

August 20, 1992

Docket Nos. 50-282
and 50-306

Mr. T. M. Parker, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

Dear Mr. Parker:

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2 -
AMENDMENT NOS. 101 AND 94 TO FACILITY OPERATING LICENSE NOS. DPR-42
AND DPR-60 (TAC NOS. M83793 AND M83794)

The Commission has issued the enclosed Amendment No. 101 to Facility Operating License No. DPR-42 and Amendment No. 94 to the Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated June 15, 1992. The application requests changes that would implement Generic Letter 87-09 provisions relating to missed surveillances and applicability of action requirements.

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Marsha Gamberoni, Acting Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 101 to DPR-42
2. Amendment No. 94 to DPR-60
3. Safety Evaluation

cc w/enclosures:
See next page

OFFICE	LA:PD31	APM:PD31	OGC	D:PD31	
NAME	MShuttleworth	MGamberoni:dy	LM	LM	
DATE	8/11/92	9/17/92	03/11/92	8/19/92	1/1

OFFICIAL RECORD COPY

FILENAME: 83793.amd

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Mr. T. M. Parker
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

Gerald Charnoff, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington DC 20037

Mr. E. L. Watzl, Site General Manger
Prairie Island Nuclear Generating
Plant
Northern States Power Company
Route 2
Welch, Minnesota 55089

Lisa R. Tiegel
Assistant Attorney General
Environmental Protection Division
Suite 200
520 Lafayette Road
St. Paul, Minnesota 55155

U.S. Nuclear Regulatory Commission
Resident Inspector Office
1719 Wakonade Drive
East Welch, Minnesota 55089

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Mr. William Miller, Auditor
Goodhue County Courthouse
Red Wing, Minnesota 55066

Kris Sanda, Commissioner
Director of Public Service
790 American Center Building
150 East Kellogg Boulevard
St. Paul, Minnesota 55101

DATED: August 20, 1992

AMENDMENT NO.101 TO FACILITY OPERATING LICENSE NO. DPR-42-PRAIRIE ISLAND UNIT 1
AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NO. DPR-60-PRAIRIE ISLAND UNIT 2

Docket File
NRC & Local PDRs
PDIII-1 Reading
PI Plant File
B. Boger, 13/E/4
J. Zwolinski, 13/H/24
L. Marsh
M. Shuttleworth
M. Gamberoni
OGC-WF
D. Hagan, 3206 MNBB
G. Hill (8), P-137
Wanda Jones, MNBB-3701
C. Grimes, 11/F/23
W. Long
ACRS (10)
GPA/PA
OC/LFMB
W. Shafer, R-III

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated June 15, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

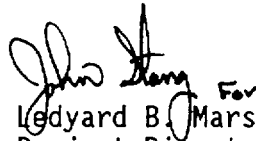
9209040122 920820
PDR ADDCK 05000282
P PDR

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.101, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 101

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

TS-v

TS.4.1-1

INSERT

TS-v
TS.4.0-1
TS.4.1-1

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<u>TS SECTION</u>	<u>TITLE</u>	<u>PAGE</u>
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4.1	Operational Safety Review	TS.4.1-1
4.2	Inservice Inspection and Testing of Pumps and Valves Requirements	TS.4.2-1
	A. Inspection Requirements	TS.4.2-1
	B. Corrective Measures	TS.4.2-2
	C. Records	TS.4.2-3
4.3	Primary Coolant System Pressure Isolation Valves	TS.4.3-1
4.4	Containment System Tests	TS.4.4-1
	A. Containment Leakage Tests	TS.4.4-1
	B. Emergency Charcoal Filter Systems	TS.4.4-3
	C. Containment Vacuum Breakers	TS.4.4-4
	D. Residual Heat Removal System	TS.4.4-4
	E. Containment Isolation Valves	TS.4.4-5
	F. Post Accident Containment Ventilation System	TS.4.4-5
	G. Containment and Shield Building Air Temperature	TS.4.4-5
	H. Containment Shell Temperature	TS.4.4-5
	I. Electric Hydrogen Recombiners	TS.4.4-5
4.5	Engineered Safety Features	TS.4.5-1
	A. System Tests	TS.4.5-1
	1. Safety Injection System	TS.4.5-1
	2. Containment Spray System	TS.4.5-1
	3. Containment Fan Coolers	TS.4.5-2
	4. Component Cooling Water System	TS.4.5-2
	5. Cooling Water System	TS.4.5-2
	B. Component Tests	TS.4.5-2
	1. Pumps	TS.4.5-2
	2. Containment Fan Motors	TS.4.5-3
	3. Valves	TS.4.5-3
4.6	Periodic Testing of Emergency Power System	TS.4.6-1
	A. Diesel Generators	TS.4.6-1
	B. Station Batteries	TS.4.6-3
	C. Pressurizer Heater Emergency Power Supply	TS.4.6-3
4.7	Main Steam Isolation Valves	TS.4.7-1
4.8	Steam and Power Conversion Systems	TS.4.8-1
	A. Auxiliary Feedwater System	TS.4.8-1
	B. Steam Generator Power Operated Relief Valves	TS.4.8-2
	C. Steam Exclusion System	TS.4.8-2
4.9	Reactivity Anomalies	TS.4.9-1
4.10	Radiation Environmental Monitoring Program	TS.4.10-1
	A. Sample Collection and Analysis	TS.4.10-1
	B. Land Use Census	TS.4.10-2
	C. Interlaboratory Comparison Program	TS.4.10-2
4.11	Radioactive Source Leakage Test	TS.4.11-1

Prairie Island Unit 1 - Amendment No. 88,78,81,101

Prairie Island Unit 2 - Amendment No. 88,88,89,94

4. SURVEILLANCE REQUIREMENTS

4.0 Applicability

- A. Each Surveillance Requirement shall be performed within the specified time interval with the following exceptions:
1. Specific time intervals between tests may be adjusted plus or minus 25% to accommodate normal test schedules.
 2. The intervals between tests scheduled for refueling shutdowns shall not exceed two years.
- B. Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.A, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.

4.1 OPERATIONAL SAFETY REVIEW

Applicability

Applies to items directly related to safety limits and limiting conditions for operation.

Objective

To specify the minimum frequency and type of surveillance to be applied to plant equipment and conditions.

Specification

- A. Calibration, testing, and checking of instrumentation channels and testing of logic channels shall be performed as specified in Table TS.4.1-1.
- B. Equipment tests shall be conducted as specified in Table TS.4.1-2A.
- C. Sampling tests shall be conducted as specified in Table TS.4.1-2B.
- D. Whenever the plant condition is such that a system or component is not required to be operable the surveillance testing associated with that system or component may be discontinued. The asterisked items in Tables 4.1-1, 4.1-2A, and 4.1-2B are required at all times, however. Discontinued surveillance tests shall be resumed less than one test interval before establishing plant conditions requiring OPERABILITY of the associated system or component, unless such testing is not practicable (i.e., nuclear power range calibration cannot be done prior to reaching power operation) in which case the testing will be resumed within 48 hours of attaining the plant condition which permits testing to be accomplished.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 94
License No. DPR-60

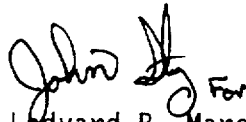
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated June 15, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 94 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate III-1
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 94

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

TS-v

TS.4.1-1

INSERT

TS-v
TS.4.0-1
TS.4.1-1

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4.2	Inservice Inspection and Testing of Pumps and Valves Requirements	TS.4.2-1
	A. Inspection Requirements	TS.4.2-1
	B. Corrective Measures	TS.4.2-2
	C. Records	TS.4.2-3
4.3	Primary Coolant System Pressure Isolation Valves	TS.4.3-1
4.4	Containment System Tests	TS.4.4-1
	A. Containment Leakage Tests	TS.4.4-1
	B. Emergency Charcoal Filter Systems	TS.4.4-3
	C. Containment Vacuum Breakers	TS.4.4-4
	D. Residual Heat Removal System	TS.4.4-4
	E. Containment Isolation Valves	TS.4.4-5
	F. Post Accident Containment Ventilation System	TS.4.4-5
	G. Containment and Shield Building Air Temperature	TS.4.4-5
	H. Containment Shell Temperature	TS.4.4-5
	I. Electric Hydrogen Recombiners	TS.4.4-5
4.5	Engineered Safety Features	TS.4.5-1
	A. System Tests	TS.4.5-1
	1. Safety Injection System	TS.4.5-1
	2. Containment Spray System	TS.4.5-1
	3. Containment Fan Coolers	TS.4.5-2
	4. Component Cooling Water System	TS.4.5-2
	5. Cooling Water System	TS.4.5-2
	B. Component Tests	TS.4.5-2
	1. Pumps	TS.4.5-2
	2. Containment Fan Motors	TS.4.5-3
	3. Valves	TS.4.5-3
4.6	Periodic Testing of Emergency Power System	TS.4.6-1
	A. Diesel Generators	TS.4.6-1
	B. Station Batteries	TS.4.6-3
	C. Pressurizer Heater Emergency Power Supply	TS.4.6-3
4.7	Main Steam Isolation Valves	TS.4.7-1
4.8	Steam and Power Conversion Systems	TS.4.8-1
	A. Auxiliary Feedwater System	TS.4.8-1
	B. Steam Generator Power Operated Relief Valves	TS.4.8-2
	C. Steam Exclusion System	TS.4.8-2
4.9	Reactivity Anomalies	TS.4.9-1
4.10	Radiation Environmental Monitoring Program	TS.4.10-1
	A. Sample Collection and Analysis	TS.4.10-1
	B. Land Use Census	TS.4.10-2
	C. Interlaboratory Comparison Program	TS.4.10-2
4.11	Radioactive Source Leakage Test	TS.4.11-1

Prairie Island Unit 1 - Amendment No. 69,73,81,101

Prairie Island Unit 2 - Amendment No. 63,66,84,94

4. SURVEILLANCE REQUIREMENTS

4.0 Applicability

- A. Each Surveillance Requirement shall be performed within the specified time interval with the following exceptions:
1. Specific time intervals between tests may be adjusted plus or minus 25% to accommodate normal test schedules.
 2. The intervals between tests scheduled for refueling shutdowns shall not exceed two years.
- B. Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.A, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation. The time limits of the ACTION requirements are applicable at the time it is identified that a Surveillance Requirement has not been performed. The ACTION requirements may be delayed for up to 24 hours to permit the completion of the surveillance when the allowable outage time limits of the ACTION requirements are less than 24 hours. Surveillance Requirements do not have to be performed on inoperable equipment.

4.1 OPERATIONAL SAFETY REVIEW

Applicability

Applies to items directly related to safety limits and limiting conditions for operation.

Objective

To specify the minimum frequency and type of surveillance to be applied to plant equipment and conditions.

Specification

- A. Calibration, testing, and checking of instrumentation channels and testing of logic channels shall be performed as specified in Table TS.4.1-1.
- B. Equipment tests shall be conducted as specified in Table TS.4.1-2A.
- C. Sampling tests shall be conducted as specified in Table TS.4.1-2B.
- D. Whenever the plant condition is such that a system or component is not required to be operable the surveillance testing associated with that system or component may be discontinued. The asterisked items in Tables 4.1-1, 4.1-2A, and 4.1-2B are required at all times, however. Discontinued surveillance tests shall be resumed less than one test interval before establishing plant conditions requiring OPERABILITY of the associated system or component, unless such testing is not practicable (i.e., nuclear power range calibration cannot be done prior to reaching power operation) in which case the testing will be resumed within 48 hours of attaining the plant condition which permits testing to be accomplished.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 101 AND 94TO

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated June 15, 1992, Northern States Power Company (the licensee) submitted an application to amend the Technical Specifications (TS). The application requests changes that would implement Generic Letter 87-09 provisions relating to missed surveillances and applicability of action requirements.

2.0 DISCUSSION AND EVALUATION

Discussion: Generic Letter 87-09, "Sections 3.0 and 4.0 of the Standard Technical Specifications (STS) on the Applicability of Limiting Conditions for Operation and Surveillance Requirements," identifies improvements to Section 4.0 of the Standard Technical Specifications which may be requested by licensees as amendments to facility Technical Specifications. One of the improvements described in Generic Letter 87-09, revised Standard Technical Specification 4.0.3 to clarify when a missed surveillance constitutes a violation of the operability requirements of a Limiting Conditions for Operation (LCO) and to clarify the applicability of the action requirements and the time during which the limits apply.

The proposed changes to the Prairie Island Technical Specifications requested in response to Generic Letter 87-09 are described below.

- A. In the Table of contents, the page number for "4.0 SURVEILLANCE REQUIREMENTS" would be changed from "4.1-1" to "4.0-1" to reflect the new Section 4.0 described below.
- B. Section 4.0 would be relocated and reformatted to be consistent with Section 3.0, which was incorporated into the Prairie Island Technical Specifications by License Amendment Nos. 91 and 84. The requirements of the current Section 4.0 would be relocated to Specification 4.0.A and be expanded to include a statement similar to Standard Technical Specification 4.0.2 which states that surveillance requirements shall be performed within the specified time intervals. The current Section 4.0 requirements would be incorporated, with only editorial changes, into the

new Specification 4.0.A as exceptions to the requirement that surveillances shall be performed within the specified time interval. These exceptions allow a plus or minus 25% to accommodate normal test schedules, with the further exception that intervals specified as refueling shutdowns not exceed two years. The proposed Specification 4.0.A will more clearly state the requirements for completion of surveillance requirements and the allowed exceptions to those requirements.

A new Specification 4.0.B would be incorporated into Section 4.0 to add Standard Technical Specification Section 4.0.3 requirements, as modified by Generic Letter 87-09, to the Prairie Island Technical Specifications. The incorporation of the proposed Specification 4.0.B would allow a delay of up to 24 hours before implementation of action requirements following discovery of a missed surveillance, thereby, giving the licensee time to complete the overdue surveillance. Also, 4.0.B would clarify that Surveillance Requirements need not be performed on inoperable equipment.

Standard Technical Specifications related to mode changes, also identified in GL 87-09, are not included in the proposed amendment.

Evaluation: The NRC Staff concluded in Generic Letter 87-09, after taking several factors into account, that 24 hours would be an acceptable time limit for completing a missed surveillance when the allowed out-of-service times of the action requirements are less than this time limit or when shutdown action requirements apply. The NRC Staff concluded that the 24-hour time limit would balance the risks associated with an allowance for completing the surveillance within this period against the risks associated with the potential for a plant upset and challenge to safety systems when the alternative is a shutdown to comply with action requirements before the surveillance can be completed. Based on the conclusion of Generic Letter 87-09 cited above, and the licensee's satisfactory past performance record with respect to timely compliance with Technical Specifications surveillance requirements, the staff concludes that the requested amendments are acceptable and should be granted.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State Official was notified of the proposed issuance of the amendment. The State Official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a

proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (57 FR 30254). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Long

Date: August 20, 1992