically. The other has remote-manual actuation.

At a meeting on December 22, 1982, this issue was discussed with SCE. By its letter dated December 22, 1982, SCE committed to correct the situation within 90 days by changing the remote-manual valves (HV-6223 and 6236) to automatic isolation. The 90 day time period was justified on the basis of material delivery time and installation and testing time. In the interim, justification for continued operation will be based on procedures requiring operator verification of non-critical loop isolation, and operator action to close the remote-manual valves should the automatic valves fail to close.

SCE further stated that the remote-manual valves will be unlocked to allow closure by the operators, rather than locked open as stated in the FSAR.

The NRC staff has reviewed the SCE letter of December 22, 1982, and has discussed the issue with SCE, and has concluded that operation prior to installation of the automatic isolation signal to valves HV-6223 and 6236 is acceptable, based on (1) automatic isolation of valves HV-6211 and 6216 which are in series with HV-6223 and 6236, and (2) the ability to promptly isolate valves HV-6223 and 6236 from the control room should the need arise.

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Environmental Consideration

The NRC staff has determined that this amendment does not authorize a change in effluent types or total amount nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that this amendment involves action which is insignificant from the standpoint of environmental impact and pursuant 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

Based upon our evaluation of the proposed changes to the San Onofre, Unit 2 Technical Specifications, we have concluded that: (1) because this amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, this amendment does not involve a significant safety hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of public. We, therefore, conclude that the proposed change is acceptable.

Dated: DEC 23 1982

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-361

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL

NOTICE OF ISSUANCE OF AMENDMENT

FACILITY OPERATING LICENSE NO. NPF-10

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. NPF-10, issued to Southern California Edison Company, San Diego Gas and Electric Company, The City of Riverside, California and The City of Anaheim, California (licensees) for the San Onofre Nuclear Generating Station, Unit 2 (the facility) located in San Diego County, California. This amendment is effective December 22, 1982.

Amendment No. 12 changes the technical specifications to allow the non-critical component cooling water (CCW) loop to circulate water to the reactor coolant pumps after safety injection initiation. The non-critical CCW loop will still be required to isolate on high containment pressure.

Issuance of this amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) Southern California Edison Company's letters dated December 20 and 21, 1982, (2) Amendment No. 12 to Facility Operating License No. NPF-10, and (3) the Commission's related Safety Evaluation.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and the San Clemente Library, 242 Avenida Del Mar, San Clemente, California 92672. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this day of December, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

~~Original Signed By:~~

George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing

W. Knighton
Special

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Document Control (50-361)
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) Southern California Edison Company's letters dated December 20, 21 and 22, 1982, (2) Amendment No. 12 to Facility Operating License No. NPF-10, and (3) the Commission's related Safety Evaluation.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and the San Clemente Library, 242 Avenida Del Mar, San Clemente, California 92672. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this **23** day of December, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Victor Merses, Acting Chief
Licensing Branch No. 3
Division of Licensing

*See previous concurrences/initials *DL*. 12/23/82

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