

January 26, 1998

Mr. Roger O. Anderson, Director  
Licensing and Management Issues  
Northern State Power Company  
414 Nicollet Mall  
Minneapolis, MN 55401

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION AND OPPORTUNITY FOR HEARING - PRAIRIE ISLAND  
NUCLEAR GENERATING PLANT, UNIT 1 (TAC NO. MA0650)

Dear Mr. Anderson:

Enclosed is a copy of the subject notice that relates to Northern States Power Company's application for amendment for Prairie Island Nuclear Generating Plant, Unit 1 dated January 15, 1998.

The proposed amendment would initiate a one-time only change for Prairie Island Unit 1 Cycle 19 that would allow the use of the moveable incore detector system for measurement of the core peaking factors with less than 75% and greater than or equal to 50% of the detector thimbles available

The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original Signed By

Beth A. Wetzel, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282  
and 50-306

Enclosure: Notice

cc w/encl: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, reading "Beth A. Wetzel", is positioned above the typed name.

Beth A. Wetzel, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. 50-282  
and 50-306

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cc w/encl: See next page

Mr. Roger O. Anderson, Director  
Northern States Power Company

Prairie Island Nuclear Generating  
Plant

cc:

J. E. Silberg, Esquire  
Shaw, Pittman, Potts and Trowbridge  
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Washington DC 20037

Plant Manager  
Prairie Island Nuclear Generating  
Plant  
Northern States Power Company  
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U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
1719 Wakonade Drive East  
Welch, Minnesota 55089-9642

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
801 Warrenville Road  
Lisle, Illinois 60532-4351

Mr. Jeff Cole, Auditor/Treasurer  
Goodhue County Courthouse  
Box 408  
Red Wing, Minnesota 55066-0408

Kris Sanda, Commissioner  
Department of Public Service  
121 Seventh Place East  
Suite 200  
St. Paul, Minnesota 55101-2145

Site Licensing  
Prairie Island Nuclear Generating  
Plant  
Northern States Power Company  
1717 Wakonade Drive East  
Welch, Minnesota 55089

Tribal Council  
Prairie Island Indian Community  
ATTN: Environmental Department  
5636 Sturgeon Lake Road  
Welch, Minnesota 55089

UNITED STATES NUCLEAR REGULATORY COMMISSIONNORTHERN STATES POWER COMPANYDOCKET NO. 50-282NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TOFACILITY OPERATING LICENSE NO. DPR-42PROPOSED NO SIGNIFICANT HAZARDSCONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-42 issued to Northern States Power Company (the licensee) for operation of the Prairie Island Nuclear Generating Plant, Unit 1, located in Goodhue County, Minnesota.

The proposed amendment would initiate a one-time only change for Prairie Island Unit 1 Cycle 19 that would allow the use of the moveable incore detector system for measurement of the core peaking factors with less than 75% and greater than or equal to 50% of the detector minutes available.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.12, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any

accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes do not involve an increase in the probability of an accident previously evaluated. The moveable incore detector system is used only to provide confirmatory information on the neutron flux distribution and is not required for the daily safe operation of the core. The system is not a process variable that is an initial condition in the accident analyses. The only accident that the moveable incore detector system could be involved in is the breaching of the detector thimbles which would be enveloped by the small break loss of coolant accident (LOCA) analysis. As the proposed changes do not involve any changes to the system's equipment and no equipment is operated in a new or more harmful manner, there is no increase in the probability of such an accident.

The proposed [amendment] would not involve an increase in the consequences of an accident previously evaluated. The moveable incore detector system provides a monitoring function that is not used for accident mitigation (the system is not used in the primary success path for mitigation of a design basis accident). The ability of the reactor protection system or engineered safety features system instrumentation to mitigate the consequences of an accident will not be impaired by the proposed changes. The small break LOCA analysis (and thus its consequences) continues to bound potential breaching of the system's detector thimbles.

With greater than or equal to 50% and less than 75% of the detector thimbles available, core peaking factor measurement uncertainties will be increased, which could impact the core peaking factors and as a result could affect the consequences of certain accidents. However, any changes in the core peaking factors resulting from increased measurement uncertainties will be compensated for by conservative measurement uncertainty adjustments in the Technical Specifications to ensure that pertinent core design parameters are maintained. Sufficient additional penalty is added to the power distribution measurements such that this change will not impact the consequences of any accident previously evaluated.

Therefore, based on the conclusions of the above analysis, the proposed changes will not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The proposed [amendment] would not create the possibility of a new or different kind of accident previously evaluated as [it] only affect[s] the minimum complement of equipment necessary for operability of the moveable incore detector system. There is no change in plant configuration, equipment or equipment design. No equipment is operated in a new manner. Thus the changes will not create any new or different accident causal mechanisms. The accident analysis in the Updated Safety Analysis Report remains bounding.

Therefore, based on the conclusions of the above analysis, the proposed changes will not create the possibility of a new or different kind of accident.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

The proposed changes will not involve a significant reduction in a margin of safety. The reduction in the minimum complement of equipment necessary for the operability of the moveable incore detector system could only impact the monitoring/calibration functions of the system. Reduction of the number of available moveable incore detector thimbles to the 50% level does not significantly degrade the ability of the system to measure core power distributions. With greater than or equal to 50% and less than 75% of the detector thimbles available, core peaking factor measurement uncertainties will be increased, but will be compensated for by conservative measurement uncertainty adjustments in the Technical Specifications to ensure that pertinent core design parameters are maintained. Sufficient additional penalty is added to the power distribution measurements such that this change does not impact the safety margins which currently exist. Also, the reduction of available detector thimbles has negligible impact on the quadrant power tilt and core average axial power shape measurements. Sufficient detector thimbles will be available to ensure that no quadrant will be unmonitored.

Based on these factors, the proposed changes in this license amendment will not result in a significant reduction in the plant's margin of safety, as the core will continue to be adequately monitored.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By March 2, 1998, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the

proceeding must file a written request for a hearing and a petition for leave to intervene.

Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2.

Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave



of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

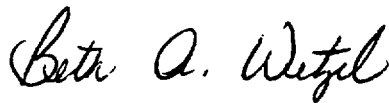
A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by close of business on the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Jay Silberg, Esq., Shaw, Pittman, Potts, and Trowbridge, 2300 N Street, NW, Washington, DC 20037, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated January 15, 1998, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401.

Dated at Rockville, Maryland, this 26th day of January 1998.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Beth A. Wetzel".

Beth A. Wetzel, Senior Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation