

November 9, 1995

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: POST-ACCIDENT MONITORING TECHNICAL
SPECIFICATIONS (TAC NOS. M91336 AND M91337)

Dear Mr. Anderson:

The Commission has issued the enclosed Amendment No. 121 to Facility Operating License No. DPR-42 and Amendment No. 114 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 10, 1995, as supplemented August 9 and September 20, 1995.

The amendments incorporate Limiting Condition for Operation (LCO) requirements from NUREG-1431, "Standard Technical Specifications (STS) for Westinghouse Plants," dated September 1992, into the plant TS Section 3.15, Event Monitoring Instrumentation. The amendments also make necessary changes in Table 4.1-1C, Miscellaneous Instrumentation Surveillance Requirements, and BASES 3.15, Event Monitoring Instrumentation, to reflect TS Section 3.15 changes.

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No. 121 to DPR-42
2. Amendment No. 114 to DPR-60
3. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\WPDOCS\PI91336.AMD

To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure
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DATE	10/25/95		10/25/95		10/30/95	11/9/95

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Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

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DATED: November 9, 1995

AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NO. DPR-42-PRAIRIE ISLAND UNIT 1
AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-60-PRAIRIE ISLAND UNIT 2

Docket File

PUBLIC

PDIII-1 Reading

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C. Jamerson

B. Wetzel (2)

OGC-WF

G. Hill (4)

C. Grimes, O-11F23

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cc: Plant Service list

100050

DF011



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. DPR-42

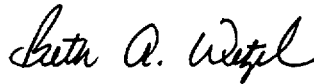
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 10, 1995, as supplemented August 9, and September 20, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 121, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 9, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

INSERT

TS iv

TS iv

TS xii

TS xii

TS 3.15-1

TS 3.15-1

TS 3.15-2

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Table 3.15-1

Table 3.15-1 (p. 1 of 2)

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Table 3.15-1 (p. 2 of 2)

Table 3.15-2

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Table 3.15-3

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Table 4.1-1C (p. 2 of 4)

Table 4.1-1C (p. 2 of 4)

Table 4.1-1C (p. 3 of 4)

Table 4.1-1C (p. 3 of 4)

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Bases 3.15-1

Bases 3.15-1

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	B. Power Distribution Limits	TS.3.10-1
	C. Quadrant Power Tilt Ratio	TS.3.10-4
	D. Rod Insertion Limits	TS.3.10-5
	E. Rod Misalignment Limitations	TS.3.10-6
	F. Inoperable Rod Position Indicator Channels	TS.3.10-6
	G. Control Rod Operability Limitations	TS.3.10-7
	H. Rod Drop Time	TS.3.10-7
	I. Monitor Inoperability Requirements	TS.3.10-8
	J. DNB Parameters	TS.3.10-8
3.11	Core Surveillance Instrumentation	TS.3.11-1
3.12	Snubbers	TS.3.12-1
3.13	Control Room Air Treatment System	TS.3.13-1
	A. Control Room Special Ventilation System	TS.3.13-1
3.14	Deleted	
3.15	Event Monitoring Instrumentation	TS.3.15-1

TECHNICAL SPECIFICATIONSLIST OF TABLES

<u>TS TABLE</u>	<u>TITLE</u>
1-1	Operational Modes
3.5-1	Engineered Safety Features Initiation Instrument Limiting Set Points
3.5-2A	Reactor Trip System Instrumentation
3.5-2B	Engineered Safety Feature Actuation System Instrumentation
3.9-1	Radioactive Liquid Effluent Monitoring Instrumentation
3.9-2	Radioactive Gaseous Effluent Monitoring Instrumentation
3.15-1	Event Monitoring Instrumentation
4.1-1A	Reactor Trip System Instrumentation Surveillance Requirements
4.1-1B	Engineered Safety Feature Actuation System Instrumentation Surveillance Requirements
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4.1-2A	Minimum Frequencies for Equipment Tests
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4.2-1	Special Inservice Inspection Requirements
4.10-1	Radiation Environmental Monitoring Program (REMP) Sample Collection and Analysis
4.10-2	RFMP - Maximum Values for the Lower Limits of Detection
4.10-3	RFMP - Reporting Levels for Radioactivity Concentrations in Environmental Samples
4.12-1	Steam Generator Tube Inspection
4.13-1	Snubber Visual Inspection Interval
4.17-1	Radioactive Liquid Effluent Monitoring Instrumentation Surveillance Requirements
4.17-2	Radioactive Gaseous Effluent Monitoring Instrumentation Surveillance Requirements
4.17-3	Radioactive Liquid Waste Sampling and Analysis Program
4.17-4	Radioactive Gaseous Waste Sampling and Analysis Program
5.5-1	Anticipated Annual Release of Radioactive Material in Liquid Effluents From Prairie Island Nuclear Generating Plant (Per Unit)
5.5-2	Anticipated Annual Release of Radioactive Nuclides in Gaseous Effluent From Prairie Island Nuclear Generating Plant (Per Unit)
6.1-1	Minimum Shift Crew Composition

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 111, 120, 121
Amendment No. 104, 113, 114

3.15 EVENT MONITORING INSTRUMENTATION

Applicability

Applies during MODES 1, and 2.

Objective

To ensure that sufficient information is available to operators to determine the effects of and determine the course of an accident to the extent required to carry out required manual actions.

Specification

- A. Event monitoring instrumentation shall be OPERABLE as specified in Table TS.3.15-1.
- B. Action statements applicable by reference from Table TS.3.15-1.
- C. MODES 1, and 2 may be entered when a LIMITING CONDITION FOR OPERATION is not met.
- D. The provisions of specification 3.0.C are not applicable.

TABLE TS.3.15-1 (Page 1 of 2)
EVENT MONITORING INSTRUMENTATION

<u>Function</u>	<u>Required Channels</u>	<u>Action</u> ^(a)
1. Power Range Neutron Flux (Logarithmic Scale)	2	1, 2
2. Source Range Neutron Flux (Logarithmic Scale)	2	1, 2
3. Reactor Coolant System (RCS) Hot Leg Temperature	2	1, 2
4. RCS Cold Leg Temperature	2	1, 2
5. RCS Pressure (Wide Range)	2	1, 2
6. Reactor Vessel Water Level	2	1, 3
7. Containment Sump Water Level (Wide Range)	2	1, 2
8. Containment Pressure (Wide Range)	2	1, 2
9. Automatic Containment Isolation Valve Position	2 per penetration flow path ^{(b)(c)}	1, 2
10. Containment Area Radiation (High Range)	2	1, 3
11. Hydrogen Monitors	2	1, 4
12. Pressurizer Level	2	1, 2
13. Steam Generator Water Level (Wide Range)	2 per steam generator	1, 2
14. Condensate Storage Tank Level	2	1, 2
15. Core Exit Thermocouples	4 per core quadrant	5, 6
16. Refueling Water Storage Tank Level	2	1, 2

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 68, 78, 121
Amendment No. 62, 71, 114

TABLE TS.3.15-1 (Page 2 of 2)
EVENT MONITORING INSTRUMENTATION

(a) Action Statements

Separate Action Statement entry is allowed for each Function.

1. With one required channel inoperable, either restore the required channel to OPERABLE status within 30 days, or submit a report to the Commission within the following 14 days.
 2. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days or be in at least MODE 3 within the next 6 hours.
 3. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days, or submit a report to the Commission within the following 14 days.
 4. With two required channels inoperable, either restore one channel to OPERABLE status within 72 hours or be in at least MODE 3 within the next 6 hours.
 5. With the number of OPERABLE channels for the core exit thermocouples less than the Required Channels shown on Table TS.3.15-1, but with greater than or equal to 4 core exit thermocouples OPERABLE in the center core region and greater than or equal to one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 30 days, or submit a report to the Commission within the next 14 days. As a minimum, the Required Channels will be restored prior to startup following the next refueling outage.
 6. With the less than two core exit thermocouple channels OPERABLE in one or more quadrants, and with either less than 4 core exit thermocouples OPERABLE in the center region or less than one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 7 days, or be in at least MODE 3 within the next 6 hours.
- (b) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, or blind flange.
- (c) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

TABLE TS.4.1-1C (Page 2 of 4)

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
13. Containment Sump A, B and C Level	N.A.	R	R	N.A.	1, 2, 3, 4
14. Deleted					
15. Turbine First Stage Pressure	S	R	Q	N.A.	1
16. Emergency Plan Radiation Instruments ⁽³⁵⁾	M	R	M	N.A.	1, 2, 3, 4, 5, 6
17. Seismic Monitors	R	R	N.A.	N.A.	1, 2, 3, 4, 5, 6
18. Coolant Flow - RTD Bypass Flowmeter	S	R	M	N.A.	1, 2, 3 ⁽³⁴⁾
19. CRDM Cooling Shroud Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3 ⁽³¹⁾ , 4 ⁽³¹⁾ , 5 ⁽³¹⁾
20. Reactor Gap Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3, 4
21. Post-Accident Monitoring Instruments (Table TS.3.15-1) ⁽³⁶⁾	M	R	N.A.	N.A.	1, 2
22. Deleted					

TABLE TS.4.1-1C (Page 3 of 4)

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
23. Deleted					
24. Steam Exclusion Actuation	W	Y	M	N.A.	1, 2, 3
25. Overpressure Mitigation	N.A.	R	R	N.A.	4 ⁽³⁸⁾ , 5
26. Auxiliary Feedwater Pump Suction Pressure	N.A.	R	R	N.A.	1, 2, 3,
27. Auxiliary Feedwater Pump Discharge Pressure	N.A.	R	R	N.A.	1, 2, 3
28. NaOH Caustic Stand Pipe Level	W	R	M	N.A.	1, 2, 3, 4
29. Hydrogen Monitors	S	Q	M	N.A.	1, 2
30. Containment Temperature Monitors	M	R	N.A.	N.A.	1, 2, 3, 4
31. Turbine Overspeed Protection Trip Channel	N.A.	R	M	N.A.	1

Prairie Island Unit 1
Prairie Island Unit 2Amendment No. 78, 111, 121
Amendment No. 88, 104, 114Table TS.4.1-1C
(Page 3 of 4)

TABLE 4.1-1C (Page 4 of 4)

TABLE NOTATIONS

FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	Shift
D	Daily
W	Weekly
M	Monthly
Q	Quarterly
S/U	Prior to each startup
Y	Yearly
R	Each refueling shutdown
N.A.	Not Applicable

TABLE NOTATION

- | | |
|--|---|
| <p>(30) Prior to each startup following shutdown in excess of two days if not done in previous 30 days.</p> <p>(31) When the reactor trip system breakers are closed and the control rod drive system is capable of rod withdrawal.</p> <p>(32) Following rod motion in excess of six inches when the computer is out of service.</p> <p>(33) Transfer logic to Refueling Water Storage Tank.</p> <p>(34) When either main steam isolation valve is open.</p> <p>(35) Includes those instruments named in the emergency procedure.</p> | <p>(36) Except for containment hydrogen monitors and refueling water storage tank level which are separately specified in this table.</p> <p>(37) When RHR is in operation.</p> <p>(38) When the reactor coolant system average temperature is less than 310°F.</p> <p>(39) Whenever CONTAINMENT INTEGRITY is required.</p> |
|--|---|

3.15 EVENT MONITORING INSTRUMENTATIONBases

The OPERABILITY of the event monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, NUREG-0737 and the Regulatory Guide 1.97 evaluation of Prairie Island instrumentation.

The Action Statements for one or more inoperable channels in some circumstances require submittal of a report to the Commission. The report shall outline preplanned alternate method of monitoring as applicable, the cause of the inoperability, and the plans and schedule for restoring the instrument channels of the Function to OPERABLE status.

Some containment penetration flow paths may have one valve with position indication based on the justification that another mechanism is provided to assure containment integrity is maintained. In the event the position indication on the one valve fails, the justification for the penetration to have a single valve with position indication is assumed to remain valid. Since another mechanism continues to provide containment integrity, the applicable action is Action Statement 1.

The following core exit thermocouples are included in the center core region referenced in Table TS.3.15-1, Action Statement 5. If a thermocouple is not listed below, it is located in the outside core region.

<u>Thermocouple Number</u>	<u>Core Location</u>
9	D-5
10	D-7
12	E-4
13	E-6
14	E-10
16	F-7
18	G-4
19	G-6
22	H-5
23	H-9
28	I-4
29	I-8
30	I-10
32	J-6
33	J-8
34	J-9

Core exit thermocouple readings necessary to meet the requirements of Specification 3.15.A are available from the Plant Process Computer, the Control Room Core Exit Thermocouple Display or if no other readout is available, from test equipment readings from the Core Exit Thermocouple Junction Boxes.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114
License No. DPR-60

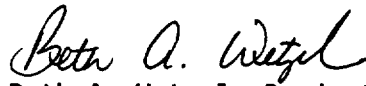
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 10, 1995, as supplemented August 9, and September 20, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 9, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 114

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

INSERT

TS iv

TS iv

TS xii

TS xii

TS 3.15-1

TS 3.15-1

TS 3.15-2

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Table 3.15-1

Table 3.15-1 (p. 1 of 2)

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Table 3.15-1 (p. 2 of 2)

Table 3.15-2

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Table 3.15-3

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Table 4.1-1C (p. 2 of 4)

Table 4.1-1C (p. 2 of 4)

Table 4.1-1C (p. 3 of 4)

Table 4.1-1C (p. 3 of 4)

Table 4.1-1C (p. 4 of 4)

Table 4.1-1C (p. 4 of 4)

Bases 3.15-1

Bases 3.15-1

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	D. Rod Insertion Limits	TS.3.10-5
	E. Rod Misalignment Limitations	TS.3.10-6
	F. Inoperable Rod Position Indicator Channels	TS.3.10-6
	G. Control Rod Operability Limitations	TS.3.10-7
	H. Rod Drop Time	TS.3.10-7
	I. Monitor Inoperability Requirements	TS.3.10-8
	J. DNB Parameters	TS.3.10-8
3.11	Core Surveillance Instrumentation	TS.3.11-1
3.12	Snubbers	TS.3.12-1
3.13	Control Room Air Treatment System	TS.3.13-1
	A. Control Room Special Ventilation System	TS.3.13-1
3.14	Deleted	
3.15	Event Monitoring Instrumentation	TS.3.15-1

TECHNICAL SPECIFICATIONSLIST OF TABLES

<u>TS TABLE</u>	<u>TITLE</u>
1-1	Operational Modes
3.5-1	Engineered Safety Features Initiation Instrument Limiting Set Points
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3.5-2B	Engineered Safety Feature Actuation System Instrumentation
3.9-1	Radioactive Liquid Effluent Monitoring Instrumentation
3.9-2	Radioactive Gaseous Effluent Monitoring Instrumentation
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4.17-3	Radioactive Liquid Waste Sampling and Analysis Program
4.17-4	Radioactive Gaseous Waste Sampling and Analysis Program
5.5-1	Anticipated Annual Release of Radioactive Material in Liquid Effluents From Prairie Island Nuclear Generating Plant (Per Unit)
5.5-2	Anticipated Annual Release of Radioactive Nuclides in Gaseous Effluent From Prairie Island Nuclear Generating Plant (Per Unit)
6.1-1	Minimum Shift Crew Composition

Prairie Island Unit 1
Prairie Island Unit 2

Amendment No. 111, 120, 121
Amendment No. 104, 113, 114

3.15 EVENT MONITORING INSTRUMENTATION

Applicability

Applies during MODES 1, and 2.

Objective

To ensure that sufficient information is available to operators to determine the effects of and determine the course of an accident to the extent required to carry out required manual actions.

Specification

- A. Event monitoring instrumentation shall be OPERABLE as specified in Table TS.3.15-1.
- B. Action statements applicable by reference from Table TS.3.15-1.
- C. MODES 1, and 2 may be entered when a LIMITING CONDITION FOR OPERATION is not met.
- D. The provisions of specification 3.0.C are not applicable.

TABLE TS.3.15-1 (Page 1 of 2)
EVENT MONITORING INSTRUMENTATION

<u>Function</u>	<u>Required Channels</u>	<u>Action^(a)</u>
1. Power Range Neutron Flux (Logarithmic Scale)	2	1, 2
2. Source Range Neutron Flux (Logarithmic Scale)	2	1, 2
3. Reactor Coolant System (RCS) Hot Leg Temperature	2	1, 2
4. RCS Cold Leg Temperature	2	1, 2
5. RCS Pressure (Wide Range)	2	1, 2
6. Reactor Vessel Water Level	2	1, 3
7. Containment Sump Water Level (Wide Range)	2	1, 2
8. Containment Pressure (Wide Range)	2	1, 2
9. Automatic Containment Isolation Valve Position	2 per penetration flow path ^{(b)(c)}	1, 2
10. Containment Area Radiation (High Range)	2	1, 3
11. Hydrogen Monitors	2	1, 4
12. Pressurizer Level	2	1, 2
13. Steam Generator Water Level (Wide Range)	2 per steam generator	1, 2
14. Condensate Storage Tank Level	2	1, 2
15. Core Exit Thermocouples	4 per core quadrant	5, 6
16. Refueling Water Storage Tank Level	2	1, 2

Table TS.3.15-1
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Prairie Island Unit 1
 Prairie Island Unit 2

Amendment No. 68, 121
 Amendment No. 62, 71, 114

TABLE TS.3.15-1 (Page 2 of 2)
EVENT MONITORING INSTRUMENTATION

(a) Action Statements

Separate Action Statement entry is allowed for each Function.

1. With one required channel inoperable, either restore the required channel to OPERABLE status within 30 days, or submit a report to the Commission within the following 14 days.
 2. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days or be in at least MODE 3 within the next 6 hours.
 3. With two required channels inoperable, either restore one channel to OPERABLE status within 7 days, or submit a report to the Commission within the following 14 days.
 4. With two required channels inoperable, either restore one channel to OPERABLE status within 72 hours or be in at least MODE 3 within the next 6 hours.
 5. With the number of OPERABLE channels for the core exit thermocouples less than the Required Channels shown on Table TS.3.15-1, but with greater than or equal to 4 core exit thermocouples OPERABLE in the center core region and greater than or equal to one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 30 days, or submit a report to the Commission within the next 14 days. As a minimum, the Required Channels will be restored prior to startup following the next refueling outage.
 6. With the less than two core exit thermocouple channels OPERABLE in one or more quadrants, and with either less than 4 core exit thermocouples OPERABLE in the center region or less than one core exit thermocouple OPERABLE in each quadrant of the outside core region, restore the inoperable channels to OPERABLE status within 7 days, or be in at least MODE 3 within the next 6 hours.
- (b) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, or blind flange.
- (c) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
13. Containment Sump A, B and C Level	N.A.	R	R	N.A.	1, 2, 3, 4
14. Deleted					
15. Turbine First Stage Pressure	S	R	Q	N.A.	1
16. Emergency Plan Radiation Instruments ⁽³⁵⁾	M	R	M	N.A.	1, 2, 3, 4, 5, 6
17. Seismic Monitors	R	R	N.A.	N.A.	1, 2, 3, 4, 5, 6
18. Coolant Flow - RTD Bypass Flowmeter	S	R	M	N.A.	1, 2, 3 ⁽³⁴⁾
19. CRDM Cooling Shroud Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3 ⁽³¹⁾ , 4 ⁽³¹⁾ , 5 ⁽³¹⁾
20. Reactor Gap Exhaust Air Temperature	S	N.A.	R	N.A.	1, 2, 3, 4
21. Post-Accident Monitoring Instruments (Table TS.3.15-1) ⁽³⁶⁾	M	R	N.A.	N.A.	1, 2
22. Deleted					

TABLE TS.4.1-1C (Page 3 of 4)

MISCELLANEOUS INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>FUNCTIONAL TEST</u>	<u>RESPONSE TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
23. Deleted					
24. Steam Exclusion Actuation	W	Y	M	N.A.	1, 2, 3
25. Overpressure Mitigation	N.A.	R	R	N.A.	4 ⁽³⁸⁾ , 5
26. Auxiliary Feedwater Pump Suction Pressure	N.A.	R	R	N.A.	1, 2, 3,
27. Auxiliary Feedwater Pump Discharge Pressure	N.A.	R	R	N.A.	1, 2, 3
28. NaOH Caustic Stand Pipe Level	W	R	M	N.A.	1, 2, 3, 4
29. Hydrogen Monitors	S	Q	M	N.A.	1, 2
30. Containment Temperature Monitors	M	R	N.A.	N.A.	1, 2, 3, 4
31. Turbine Overspeed Protection Trip Channel	N.A.	R	M	N.A.	1

TABLE 4.1-1C (Page 4 of 4)

TABLE NOTATIONS

FREQUENCY NOTATION

<u>NOTATION</u>	<u>FREQUENCY</u>
S	Shift
D	Daily
W	Weekly
M	Monthly
Q	Quarterly
S/U	Prior to each startup
Y	Yearly
R	Each refueling shutdown
N.A.	Not Applicable

TABLE NOTATION

- | | |
|--|---|
| <p>(30) Prior to each startup following shutdown in excess of two days if not done in previous 30 days.</p> <p>(31) When the reactor trip system breakers are closed and the control rod drive system is capable of rod withdrawal.</p> <p>(32) Following rod motion in excess of six inches when the computer is out of service.</p> <p>(33) Transfer logic to Refueling Water Storage Tank.</p> <p>(34) When either main steam isolation valve is open.</p> <p>(35) Includes those instruments named in the emergency procedure.</p> | <p>(36) Except for containment hydrogen monitors and refueling water storage tank level which are separately specified in this table.</p> <p>(37) When RHR is in operation.</p> <p>(38) When the reactor coolant system average temperature is less than 310°F.</p> <p>(39) Whenever CONTAINMENT INTEGRITY is required.</p> |
|--|---|

3.15 EVENT MONITORING INSTRUMENTATION

Bases

The OPERABILITY of the event monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, NUREG-0737 and the Regulatory Guide 1.97 evaluation of Prairie Island instrumentation.

The Action Statements for one or more inoperable channels in some circumstances require submittal of a report to the Commission. The report shall outline preplanned alternate method of monitoring as applicable, the cause of the inoperability, and the plans and schedule for restoring the instrument channels of the Function to OPERABLE status.

Some containment penetration flow paths may have one valve with position indication based on the justification that another mechanism is provided to assure containment integrity is maintained. In the event the position indication on the one valve fails, the justification for the penetration to have a single valve with position indication is assumed to remain valid. Since another mechanism continues to provide containment integrity, the applicable action is Action Statement 1.

The following core exit thermocouples are included in the center core region referenced in Table TS.3.15-1, Action Statement 5. If a thermocouple is not listed below, it is located in the outside core region.

<u>Thermocouple Number</u>	<u>Core Location</u>
9	D-5
10	D-7
12	E-4
13	E-6
14	E-10
16	F-7
18	G-4
19	G-6
22	H-5
23	H-9
28	I-4
29	I-8
30	I-10
32	J-6
33	J-8
34	J-9

Core exit thermocouple readings necessary to meet the requirements of Specification 3.15.A are available from the Plant Process Computer, the Control Room Core Exit Thermocouple Display or if no other readout is available, from test equipment readings from the Core Exit Thermocouple Junction Boxes.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 121 AND 114 TO

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated January 10, 1995, as supplemented August 9 and September 20, 1995, the Northern States Power Company (NSP or the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. The proposed amendments would incorporate Limiting Condition for Operation (LCO) requirements from NUREG-1431, "Standard Technical Specifications (STS) for Westinghouse Plants," dated September 1992, into the plant TS Section 3.15, Event Monitoring Instrumentation. The proposed amendments would also make necessary changes in TS Table 4.1-1C, Miscellaneous Instrumentation Surveillance Requirements, and Bases 3.15, Event Monitoring Instrumentation, to reflect TS Section 3.15 changes.

Incorporation of STS LCO Section 3.3.3 into the plant TS Section 3.15 would:

- a. Revise the table of contents to reflect the merging of TS Section 3.15-1 subsections A, B, and C into one section, TS Section 3.15, and the merging of TS Tables 3.15-1, 3.15-2, and 3.15-3 into one table, TS Table 3.15-1.
- b. Revise the action statements in TS Section 3.15 associated with inoperability of instrumentation for post-accident monitoring (PAM) parameters.
- c. Replace the columns titled "Required Total No. of Channels" and "Minimum Channels Operable" with a column titled "Required Channels" in TS Table 3.15-1.
- d. Delete non-Category 1 instrumentation from TS Table 3.15-1.
- e. Revise surveillance requirements for containment area radiation and reactor vessel level instruments in TS Table 4.1-1C.
- f. Revise Bases 3.15 to correspond with the changes in TS Section 3.15.

2.0 EVALUATION

The purpose of the PAM instrumentation is to display variables that provide information required by the control room operators during and following an accident. This information provides the necessary support for the operators to take the manual actions required for safety systems to accomplish their safety functions for design-basis accidents. The operability of the PAM instrumentation ensures that sufficient information is available on selected parameters for the operator to monitor and to assess the plant status and behavior following an accident. The following evaluation addresses the operability requirements, and the associated required actions.

2.1 Required Channels and Diverse Variables

The licensee proposed a revision to TS Table 3.15-1 to the format presented in STS Table 3.3.3-1 by replacing columns titled "Required Total No. of Channels" and "Minimum Channels Operable" with a column titled "Required Channels." Table 3.3.3-1 of the STS requires two operable channels for PAM functions. Two operable channels ensure that no single failure prevents the operator from getting information necessary for determining the safety status of the plant and bringing the plant to a safe condition following an accident.

Consistent with STS Table 3.3.3-1, the licensee has proposed that two channels for each parameter be provided as specified in TS Table 3.15-1, except reactor coolant system (RCS) hot leg temperature and RCS cold leg temperature. For RCS hot leg temperature and RCS cold leg temperature the licensee has proposed one channel per loop (Prairie Island, Units 1 and 2, are two-loop plants) of instrumentation for operability along with indication of diverse variables.

The licensee has proposed that core exit temperature, RCS cold leg temperature, and steam generator pressure serve as diverse variables for RCS hot leg temperature. Since core exit temperature correlates with RCS hot leg temperature, the use of core exit temperature, RCS cold leg temperature, and steam generator pressure provide appropriate alternative indications for RCS hot leg temperature and are, therefore, acceptable.

The licensee has proposed that core exit temperature, steam generator pressure, and RCS hot leg temperature serve as diverse variables for RCS cold leg temperature. Because heat is transferred from the RCS to the steam generators, steam generator temperature (which can be derived from steam generator pressure) will be lower than RCS cold leg temperature. The difference between steam generator temperature and RCS cold leg temperature is proportional to the heat removed. RCS cold leg temperature is used in calculations to determine the RCS cooldown rate. The use of steam generator temperature in place of RCS cold leg temperature would reflect a higher cooldown rate than the actual cooldown rate and thus result in a more conservative operation. The licensee has also committed to establishing standing procedural guidance for the plant operators directing them to use steam generator pressure in lieu of RCS cold leg temperature in the event of

RCS cold leg temperature unavailability. Since the use of steam generator temperature in place of RCS cold leg temperature would be conservative, the use of steam generator pressure, core exit temperature, and RCS hot leg temperature provide appropriate alternative indications for RCS cold leg temperature and are, therefore, acceptable.

2.2 Action Statements

The licensee also proposed changes to the action statements associated with each PAM variable. The staff reviewed similarities between the proposed action statements and those of the STS and found the following deviations.

2.2.1 Orderly Transition

The proposed Action Statement 2 states that with two required channels inoperable, restore one channel to operable status within 7 days, or be in at least Mode 3 within the next 6 hours. In the STS, the required action for this condition is to be in Mode 3 within 6 hours and be in Mode 4 within 12 hours. The action statement in the existing TS Table 3.15-1, which the proposed Action Statement 2 replaces, does not have a requirement to proceed to Mode 4. The proposed action statement does not change the present commitment nor appear to adversely affect continued safe operation of the plant. Therefore, the proposed change is acceptable.

2.2.2 Diverse Channels

The proposed action statements for RCS hot leg temperature and RCS cold leg temperature deviate from the STS which requires two channels per loop. The proposed action statement provides for one channel per loop and a diverse indication in place of a second channel.

The proposed action statements for these variables state that with the required channel inoperable, and at least one diverse channel operable for the affected loop, restore the required channel to operable status within 30 days. If the required channel is not restored to operable status within 30 days, prepare and submit a special report to the Commission within 14 days outlining the actions taken (including the preplanned alternate method of monitoring), the cause of the inoperability, and the plans and schedule for restoring the inoperable channel to operable status.

The above action statements are similar to the action statements for two-channel systems where one channel is inoperable and the STS require the inoperable channel be restored to operable status within 30 days or a special report be submitted to the NRC within the next 14 days providing plans for restoring the inoperable channel to operable status. The licensee takes credit in the proposed TS for the diverse channel in place of the second channel and follows the action statement for one inoperable channel. Although the operable diverse channel is not an exact measurement of the primary

variable, the operator is still able to derive the necessary information about the primary variable.

Therefore, the action statements for these variables with the appropriate diverse channel as the second channel is consistent with the STS and is acceptable.

In the event that both the required channel and the diverse channel for the affected loop are inoperable, the TS would require either the required channel or the diverse channel be restored within 7 days or the plant be shut down, which is commensurate with the required action in the STS for a total loss of indication, and is, therefore, acceptable.

2.3 Deletion of Non-Category 1 Instrumentation

Auxiliary feedwater flow, RCS subcooling margin, pressure-operated relief valve position, pressure-operated relief block valve position, pressurizer safety valve position, containment water level (narrow range), steam relief activity monitor, and high-range shield building vent releases are Regulatory Guide (R.G.) 1.97 ("Instrumentation for Light-Water Cooled Nuclear Power Plants To Assess Plant and Environs Conditions During and Following an Accident") Category 2 variables and are not included in the STS for PAM. Therefore, the licensee has proposed that these instruments be deleted from TS Table 3.15-1. The deletion of these instruments from TS Table 3.15-1 is acceptable.

2.4 Surveillance Requirements

The licensee has proposed a revision to TS Table 4.1-1C regarding surveillance requirements for containment radiation and reactor vessel water level. The licensee proposes that surveillance for these two instruments be the same as the surveillance requirements for other PAM instrumentation. The proposed surveillance requirements for containment radiation and reactor vessel water level are consistent with the STS, and are, therefore, acceptable.

2.5 Bases 3.15

The licensee proposed that Bases 3.15 be revised to define the contents of required reports, provide clarifications on containment penetration flow path status, and include core exit thermocouple changes commensurate with the changes in TS 3.15. The proposed changes to Bases 3.15 are consistent with the STS, and are, therefore, acceptable.

2.6 Summary

Based on our review of the proposed amendments, the staff concludes that the proposed changes to the PAM instrumentation operability requirements for the Prairie Island Nuclear Generating Plant, Units 1 and 2, TS conform to the STS and their bases and the guidelines of R.G. 1.97. The staff further determined that the proposed TS changes provide appropriate limiting conditions for operation and action statements for the PAM instrumentation, and are, therefore, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (60 FR 8753). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Barry S. Marcus

Date: November 9, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 9, 1995

MEMORANDUM TO: BiWeekly Notice Coordinator

FROM: Beth A. Wetzel, Project Manager *Beth A. Wetzel*
Project Directorate III-1
Division of Reactor Projects - III/IV

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE OF
ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE
(TAC NOS. M91336 AND M91337)

Northern States Power Company, Docket No. 50-282, Prairie Island Nuclear
Generating Plant, Unit No. 1, Goodhue County, Minnesota

Date of application for amendment: January 10, 1995, as supplemented
August 9 and September 20, 1995.

Brief description of amendment: The amendments revise the Prairie Island
event monitoring instrumentation Technical Specifications and associated
Bases to conform to Standard Technical Specifications for post-accident
monitoring.

Date of issuance: November 9, 1995

Effective date: November 9, 1995, with full implementation within 30 days.

Amendment No.: 121/114

Facility Operating License No. DPR-42 and DPR-60. Amendments revised the
Technical Specifications.

Date of initial notice in FEDERAL REGISTER: February 15, 1995 (60 FR 8753)

The August 9 and September 20, 1995, letters provided updated Technical
Specification pages and clarifying information in response to discussions
with the staff during various teleconferences conducted during the review
process. This information was within the scope of the original application
and did not change the staff's initial proposed no significant hazards
consideration determination. The Commission's related evaluation of the

BiWeekly Notice Coordinator

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amendment is contained in a Safety Evaluation dated November 9, 1995.

No Significant hazards consideration comments received: No.

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Minnesota 55401.

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DATE	10/25/95		10/25/95		10/30/95	10/19/95 <i>For</i>

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