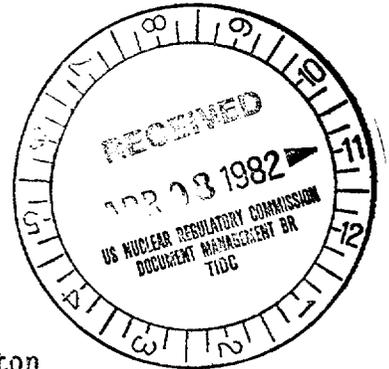


March 30, 1982

Docket No: 50-361



Mr. Robert Dietch
Vice President
Southern California Edison Company
2244 Walnut Grove Avenue
Post Office Box 800
Rosemead, California 91770

Mr. Gary D. Cotton
Mr. Louis Bernath
San Diego Gas & Electric Company
101 Ash Street
Post Office Box 1831
San Diego, California 92112

Dear Gentlemen:

Subject: Issuance of Amendment No. 1 to Facility Operating License MPF-10
San Onofre Nuclear Generating Station, Unit 2

The Nuclear Regulatory Commission has issued Amendment No. 1 to Facility Operating License MPF-10 for the San Onofre Nuclear Station, Unit 2, located in San Diego County, California.

This amendment is in response to your letter dated March 12, 1982. The amendment clarifies the Technical Specification setpoint requirements for the Shutdown Cooling System relief valve, and is effective as of March 13, 1982. Telephone approval was given to Southern California Edison Company on March 13, 1982 authorizing the implementation of the proposed change.

A copy of the related safety evaluation supporting Amendment No. 1 to Facility Operating License MPF-10 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original signed by
Frank J. Miraglia

Frank J. Miraglia, Chief
Licensing Branch No. 3
Division of Licensing

Enclosures:

1. Amendment No. 1
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures:
See next page

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hcb/bd

MM

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SURNAME	JLee:ph	HRood	RBosnak	BSheron	FMiraglia		
DATE	3/19/82	3/19/82	3/19/82	3/19/82	3/19/82		

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-361

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-10

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the San Onofre Nuclear Generating Station, Unit 2 (the facility) Facility Operating License No. NPF-10 filed by the Southern California Edison Company on behalf of itself and San Diego Gas and Electric Company, The City of Riverside and The City of Anaheim, California (licensees) dated March 12, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-10 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 1, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of March 13, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
/s/ Frank J. Miraglia

Frank J. Miraglia, Chief
Licensing Branch No. 3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance:

OFFICE	DL:LB#3	DL:LB#3	DE:MER	DST:RSB	DL:LB#3		
SURNAME	JLeech	HRood	RBosnk	BSaeron	FMiraglia		
DATE	3/19/82	3/30/82	3/23/82	3/30/82	3/30/82		

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-10

DOCKET NO. 50-361

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to main document completeness.

<u>Overleaf</u> <u>Page</u>	<u>Amended</u> <u>Page</u>
3/4 4-31	3/4 4-32
3/4 4-34	3/4 4-33

REACTOR COOLANT SYSTEM

OVERPRESSURE PROTECTION SYSTEMS

RCS TEMPERATURE > 235°F

LIMITING CONDITION FOR OPERATION

3.4.8.3.2 - At least one of the following overpressure protection systems shall be OPERABLE:

- a. The Shutdown Cooling System (SDCS) Relief Valve (PSV 9349) with a setting of 406 ± 10 psig**, or,
- b. A minimum of one pressurizer code safety valve with a lift setting of 2500 psia $\pm 1\%$.

APPLICABILITY: MODE 4 with RCS temperature above 235°F.

ACTION:

With no safety or relief valve operable, be in COLD SHUTDOWN and vent the RCS through a greater than or equal to 5.6 square inch vent within the next 8 hours.

SURVEILLANCE REQUIREMENTS

4.4.8.3.2.1 The SDCS Relief Valve shall be demonstrated OPERABLE by:

- a. Verifying at least once per 72 hours that the SDCS Relief Valve isolation valves are open when the SDCS Relief Valve is being used for overpressure protection.
- b. Testing pursuant to Specification 4.0.5 with an inservice test interval of at least once per 30 months.

4.4.8.3.2.2 The pressurizer code safety valve has no additional surveillance requirements other than those required by Specification 4.0.5.

4.4.8.3.2.3 The RCS vent shall be verified to be open at least once per 12 hours when the vent is being used for overpressure protection, except when the vent pathway is provided with a valve which is locked, sealed, or otherwise secured in the open position, then verify these valves open at least once per 31 days.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

**For relief valve temperatures less than or equal to 130°F.

REACTOR COOLANT SYSTEM

3.4.9 STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.9 The structural integrity of ASME Code Class 1, 2 and 3 components shall be maintained in accordance with Specification 4.4.9.

APPLICABILITY: ALL MODES

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component to within its limit or isolate the affected component from service.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.9 In addition to the requirements of Specification 4.0.5, each Reactor Coolant Pump flywheel shall be inspected per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

REACTOR COOLANT SYSTEM

OVERPRESSURE PROTECTION SYSTEMS

RCS TEMPERATURE \leq 235°F

LIMITING CONDITION FOR OPERATION

3.4.8.3.1 At least one of the following overpressure protection systems shall be OPERABLE:

- a. The Shutdown Cooling System (SDCS) Relief Valve (PSV9349) with a lift setting of 406 ± 10 psig*, or,
- b. The Reactor Coolant System depressurized with an RCS vent of greater than or equal to 5.6 square inches.

APPLICABILITY: MODE 4 when the temperature of one any RCS cold leg is less than or equal to 235°F; Mode 5; Mode 6 with the reactor vessel head on.

ACTION:

- a. With the SDCS Relief Valve inoperable, reduce T_{avg} to less than 200°F, depressurize and vent the RCS through a greater than or equal to 5.6 square inch vent within the next 8 hours.
- b. With one or both SDCS Relief Valve isolation valves in a single SDCS Relief Valve isolation valve pair (valve pair 2HV9337 and 2HV9339 or valve pair 2HV9377 and 2HV9378) closed, open the closed valve(s) within 7 days or reduce T_{avg} to less than 200°F, depressurize and vent the RCS through a greater than or equal to 5.6 inch vent within the next 8 hours.
- c. In the event either the SDCS Relief Valve or an RCS vent is used to mitigate a RCS pressure transient, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 30 days. The report shall describe the circumstances initiating the transient, the effect of the SDCS Relief Valve or RCS vent on the transient and any corrective action necessary to prevent recurrence.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.8.3.1.1 The SDCS Relief Valve shall be demonstrated OPERABLE by:

- a. Verifying at least once per 72 hours when the SDCS Relief Valve is being used for overpressure protection that at least one pair of SDCS Relief Valve isolation valves (valve pair 2HV9337 and 2HV9339 or valve pair 2HV9377 and 2HV9378) is open.

*For relief valve temperatures less than or equal to 130°F.

REACTOR COOLANT SYSTEM

PRESSURIZER

LIMITING CONDITION FOR OPERATION

- 3.4.8.2 The pressurizer temperature shall be limited to:
- a. A maximum heatup of 200°F in any one hour period,
 - b. A maximum cooldown of 200°F in any one hour period.

APPLICABILITY: At all times.

ACTION:

With the pressurizer temperature limits in excess of any of the above limits, restore the temperature to within the limits within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the pressurizer; determine that the pressurizer remains acceptable for continued operation or be in at least HOT STANDBY within the next 6 hours and reduce the pressurizer pressure to less than 500 psig within the following 30 hours.

SURVEILLANCE REQUIREMENTS

- 4.4.8.2.1 The pressurizer temperatures shall be determined to be within the limits at least once per 30 minutes during system heatup or cooldown.
- 4.4.8.2.2 The spray water temperature differential shall be determined for use in Table 5.7-1 at least once per 12 hours during auxiliary spray operation.

SAFETY EVALUATION

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2

DOCKET NO. 50-361

Introduction

By letter dated March 12, 1982 the Southern California Edison Company, the licensee, requested a change to the San Onofre Nuclear Generating Station, Unit 2 Technical Specifications, Section 3.4.8.3.1.a (Overpressure Protection Systems), to clarify the ASME Section III Code setpoint requirements for the Shutdown Cooling System (SDCS) relief valve (PSV 9349). The Technical Specifications currently require that the SDCS relief valve left setpoint be equal to or less than 402 psig. The licensee's proposal is to change the relief valve setpoint to 406 + 10 psig at ambient temperature.

Evaluation

By letter dated March 12, 1982 the licensee requested that the SDCS relief valve lift setpoint be changed from 402 psig to 406 + 10 psig. The ASME Section III Code left setpoint of the SDCS relief valve, as installed, is 402 psig at a design temperature of 400 F and 406 psig at ambient temperature. The proposed setpoint of 406 + 10 psig envelopes the required relief valve lift setpoints to provide the required reactor coolant system overpressure protection for the 0-5 year reactor coolant system pressure-temperature curves and is consistent with the ASME Code tolerances for these valves in this pressure range.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

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Conclusion

Based upon our evaluation of the proposed change to the San Onofre, Unit 2 Technical Specifications, we have concluded, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public. We, therefore, conclude that the proposed changes are acceptable.

Dated: March 30, 1982

OFFICE	DL: [Signature]	DL: [Signature]					
SURNAME	HRood:ph	EM Maglia					
DATE	3/30/82	3/30/82					

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-361

SOUTHERN CALIFORNIA EDISON COMPANY, ET AL

NOTICE OF ISSUANCE OF AMENDMENT

FACILITY OPERATING LICENSE NO. NPF-10

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to Facility Operating License No. NPF-10, issued to Southern California Edison Company, San Diego Gas and Electric Company, The City of Riverside, California and The City of Anaheim, California (licensees) for the San Onofre Nuclear Generating Station, Unit 2 (the facility) located in San Diego County, California. The amendment is effective as of March 13, 1982.

This amendment clarifies the Technical Specification setpoint requirements for the Shutdown Cooling System relief valve.

Issuance of this amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d) (4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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For further details with respect to this action, see (1) Southern California Edison Company's letter dated March 12, 1982, (2) Amendment No. 1 to Facility Operating License No. NPF-10 and (3) the Commission's related Safety Evaluation.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Mission Viejo Branch Library, 24851 Chrisanta Drive Mission Viejo, California 02676. A copy of these items may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 30th day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
Frank J. Miraglia

Frank J. Miraglia, Chief
Licensing Branch No. 3
Division of Licensing

Frank J. Miraglia
D. Form of Notice
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DATE	3/19/82	3/2/82	3/1/82	3/8/82	3/1/82	3/1/82	

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