December 17, 1998

Mr. Roger O. Anderson, Director
Nuclear Energy Engineering
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: REACTOR COOLANT VENT SYSTEM (TAC NOS. MA3503 AND MA3504)

Dear Mr. Anderson:

The Commission has issued the enclosed Amendment No. 142 to Facility Operating License No. DPR-42 and Amendment No. 133 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated September 4, 1998.

The amendments revise the Technical Specifications for the Prairie Island Nuclear Generating Plant Units 1 and 2 to clarify the surveillance requirements and limiting conditions for operation of the reactor coolant vent system.

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Original Signed by

Tae Kim, Senior Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:
1. Amendment No. 142 to DPR-42
2. Amendment No. 133 to DPR-60
3. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 17, 1998

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Nuclear Energy Engineering
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414 Nicollet Mall
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Division of Reactor Projects - III/IV
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cc w/encl: See next page

Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating Plant

cc:

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Office of the Attorney General
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Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
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Mr. Stephen Bloom, Administrator
Goodhue County Courthouse
Box 408
Red Wing, Minnesota 55066-0408

Kris Sanda, Commissioner
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121 Seventh Place East
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St. Paul, Minnesota 55101-2145

Site Licensing
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, Minnesota 55089

Tribal Council
Prairie Island Indian Community
ATTN: Environmental Department
5636 Sturgeon Lake Road
Welch, Minnesota 55089

DATED: December 17, 1998

AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-42-PRAIRIE ISLAND UNIT 1
AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NO. DPR-60-PRAIRIE ISLAND UNIT 2

Docket File (50-282, 50-306)
PUBLIC
PDIII-1 Reading
E. Adensam (EGA1)
C. Jamerson
T. J. Kim (2)
OGC
G. Hill (4)
W. Beckner
M. Reinhart
T. Liu
ACRS
B. Burgess, RIII
SEDB (TLH3)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated September 4, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 142 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Tae Kim, Senior Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 17, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 142

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
TS.3.1-5	TS.3.1-5
TS.4.18-1	TS.4.18-1
B.4.18-1	B.4.18-1

3.1.A.2.c.(2).(a) One PORV may be inoperable for 7 days. If these conditions cannot be met, depressurize and vent the reactor coolant system through at least a 3 square inch vent within the next 8 hours.

(b) With both PORVs inoperable, complete depressurization and venting of the RCS through at least a 3 square inch vent within 8 hours.

(3) Reactor Coolant System average temperature below the temperature specified in the PTLR for disabling both safety injection pumps

With Reactor Coolant System temperature less than the temperature specified in the PTLR for disabling both safety injection pumps, when the head is on the reactor vessel and the reactor coolant system is not vented through a 3 square inch or larger vent; both Pressurizer power operated relief valves (PORVs) shall be OPERABLE (except as specified in 3.1.A.2.c.(3).(a) and 3.1.A.2.c.(3).(b) below) with the Over Pressure Protection System enabled, the associated block valve open, and the associated backup air supply charged.

(a) One PORV may be inoperable for 24 hours. If these conditions cannot be met, depressurize and vent the reactor coolant system through at least a 3 square inch vent within 8 hours.

(b) With both PORVs inoperable, complete depressurization and venting of the RCS through at least a 3 square inch vent within 8 hours.

3.1.A.3 Reactor Coolant Vent System

a. A reactor shall not be made or maintained critical nor shall reactor coolant system average temperature exceed 200°F unless Reactor Coolant Vent System paths from both the reactor vessel head and pressurizer steam space are OPERABLE and closed (except as specified in 3.1.A.3.b and 3.1.A.3.c below).

b. During STARTUP OPERATION and POWER OPERATION, any one of the following conditions of inoperability may exist for each unit provided STARTUP OPERATION is discontinued until OPERABILITY is restored. If any one of these conditions is not restored to an OPERABLE status within 30 days, be in at least HOT SHUTDOWN within the next 6 hours and in COLD SHUTDOWN within the following 30 hours:

(1) Either both of the parallel vent valves in the reactor vessel head vent path are inoperable, or the associated vent path segment is inoperable, or

(2) Either both of the parallel vent valves in the pressurizer vent path are inoperable, or the associated vent path segment is inoperable, or

(3) The vent valve to the pressurizer relief tank discharge line or the associated vent path segment is inoperable, or

(4) The vent valve to the containment atmospheric discharge line or the associated vent path segment is inoperable.

c. With no Reactor Coolant Vent System path OPERABLE, restore at least one vent path to OPERABLE status within 72 hours or be in at least HOT SHUTDOWN within the next 6 hours and COLD SHUTDOWN within the following 30 hours.

4.18 REACTOR COOLANT VENT SYSTEM PATHS

Applicability

Applies to the surveillance performed on the Reactor Coolant Vent System paths to verify OPERABILITY.

Objective

To assure that the capability exists to vent noncondensible gases from the Reactor Coolant System that could inhibit natural circulation core cooling.

Specification

Each Reactor Coolant System vent path shall be demonstrated OPERABLE at least once each refueling cycle interval by:

a. Verifying all manual isolation valves in each vent path are blocked and tagged in the open position,

b. Cycling each solenoid operated valve in each vent path through at least one complete cycle of full travel from the control room, and

c. Verifying flow through each Reactor Coolant System vent path.

4.18 REACTOR COOLANT VENT SYSTEM PATHS

Bases

The manual valves in the Reactor Coolant Vent System are blocked and tagged in the open position to eliminate the possibility that operation of the system could be blocked by the inadvertent closure of any of the system manual valves.

The cycling of each solenoid operated vent valve once each refueling cycle interval ensures the ability of these valves to open if required to vent the Reactor Coolant System. More frequent cycling of the valves is not practical because they cannot be isolated from the Reactor Coolant System while the plant is operating.

Reactor Coolant Vent System flow will be determined qualitatively to assure that there are no blockages in the Reactor Coolant Vent System piping. This may be done in a piecewise fashion, where the flow verification for different pipe segments may be performed by different procedures using different fluids.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated September 4, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 133 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, with full implementation within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Tae Kim, Senior Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 17, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 133

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
TS.3.1-5	TS.3.1-5
TS.4.18-1	TS.4.18-1
B.4.18-1	B.4.18-1

3.1.A.2.c.(2).(a) One PORV may be inoperable for 7 days. If these conditions cannot be met, depressurize and vent the reactor coolant system through at least a 3 square inch vent within the next 8 hours.

(b) With both PORVs inoperable, complete depressurization and venting of the RCS through at least a 3 square inch vent within 8 hours.

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3.1.A.3 Reactor Coolant Vent System

a. A reactor shall not be made or maintained critical nor shall reactor coolant system average temperature exceed 200°F unless Reactor Coolant Vent System paths from both the reactor vessel head and pressurizer steam space are OPERABLE and closed (except as specified in 3.1.A.3.b and 3.1.A.3.c below).

b. During STARTUP OPERATION and POWER OPERATION, any one of the following conditions of inoperability may exist for each unit provided STARTUP OPERATION is discontinued until OPERABILITY is restored. If any one of these conditions is not restored to an OPERABLE status within 30 days, be in at least HOT SHUTDOWN within the next 6 hours and in COLD SHUTDOWN within the following 30 hours:

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(2) Either both of the parallel vent valves in the pressurizer vent path are inoperable, or the associated vent path segment is inoperable, or

(3) The vent valve to the pressurizer relief tank discharge line or the associated vent path segment is inoperable, or

(4) The vent valve to the containment atmospheric discharge line or the associated vent path segment is inoperable.

c. With no Reactor Coolant Vent System path OPERABLE, restore at least one vent path to OPERABLE status within 72 hours or be in at least HOT SHUTDOWN within the next 6 hours and COLD SHUTDOWN within the following 30 hours.

4.18 REACTOR COOLANT VENT SYSTEM PATHS

Applicability

Applies to the surveillance performed on the Reactor Coolant Vent System paths to verify OPERABILITY.

Objective

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 142

TO FACILITY OPERATING LICENSE NO. DPR-42

AND AMENDMENT NO. 133 TO FACILITY OPERATION LICENSE NO. DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated September 4, 1998, Northern States Power Company (NSP or the licensee) proposed Technical Specifications (TS) changes to Prairie Island Nuclear Generating Plant (PINGP), Units 1 and 2, to clarify the surveillance requirements and limiting conditions for operation (LCOs) of the reactor coolant vent system. The proposed clarification also satisfies a recent corrective action commitment made by the licensee in Licensee Event Report 1-98-09.

2.0 BACKGROUND

The TS establish surveillance requirements for systems and components directly related to safety limits and LCOs. These requirements ensure that the systems and components will perform their intended plant function consistent with safe plant operation and as assumed by the safety analyses.

The current PINGP TS require that the reactor coolant vent valve testing be performed at the end of each refueling outage to demonstrate vent system operability. If the valve surveillance testing is performed at the end of the outage, and because the solenoid-operated vent valves have failed the surveillance testing and required repairs on occasion, the current TS requirement has the potential to delay a unit's return to service.

The reactor coolant vent system was installed in PINGP using the guidance of NUREG-0737, "Clarification of TMI Action Plan Requirements," Item II.B.1, to meet the design requirements of 10 CFR 50.44 (c)(3)(iii). This design was approved by NRC safety evaluation dated September 13, 1983. Generic Letter (GL) 83-37, "NUREG-0737, Technical Specifications," provided guidance on the reactor coolant vent system TS requirements. The licensee submitted a license amendment request dated January 13, 1984, in response to GL 83-37, and the NRC approved this request by Amendments 69 and 63 dated March 27, 1984.

3.0 EVALUATION

The following are the proposed TS revisions and the associated discussions:

1) TS 3.1.A.3.b, Reactor Coolant System

The licensee proposed to append the associated vent path segment for applicability for each solenoid-operated vent and block valve LCO.

The licensee stated in its submittal that "Inoperability of a reactor coolant vent path due to the loss of function in a segment of vent piping is little different from inoperability of the same vent path due to a loss of function in the vent path valve(s) along that piping segment . Vent path inoperability due to a loss of function (inoperability) of a vent pipe segment should receive the same opportunity (time limit) to restore function (operability) as the case where vent path inoperability is due to the loss of function in solenoid operated vent valves."

The failure of the current PINGP TS to provide requisite action for the condition, where the loss of function in a reactor coolant vent system piping segment makes a single vent path inoperable, forces entry into TS 3.0.C, which may place PINGP in an unnecessary shutdown transient and challenge plant equipment and its performance. Revising the LCO for solenoid-operated valves to include associated vent path segments would prevent unnecessary plant shutdown with no significant impact to safety.

2) TS 4.18, Reactor Coolant Vent Paths

The licensee proposed to regroup the TS into a single requirement with the following three items: manual isolation valves, solenoid-operated valve, and system flow through each reactor coolant system vent path. In addition, the licensee proposed to change the testing frequency from "prior to commencing STARTUP OPERATION after each refueling" to "at least once each refueling cycle interval."

The PINGP TS 4.18 requires that the reactor coolant vent system be demonstrated operable at the end of each refueling outage by the prescribed surveillance. The TS requirement is twofold. First, it performs testing on a frequency set by the length of each refueling outage. Second, it provides for configuration management of the vent system by waiting until the last available time period to verify operation of the system and its components. Hence, TS 4.18 combines activities that do not need to be performed together.

Revising the surveillance frequency for reactor coolant vent system path operability and system flow testing on a refueling cycle interval basis, without further prescribing when that testing is to take place, is consistent with the intent of the TS changes made by

Amendments 69 and 63 on March 27, 1984. Furthermore, proposed changes to the surveillance frequency for manual isolation valves, solenoid-operated valves, and system flow through each reactor coolant system vent path are consistent with the guidance provided in GL 83-37.

3) Basis 4.18, Reactor Coolant Vent System Paths

This change would modify the associated bases to reflect and to support the revised TS 4.18.

This revision supports the changes proposed for TS 4.18.

4.0 SUMMARY

The proposed changes meet the intent of the TS requirements as approved by NRC in the amendments dated March 27, 1984. The proposed TS revision for TS 3.1.A.3.b, which would modify the LCO for solenoid-operated valves to include associated vent path segments, would prevent unnecessary plant shutdown with no significant impact to safety. Revisions to TS 4.18 would serve to clarify the existing TS requirements and are consistent with the guidance provided in GL 83-37. These TS changes will not increase the surveillance intervals, and the surveillance intervals will continue to validate system/component availability and performance. Therefore, the staff finds the proposed TS changes to TS 3.1.A.3.b and TS 4.18 acceptable.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (63 *FR 50938*). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Liu

Date: December 17, 1998

December 17, 1998

Mr. Roger O. Anderson, Director
Nuclear Energy Engineering
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: REACTOR COOLANT VENT SYSTEM (TAC NOS. MA3503 AND MA3504)

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Sincerely,

Original Signed by

Tae Kim, Senior Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
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Docket Nos. 50-282 and 50-306

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cc w/encl: See next page

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