

August 29, 2002

Mr. Fred J. Cayia  
Acting Site Vice President  
Point Beach Nuclear Plant  
Nuclear Management Company, LLC  
6610 Nuclear Road  
Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF  
AMENDMENTS RE: TECHNICAL SPECIFICATION 3.8.1, "AC SOURCES -  
OPERATING" (TAC NOS. MB5198 AND MB5199)

Dear Mr. Cayia:

The Commission has issued the enclosed Amendment No. 204 to Facility Operating License No. DPR-24 and Amendment No. 209 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated May 29, 2002, as supplemented July 12, 2002.

These amendments revise TS 3.8.1, "AC Sources - Operating," to allow portions of Surveillance Requirement 3.8.1 to be performed with the units in Mode 1, 2, 3, or 4. The proposed amendments are consistent with changes made to NUREG-1431, Standard Technical Specifications, Westinghouse Plants, by Technical Specification Task Force (TSTF) Traveler, TSTF-283, Revision 3.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

*/RA/*

Deirdre W. Spaulding, Project Manager, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures: 1. Amendment No. 204 to DPR-24  
2. Amendment No. 209 to DPR-27  
3. Safety Evaluation

cc w/encls: See next page

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Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: TECHNICAL SPECIFICATION 3.8.1, "AC SOURCES - OPERATING" (TAC NOS. MB5198 AND MB5199)

Dear Mr. Cayia:

The Commission has issued the enclosed Amendment No. 204 to Facility Operating License No. DPR-24 and Amendment No. 209 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated May 29, 2002, as supplemented July 12, 2002.

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cc w/encls: See next page

DISTRIBUTION

PUBLIC OGC SSaba  
PDIII-1 Reading ACRS CHolden  
LRaghavan WBeckner  
DSpaulding GHill(4)  
RBouling RLanksbury, RGN-III

\*Provided SE input by memo  
\*\*Previously Concurred

OFFICE	PDIII-1/PM	PDIII-1/LA	EEIB/SC*	OGC**	RORP/SC**	PDIII-1/SC
NAME	DSpaulding	RBouling	CHolden	AHodgdon	RDennig	DHood for LRaghavan
DATE	09/29/02	08/29/02	07/26/02	08/29/02	08/28/02	08/29/02

ADAMS Accession No. ML022250584

OFFICIAL RECORD COPY

Point Beach Nuclear Plant, Units 1 and 2

cc:

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March 2002

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 204  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated May 29, 2002, as supplemented July 12, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 204 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA by Darl Hood for/*

L. Raghavan, Chief, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: August 29, 2002

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 209  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Nuclear Management Company, LLC (the licensee), dated May 29, 2002, as supplemented July 12, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 209 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 45 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA by Darl Hood for/*

L. Raghavan, Chief, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: August 29, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 204  
TO FACILITY OPERATING LICENSE NO. DPR-24  
AND LICENSE AMENDMENT NO. 209  
TO FACILITY OPERATING LICENSE NO. DPR-27  
DOCKET NOS. 50-266 AND 50-301

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3.8.1-7  
B 3.8.1-21  
B 3.8.1-22

INSERT

3.8.1-7  
B 3.8.1-21  
B 3.8.1-22

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 204 TO FACILITY OPERATING LICENSE NO. DPR-24  
AND AMENDMENT NO. 209 TO FACILITY OPERATING LICENSE NO. DPR-27  
NUCLEAR MANAGEMENT COMPANY, LLC  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-266 AND 50-301

## 1.0 INTRODUCTION

By application dated May 29, 2002, as supplemented July 12, 2002, the Nuclear Management Company, LLC (the licensee), requested changes to the Technical Specifications (TSs) for the Point Beach Nuclear Plant, Units 1 and 2. The proposed changes would revise TS 3.8.1, "AC Sources - Operating," to allow portions of Surveillance Requirement (SR) 3.8.1 to be performed with the units in Mode 1, 2, 3, or 4 for the purpose of reestablishing operability (e.g., post work testing following corrective maintenance, corrective modification, deficient or incomplete surveillance testing, and other unanticipated operability concerns). The licensee has also proposed changes to the corresponding TS Bases. The proposed amendments are consistent with changes made to NUREG-1431, Standard Technical Specifications, Westinghouse Plants, by Technical Specification Task Force (TSTF) Traveler, TSTF-283, Revision 3. An assessment is required to be performed to determine that the safety of the plant is maintained or enhanced if conducting portions of this SR in Mode 1, 2, 3, or 4.

The July 12, 2002, supplemental letter provided additional clarifying information that was within the scope of the original application and did not change the Nuclear Regulatory Commission (NRC) staff's initial proposed no significant hazards consideration determination.

## 2.0 REGULATORY EVALUATION

General Design Criterion (GDC)-17, "Electric Power System," of Appendix A, "General Design Criteria for Nuclear Power Plants," to 10 CFR Part 50 requires, in part, that nuclear power plants have an onsite and offsite electric power system to permit the functioning of structures, systems, and components important to safety. The onsite system is required to have sufficient independence, redundancy, and testability to perform its safety function, assuming a single failure; and the offsite system is required to be supplied by two independent circuits. In addition, GDC-17 requires provisions to minimize the probability of losing electric power from the remaining electric power supplies as the result of loss of power from the unit, the offsite transmission network, or the onsite power supplies. GDC-18, "Inspection and Testing of Electric Power System," requires that electric power systems important to safety be designed to permit appropriate periodic inspection and testing.

As described by the licensee in its May 29, 2002, application, the offsite power is supplied to the switchyard from the offsite transmission network by four transmission lines. From the switchyard, two separate circuits provide AC power through high voltage (345/13.8 kV) station auxiliary transformers to the 13.8 kV distribution network. The high voltage station auxiliary transformers are the normal supplies to the 13.8 kV distribution network. The high voltage station auxiliary transformer normally supplies offsite power to the low voltage station auxiliary transformer for each unit. Alternate power supplies to the station auxiliary transformer are the gas turbine generator and the opposite unit's high voltage station transformer.

The 13.8 kV distribution network supplies power to the low voltage station auxiliary transformers, which in turn supply power to 4.16 kV distribution buses. The 4.16 kV distribution buses supply power to safeguards buses.

The onsite standby emergency power system is comprised of four emergency diesel generators (EDGs) that directly supply the 4.16 kV safeguards electrical distribution buses. Normally, all four EDGs are operable and aligned to their normal bus; however, the EDGs can be aligned such that only one EDG per safeguards train is required to be operable to support one or both units. In addition, either EDG may be manually connected to one or both of its respective trains' 4.16 kV safeguards distribution buses. The TSs only require two of the four EDGs onsite to be operable for full power operation of both units.

Each EDG will automatically start on an undervoltage signal from its associated 4.16 kV train in either unit, and will restore power on the bus(es) to which it is aligned. All four EDGs will automatically start on a safety injection signal from either unit.

The offsite and onsite standby ac power systems at Point Beach are designed to comply with the requirements of GDC-17 and GDC-18. The proposed amendments are requested in accordance with 10 CFR 2.101, 10 CFR 50.59, and 10 CFR 50.90 to assure continued compliance with GDC-17 and GDC-18.

### 3.0 TECHNICAL EVALUATION

SR 3.8.1.5 currently requires demonstration of the operation of the onsite standby emergency power system during a loss of the offsite power actuation test signal in conjunction with an engineered safety feature actuation signal. This test verifies all actions encountered from the loss of offsite power, including shedding of the nonessential loads and energization of the emergency buses and respective permanent and auto-connected loads from the standby emergency power source. It further demonstrates the capability of the standby emergency power source to automatically achieve the required voltage and frequency within analysis limits. It contains a note which restricts performance of this SR during Mode 1, 2, 3 or 4. The reason for the note is that, during operation with the reactor critical, performance of this SR could cause perturbations to the electrical distribution system that could challenge continued steady state operation and, as a result, unit safety systems. The licensee has proposed to modify this note to allow the SR to be performed for the purpose of reestablishing OPERABILITY (e.g., post work testing following corrective maintenance, corrective modification, deficient or incomplete surveillance testing, and other unanticipated OPERABILITY concerns) provided an assessment determines plant safety is maintained or enhanced. The proposed note would read as follows:

SR 3.8.1.5 -----NOTE-----  
This surveillance shall not normally be performed with the associated unit in MODE 1, 2, 3, or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced.

(The proposed changes are underlined)

The licensee states that the proposed modification to the note will enhance the ability of Point Beach to take advantage of its redundant EDGs for completing SR 3.8.1.5 without the risk of an unnecessary plant shutdown. The revised note would also allow completion of portions of the SR outside of refueling outages should unexpected conditions preclude performing those portions of the SR during refueling outages. In addition, the licensee states that this SR may only be performed in these Modes if an assessment determines that the safety of the plant is maintained or enhanced. Thus, only those portions of the SR that could be evaluated as being able to be safely performed under the plant operating conditions would be performed. The licensee will use risk insights or deterministic method for assessing this plant safety when performing such surveillance.

Based on the above considerations, the NRC staff concludes that the licensee's proposed changes to allow portions of SR 3.8.1.5 to be performed in Mode 1, 2, 3, or 4 for the purpose of reestablishing operability of the EDG are acceptable. The proposed changes are also consistent with changes made to NUREG-1431, Standard Technical Specifications, Westinghouse Plants, by TSTF-283, Revision 3. The NRC staff concludes that the proposed TS changes do not affect the current licensing requirements and the Point Beach design continues to meet the requirements of GDC-17. Also, the NRC staff does not object to the licensee's changes to the TS Bases.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (67 FR 45571). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. N. Saba

Date: August 29, 2002