

March 27, 1998

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern State Power Company
414 Nicollet Mall
Minneapolis, MN 55401

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION AND OPPORTUNITY FOR HEARING - PRAIRIE ISLAND
NUCLEAR GENERATING PLANT, UNITS 1&2 (TAC NOS. MA1121 AND
MA1122)

Dear Mr. Anderson:

Enclosed is a copy of the subject notice that relates to Northern States Power Company's
application for amendment for Prairie Island Nuclear Generating Plant, Units 1 and 2 dated
March 6, 1998.

The proposed amendments would (1) update the Technical Specification heatup and cooldown
rate curves and extend their reactor fluence limit from the current 20 effective full power years
(EFPY) to a new value of 35 EFPY, (2) incorporate into Technical Specifications the use of a
Pressure and Temperature Limits Report, and (3) change the power-operated relief valves
temperature requirement for operability.

The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

ORIGINAL SIGNED BY

Beth A. Wetzel, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282
and 50-306

Enclosure: Notice
cc w/encl: See next page

DISTRIBUTION: Docket File PUBLIC
PD# 3-1 Reading E. Adensam OGC
ACRS J. McCormick-Barger

MAC FILE CONTROL COPY

DOCUMENT NAME: G:\WPDOCS\PRAIRIE\PIA1121.LTR

	PDIII-1	E	PDIII-1	E	PD111-1	
NAME	BWetzel:db <i>Baw</i>		CJamerson <i>CMH for</i>		CCarpenter <i>AK FOR</i>	
DATE	3/37 198		3/27 198		3/27 198	

OFFICIAL RECORD COPY

9804020003 980327
PDR ADOCK 05000282
P PDR

VI
Dial

Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

J. E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington DC 20037

Site Licensing
Prairie Island Nuclear Generating
Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, Minnesota 55089

Plant Manager
Prairie Island Nuclear Generating
Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, Minnesota 55089

Tribal Council
Prairie Island Indian Community
ATTN: Environmental Department
5636 Sturgeon Lake Road
Welch, Minnesota 55089

Adonis A. Neblett
Assistant Attorney General
Office of the Attorney General
455 Minnesota Street
Suite 900
St. Paul, Minnesota 55101-2127

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
1719 Wakonade Drive East
Welch, Minnesota 55089-9642

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, Illinois 60532-4351

Mr. Jeff Cole, Auditor/Treasurer
Goodhue County Courthouse
Box 408
Red Wing, Minnesota 55066-0408

Kris Sanda, Commissioner
Department of Public Service
121 Seventh Place East
Suite 200
St. Paul, Minnesota 55101-2145

UNITED STATES NUCLEAR REGULATORY COMMISSIONNORTHERN STATES POWER COMPANYDOCKET NOS. 50-282 AND 50-306NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating Licenses DPR-42 and DPR-60 issued to Northern States Power Company (the licensee) for operation of the Prairie Island Nuclear Generating Plant, Units 1 and 2, located in Goodhue County, Minnesota.

The proposed amendments would (1) update the Technical Specification heatup and cooldown rate curves and extend their reactor fluence limit from the current 20 effective full power years (EFPY) to a new value of 35 EFPY, (2) incorporate into Technical Specifications the use of a Pressure and Temperature Limits Report (PTLR), and (3) change the power-operated relief valves (PORVs) temperature requirement for operability.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any

ENCLOSURE

accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed amendment[s] will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to update the Prairie Island pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints for reactor vessel fluence to 35 EFPY is based upon measurements and calculations that were performed in accordance with an NRC approved methodology for performing reactor vessel fracture analysis to meet 10CFR50 Appendix G and H requirements. These calculations made application of American Society of Mechanical Engineers (ASME) Code Case N-514, "Low Temperature Overpressure Protection," in determining the acceptable OPPS setpoint for Prairie Island Units 1 and 2. This permits the OPPS pressure relief setpoint to be established such that the maximum pressure at the reactor vessel material's most limiting location is limited to 110% of the pressure determined to satisfy ASME Section XI, Appendix G, Article G-2215. As detailed in the exemption request to apply this ASME Code Case, the development of the Appendix G pressure/temperature limit curves incorporates numerous conservatisms. For this reason the ASME code committee approved this code case. Application of this code case with the approved methodology does not produce a significant increase in the probability or magnitude of brittle fracture of the reactor vessel.

The proposed change to relocate the pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints to a Pressure and Temperature Limits Report is an administrative change. It does not affect any system which is a contributor to initiating events for previously evaluated anticipated operational occurrences and therefore does not involve any significant increase in the probability or consequence of an accident previously evaluated.

The proposed change in PORV operability temperature from 310°F to a new value of 350°F does not affect any system which is a contributor to initiating events for previously evaluated anticipated operational occurrences and therefore does not involve any significant increase in the probability or consequence of an accident previously evaluated.

2. The proposed amendment[s] will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The proposed change to update the Prairie Island pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints for a

reactor fluence limit of 35 EFPY does not introduce a new mode of operation or testing, or make physical changes to the plant. (The new Technical Specification requirement to isolate the accumulators whenever the RCS [reactor coolant system] temperature is less than the OPPS enable temperature does not introduce a new mode of operation since Unit Shutdown procedures close the accumulator discharge valves and tag out their breakers when RCS pressure falls below 1000 psig.) The general methods employed to develop this change are well understood and have been previously reviewed and approved. Updating the operating restrictions, OPPS setpoints, and reactor fluence limit for operation do not create a possibility of a new or different kind of accident from those previously analyzed.

The proposed change to relocate the pressure and temperature limits curves and the Over Pressure Protection System (OPPS) setpoints to a Pressure and Temperature Limits Report is an administrative change. The proposed change does not alter the design, function, or operation of any plant component, therefore a possibility of a new or different kind of accident from those previously analyzed has not [been] created.

The proposed change in the PORV operability temperature from 310°F to a new value of 350°F does not involve a physical alteration of the plant. Since no new or different type of equipment will be installed, this change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed amendment[s] will not involve a significant reduction in the margin of safety.

Although neutron irradiation reduces the material fracture toughness of the reactor vessel, deterministic analyses have demonstrated that proposed P/T [pressure/temperature] curves, OPPS setpoints and reactor vessel fluence limits for operation will preserve the required margin of safety in the RCS boundary during postulated low temperature pressurization events.

The proposed change to use the PTLR is administrative in nature and does not impact the operation of the Prairie Island Nuclear Generating Plant in a manner that would result in any significant reduction in any margin of safety.

The proposed change in the PORV operability temperature from 310°F to a new value of 350°F does not impact any systems that are relied upon for core cooling or RCS pressure relief at RCS temperatures below 350°F. Setting the PORV operability temperature back to 350° aligns the PORVs with the Pressurizer Safety Valve operability requirement so the PORVs are still available to limit challenges to the Pressurizer Safety Valve settings during conditions of higher RCS pressure and energy when pressure surges become more significant. (In Amendment[s] 91/84 this temperature was changed for operational flexibility from its previous value of 350°F to a new value of 310°F to be coincident with

the OPSS Enable Temperature. This change was not done to establish a larger margin of safety.) For RCS temperatures below 350°F both the pressure and core energy are sufficiently decreased that pressure surges become less significant. For RCS temperatures below 350°F the RHR [residual heat removal] system is capable of removing the reactor decay heat and thereby controlling RCS pressure and temperature. In the unlikely event that a significant pressure surge were to occur in this temperature range with neither RHR nor the PORVs in service, one pressurizer safety valve would be operable to mitigate potential overpressure transients. Thus this change does not involve a significant reduction in the margin of safety associated with either the RCS boundary or fuel cladding.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received by close of business within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory

Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By April 27, 1998, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons

why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Jay Silberg, Esq., Shaw, Pittman, Potts, and Trowbridge, 2300 N Street, NW, Washington, DC 20037, attorney for the licensee.

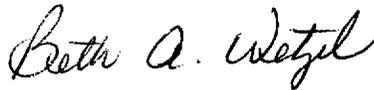
Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the

petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated March 6, 1998, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401.

Dated at Rockville, Maryland, this 24th day of March 1998.

FOR THE NUCLEAR REGULATORY COMMISSION



Beth A. Wetzel, Senior Project Manager
 Project Directorate III-1
 Division of Reactor Projects -III/IV
 Office of Nuclear Reactor Regulation

DISTRIBUTION:

Docket File	PUBLIC
PD#3-1 Reading	EAdensam (EGA1)
OGC	ACRS
JMcCormick-Barger, RIII	

DOCUMENT NAME: G:\WPDOCS\PRAIRIE\PIA1121.IND

To receive a copy of this document, indicate in the box C=Copy w/o attachment/enclosure E=Copy with attachment/enclosure N = No copy

OFFICE	PM:PD31	E	LA:PD31	E	(A)D:PD31	
NAME	BWetzel:db <i>BW</i>		CJamerson <i>CJ</i>		CACarpenter <i>CR</i>	
DATE	3 124 198		3 124 198		3 124 198	

OFFICIAL RECORD COPY