

May 9, 1991

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OGC	

Docket Nos. 50-282
and 50-306

Mr. T. M. Parker, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

Dear Mr. Parker:

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2 -
AMENDMENT NOS. 96 AND 89 TO FACILITY OPERATING LICENSE NOS. DPR-42
AND DPR-60 (TAC NOS. 79690 AND 79691)

The Commission has issued the enclosed Amendment No. 96 to Facility Operating License No. DPR-42 and Amendment No. 89 to the Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 30, 1991.

The amendments revise the TS Section 6.2.B.4 requirements for Operations Committee review of maintenance procedures. The requirements for preparation, review, and approval of maintenance procedures will be included in a new Section 6.2.C. which will be referenced by Section 6.2.B.4.

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Armando Masciantonio, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 96 to License No. DPR-42
2. Amendment No. 89 to License No. DPR-60
3. Safety Evaluation

cc w/enclosures:
See next page

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PDR ADOCK 05000282
P PDR

LA/PD31:DRP345
PShuttleworth
4/23/91

PM/PD31:DRP345
AMasciantonio
4/23/91 *com*

D/PD31:DRP345
LMarsh
5/9/91

LPEB
WBrach
4/24/91

OGC
4/30/91

DF01

*OGC
4/30/91
By C. Conner
6-3-91 OCT 10 11:11 AM
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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

May 9, 1991

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and 50-306

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The amendments revise the TS Section 6.2.B.4 requirements for Operations Committee review of maintenance procedures. The requirements for preparation, review, and approval of maintenance procedures will be included in a new Section 6.2.C. which will be referenced by Section 6.2.B.4.

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "A.S. Masciantonio".

Armando Masciantonio, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 96 to
License No. DPR-42
2. Amendment No. 89 to
License No. DPR-60
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. T. M. Parker
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

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U.S. Nuclear Regulatory Commission
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Mr. William Miller, Auditor
Goodhue County Courthouse
Red Wing, Minnesota 55066



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 30, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

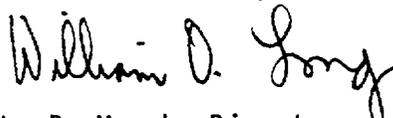
9105230154 910509
PDR ADDCK 05000282
P PDR

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 96, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for 

L. B. Marsh, Director
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 9, 1991



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 89
License No. DPR-60

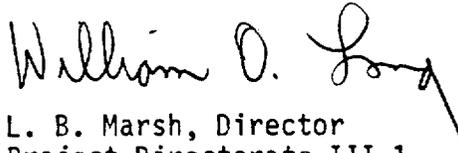
1. The Nuclear Regulatory Commission (the Commission) has found that:
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 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.89, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



for

L. B. Marsh, Director
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 9, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 96

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

TS-viii
TS.6.2-6
TS.6.2-7

INSERT

TS-viii
TS.6.2-6
TS.6.2-7

TABLE OF CONTENTS (Continued)

<u>TS SECTION</u>	<u>TITLE</u>	<u>PAGE</u>
6.0	ADMINISTRATIVE CONTROLS	TS.6.1-1
6.1	Organization	TS.6.1-1
6.2	Review and Audit	TS.6.2-1
	A. Safety Audit Committee (SAC)	TS.6.2-1
	1. Membership	TS.6.2-1
	2. Qualifications	TS.6.2-1
	3. Meeting Frequency	TS.6.2-2
	4. Quorum	TS.6.2-2
	5. Responsibilities	TS.6.2-2
	6. Audit	TS.6.2-3
	7. Authority	TS.6.2-4
	8. Records	TS.6.2-4
	9. Procedures	TS.6.2-4
	B. Operations Committee (OC)	TS.6.2-5
	1. Membership	TS.6.2-5
	2. Meeting Frequency	TS.6.2-5
	3. Quorum	TS.6.2-5
	4. Responsibilities	TS.6.2-5
	5. Authority	TS.6.2-6
	6. Records	TS.6.2-6
	7. Procedures	TS.6.2-6
	C. Maintenance Procedures	TS.6.2-7
6.3	Special Inspections and Audits	TS.6.3-1
6.4	Safety Limit Violation	TS.6.4-1
6.5	Plant Operating Procedures	TS.6.5-1
	A. Plant Operations	TS.6.5-1
	B. Radiological	TS.6.5-1
	C. Maintenance and Test	TS.6.5-3
	D. Process Control Program (PCP)	TS.6.5-3
	E. Offsite Dose Calculation Manual (ODCM)	TS.6.5-4
	F. Security	TS.6.5-4
	G. Temporary Changes to Procedures	TS.6.5-4
6.6	Plant Operating Records	TS.6.6-1
	A. Records Retained for Five Years	TS.6.6-1
	B. Records Retained for the Life of the Plant	TS.6.6-1

Prairie Island Unit 1 - Amendment No. ~~48~~, ~~73~~, ~~80~~, ~~91~~, 96
Prairie Island Unit 2 - Amendment No. ~~40~~, ~~66~~, ~~73~~, ~~84~~, 89

- f. Investigations of all Reportable Events and events requiring Special Reports to the Commission.
- g. Drills on emergency procedures (including plant evacuation) and adequacy of communication with offsite support groups.
- h. All procedures required by these Technical Specifications, including implementing procedures of the Emergency Plan, and the Security Plan (except as exempted in Section 6.5.F), shall be reviewed initially and periodically with a frequency commensurate with their safety significance but at an interval of not more than two years. Maintenance work requests and their associated procedures shall be reviewed per the requirements of Section 6.2.C.
- i. Special reviews and investigations, as requested by the Safety Audit Committee.
- j. Review of investigative reports of unplanned releases of radioactive material to the environs.
- k. All changes to the Process Control Program (PCP) and the Offsite Dose Calculation Manual (ODCM).
- l. The review of safety evaluations, when safety evaluations are required by 10 CFR Part 50, Section 50.59, for procedures or procedure changes to verify that such actions do not constitute an unreviewed safety question.

5. Authority

The OC shall be advisory to the Plant Manager. In the event of a disagreement between the recommendations of the OC and the Plant Manager, the course determined by the Plant Manager to be the more conservative will be followed. A written summary of the disagreement will be sent to the General Manager Nuclear Plants and the Chairman of the SAC for review.

6. Records

Minutes shall be recorded for all meetings of the OC and shall identify all documentary material reviewed. The minutes shall be distributed to each member of the OC, the Chairman and each member of the Safety Audit Committee, the General Manager Nuclear Plants and others designated by the OC Chairman or Vice Chairman.

7. Procedures

A written charter for the OC shall be prepared that contains:

- a. Responsibility and authority of the group

- b. Content and method of submission of presentations to the Operations Committee
- c. Mechanism for scheduling meetings
- d. Provision for meeting agenda

C. Maintenance Procedures

1. Maintenance work requests and their associated procedures required by specification 6.5.C and changes thereto, shall be prepared, reviewed and approved. Each procedure or procedure change shall be reviewed by a qualified individual(s) other than the individual(s) who prepared the procedure or procedure change. The appropriate superintendent (as previously designated by the Plant Manager) shall approve these procedures.
2. Individuals responsible for reviews performed in accordance with the requirements of specification 6.2.C.1 above shall meet or exceed the qualifications described in section 4.2 or 4.4 of ANSI N18.1-1971 and shall be previously designated by the Plant Manager. Such reviews shall include a determination of whether or not additional cross disciplinary review is necessary.
3. All reviews shall include a determination of the need for a safety evaluation to establish whether or not an unreviewed safety question is involved.

ATTACHMENT TO LICENSE AMENDMENT NO. 89

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

TS-viii
TS.6.2-6
TS.6.2-7

INSERT

TS-viii
TS.6.2-6
TS.6.2-7

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	5. Authority	TS.6.2-6
	6. Records	TS.6.2-6
	7. Procedures	TS.6.2-6
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	A. Plant Operations	TS.6.5-1
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Prairie Island Unit 1 - Amendment No. 48, 73, 80, 81, 96
Prairie Island Unit 2 - Amendment No. 40, 66, 73, 84, 89

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3. All reviews shall include a determination of the need for a safety evaluation to establish whether or not an unreviewed safety question is involved.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 96 AND 89 TO

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated January 30, 1991, Northern States Power Company (NSP or the licensee) requested amendments to the Technical Specifications (TS) appended to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. The proposed amendments would revise the TS Section 6.2.B.4 requirements for Operations Committee review of maintenance procedures. The requirements for preparation, review, and approval of maintenance procedures will be included in a new Section 6.2.C. which will be referenced by TS Section 6.2.B.4

2.0 EVALUATION

The Prairie Island TS currently require review of maintenance procedures by the Operations Committee and also allow the use of Operations Committee subcommittees. The time demand for the multiple responsibilities of Operations Committee members led to the use of an Operations Committee subcommittee for the review of maintenance procedures at Prairie Island. The Prairie Island use of subcommittee review of maintenance procedures could be interpreted as being outside the requirements of the Prairie Island TS as currently written. The proposed changes are intended to meet both the need for appropriate review of maintenance procedures and the need to restrict the time demands on the Operations Committee, and to eliminate the divergent interpretations of the existing specification.

One component of the change is the addition of the requirement for the Operations Committee to review safety evaluations for procedures or procedure changes to verify that such actions do not constitute an unreviewed safety question. Previously, there has been no TS requirement for the Operations Committee to review safety evaluations related to procedures or procedure changes. However, all safety evaluations have been reviewed by the Operations Committee and the requirement for such reviews has been in the Prairie Island administrative controls associated with safety evaluations. Safety evaluations for procedures or procedure changes will be prepared as required by 10 CFR Part 50, Section 50.59.

The proposed change provides a review process for maintenance work requests and their associated procedures by a qualified individual and approval by the appropriate superintendent. Appropriate controls are included to assure proper qualifications of the reviewing individual and a determination of whether or not an unreviewed safety question is involved.

The staff has reviewed the proposed changes and finds that the process provides reasonable assurance that an adequate review of safety-related maintenance procedures will be available and the changes will not cause a decline in the effectiveness of the reviews of safety-related maintenance procedures.

3.0 STATE CONSULTATION

In accordance with the requirements of 10 CFR 50.91(b), a state consultation was attempted. We were informed, however, that our contact no longer has any official interest in the activities of Minnesota's nuclear power plants.

By memo dated January 4, 1991 from L. B. Marsh to C. Kammerer, the NRC Office of Governmental and Public Affairs was requested to identify an appropriate Minnesota contact so that state consultation may resume. Accordingly, there was no state consultation as to this amendment.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments involves a change in recordkeeping, reporting or administrative procedures or requirements. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. S. Masciantonio, NRR

Dated: May 9, 1991