



July 29, 2002

Technical Specification
6.8.K

US Nuclear Regulatory Commission
Washington, DC 20555

Attn: Document Control Desk

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Technical Specification Bases Change 128a
Correct Drywell to Suppression Chamber Vacuum Breaker Indicating Light Description

Using the Monticello Technical Specification Bases Control Program, a Monticello Technical Specification Bases page has been changed. The affected page is designated with the amendment applicable at the time and the suffix "a." The change is summarized in Attachment A. A marked up page applicable at the time the change was made is provided in Attachment B. A final typed copy of the changed page that is applicable, for entry into the NRC authority copy, is provided in Attachment C. The current copy of our list of effective pages and record of revision is attached for your information, as Attachment D.

Please contact Paul Hartmann, at 763-271-5172 with any questions or comments.

Jeffrey S. Forbes
Site Vice President
Monticello Nuclear Generating Plant

- Attachment A – Summary of Technical Specification Bases Change (TSBC)
- Attachment B – Monticello Technical Specification Bases Page Marked Up With Changes
- Attachment C – Revised Monticello Technical Specification Bases Page
- Attachment D – Monticello Technical Specification List of Effective Pages and Record of Revision

cc: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
Minnesota Department of Commerce

A001

Attachment A

Summary of Technical Specification Bases Change (TSBC)

Following is a summary of the bases change forwarded herein. The change has been processed in accordance with the Monticello Technical Specification Bases Control Program described in Technical Specification 6.8.K.

TSBC-128a

Technical Specification Involved – 3.7

Page affected – 180

Summary of Change: This TSBC corrected the drywell to suppression chamber vacuum breaker indicating light description.

Attachment B

Monticello Technical Specification Bases Page
Marked Up With Changes

This attachment consists of Monticello Technical Specification bases page marked up with changes. The page included is listed below:

Page

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Bases 3.7 (Continued):

vacuum breaker selector switch, and a common test switch. The reactor building vacuum breaker panel contains one red light and one green light for each of the eight valves. There are four independent limit switches on each valve. The two switches controlling the red lights are adjusted to provide an indication of disc opening of less than 1/8" at the bottom of the disc. These switches are also used to activate the valve position alarm circuits. The two switches controlling the green lights are adjusted to provide indication of the disc very near the full open position.

The control room alarm circuits are redundant and fail safe. This assures that no simple failure will defeat alarming to the control room when a valve is open beyond allowable and when power to the switches fails. The alarm is needed to alert the operator that action must be taken to correct a malfunction or to investigate possible changes in valve position status, or both. If the alarm cannot be cleared due to the inability to establish indication of closure of one or more valves, additional testing is required. The alarm system allows the operator to make this evaluation on a timely basis. The frequency of the testing of the alarms is the same as that required for the position indication system.

Operability of a vacuum breaker valve and the four associated indicating light circuits shall be established by cycling the valve. The sequence of the indicating lights will be observed to be that previously described. If both green light circuits are inoperable, the valve shall be considered inoperable and a pressure test is required immediately and upon indication of subsequent operation. If both red light circuits are inoperable, the valve shall be considered inoperable, however, no pressure test is required if positive closure indication is present.

Oxygen concentration is limited to 4% by volume to minimize the possibility of hydrogen combustion following a loss of coolant accident. Significant quantities of hydrogen could be generated if the core cooling systems failed to sufficiently cool the core. The occurrence of primary system leakage following a major refueling outage or other scheduled shutdown is more probable than the occurrence of the loss of coolant accident upon which the specified oxygen concentration limit is based. Permitting access to the drywell for leak inspections during a startup is judged prudent in terms of the added plant safety offered without significantly reducing the margin of safety. Thus, to preclude the possibility of starting the reactor and operating for extended periods of time with significant leaks in the primary system, leak inspections are scheduled during startup periods, when the primary system is at or near rated operating temperature and pressure. The 24-hour period to provide inerting is judged to be sufficient to perform the leak inspection and establish the required oxygen concentration. The primary containment is normally slightly pressurized during periods of reactor operation. Nitrogen used for inerting could leak out of the containment but air could not leak in to increase oxygen concentration. Once the containment is filled with nitrogen to the required concentration, no monitoring of oxygen concentration is necessary. However, at least once a week the oxygen concentration will be determined as added assurance.

Attachment C

Revised Monticello Technical Specification Bases Page

This attachment consists of the revised Monticello Technical Specification Bases page that incorporates the change. This page should be entered into the NRC Authority copies of Technical Specifications. The page included is listed below:

Page

180

Bases 3.7 (Continued):

vacuum breaker selector switch, and a common test switch. The reactor building vacuum breaker panel contains one red light and one green light for each of the eight valves. There are four independent limit switches on each valve. The two switches controlling the red lights are adjusted to provide an indication of disc opening of less than 1/8" at the bottom of the disc. These switches are also used to activate the valve position alarm circuits. The two switches controlling the green lights are adjusted to provide indication of the disc very near the full open position.

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Attachment D

Monticello Technical Specification
List of Effective Pages and Record of Revision

This attachment consists of the current Monticello Technical Specification List of Effective Pages and Record of Revision. The pages included are listed below:

Page

A
B
C
D
E
F
G
H
I

MONTICELLO NUCLEAR GENERATING PLANT
APPENDIX A TECHNICAL SPECIFICATIONS RECORD OF REVISIONS

<u>Page</u>	<u>Amend No.</u>						
A	128a	37	128	72	104	124	121
B	128a	38	128	76	0	125	104
C	115	39	128	77	86	126	104
D	115	40	128	78	0	126a	87
E	115	42	103	79	0	127	128
F	115	45	0	80	29	128	42
G	115	46	70	81	3	129	122
H	119	46a	37	82	123	130	82
I	128a	47	40	82a	63	131	122
i	128	48	89	83	24	132	39
ii	104	49	128	83a	24	132a	122
iii	120	50	128	84	100a	133	106
iv	128	50a	117	85	100a	134	106
v	120	51	117	86	100a	135	106
vi	121	51a	117	87	100a	136	106
vii	122	52	128	88	100a	137	0
1	119	53	128	89	104	138	100a
2	70	54	128	90	100a	145	118a
3	21	55	103	91	123	146	106
4	102	56	102	92	100a	147	107
5	120	57	70	93	122	148	117
5a	120	58	84	94	106	149	100a
6	128	58a	29	95	77	150	128
7	128	59	128	96	77	151	128
8	128	59a	103	97	57	153	100a
9	128	60	128	98	56	154	100a
10	128	60a	31	99	104	155	122
11	128	60b	62	100	100a	156	93
12	128	60c	30	101	122	157	117
25a	127	60d	128	102	122	158	107
25b	127	60e	89	103	122	159	95
25c	127	61	104	104	122	160	95
25d	127	62	117	105	122	163	0
26	5	63	117	106	79	164	104
27	81	63a	117	107	97	165	64
27a	81	64	128	108	128	166	94
28	128	65	117	109	100a	167	112
29	128	66	119a	110	100a	168	94
30	103	67	117	111	122	169	94
31	104	68	102	112	124a	170	122
32	103	69	118a	113	122	171	96
33	103	69a	100a	114	122	172	71
34	83	70	117	121	0	175	107
35	100a	71	100a	122	106	175a	117
36	128	71a	105	123	117	176	100a

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<u>Page</u>	<u>Amend No.</u>	<u>Page</u>	<u>Amend No.</u>
177	117	229a	63
178	100a	229b	104
179	123a	229c	104
180	128a	229d	63
181	100a	229e	122
182	112	229u	104
183	117	229v	112
184	100a	229v v	112
185	100a	229w	112
188	104	229ww	112
189	100a	229x	112
190	104	229y	115a
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193	121	231	34
196	126a	232	119
197	121	233	124
198	121	234	119
199	51	235	115
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201	77	243	128
202	80	244	124
203	41	248	59
204	100a	249	120
205	100a	250	128
206	0	251	124
207	123	252	120
208	63	253	120
209	123	254	120
209a	100a	255	120
210	100a	256	122
211	109	257	122
212	109	258	122
213	99	259	120
216	100a	260	120
217	128	261	120
218	120	262	120
223	119		
224	119		
225	119		
226	119		

**MONTICELLO NUCLEAR GENERATING PLANT
RECORD OF TECHNICAL SPECIFICATION CHANGES AND LICENSE AMENDMENTS**

<u>NSP Page Revision (REV) No.</u>	<u>License DPR-22 Amend No. & Date</u>	<u>AEC Tech Spec Change Issuance No. and date</u>	<u>Major Subject</u>
Original	-	-	Appendix A Technical Specifications incorporated in DPR-22 on 9/8/70
-	1 1/19/71	Note 1	Removed 5 MWt restriction
-	Note 2	2 1/14/72	MOGS Technical Specification changes issued by AEC but never distributed or put into effect, superseded by TS Change 12 11/15/73
1	Note 2	3 10/31/72	RHR service water pump capability change
-	Note 2	4 12/8/72	Temporary surveillance test waiver
-	2 2/20/73	Note 1	Increase in U-235 allowed in fission chambers
2	Note 2	5 3/2/73	Miscellaneous Technical Specification changes,
3	Note 2	1 4/28/71& 6 4/3/73	Respiratory Protection, & Administrative Control Changes
4	Note 2	7 5/4/73	Respiratory Protection Changes
5	Note 2	8 7/2/73	Relief Valve and CRD Scram Time Changes
6	Note 2	9 8/24/73	Fuel Densification Limits
7	Note 2	10 10/2/73	Safety Valve Setpoint Change
8	Note 2	11 11/27/73& 12 11/15/73	Offgas Holdup System, RWM, and Miscellaneous Changes
9	Note 2	13 3/30/74	8x8 Fuel Load Authorization
10	3	14 5/14/74	8x8 Full Power authorization
-	4 6/17/74	Note 1	Changed byproduct material allowance
-	6 8/20/74	Note 1	Changed byproduct material allowance
11	Note 3	Note 3 10/24/74	Inverted Tube (CRD) Limits
12	5	15 1/15/75	REMP Changes
13	7	16 2/3/75	Reactor Vessel Surveillance Program Changes
14	8	17 2/26/75	Vacuum Breaker Test Changes
15	9	18 4/10/75	Corrects Errors & Provides Clarification
-	10 7/8/75	Note 1	Increased allowed quantity of U-235
16	12	20 9/15/75	Snubber Requirements
17	11	19 9/17/75	Removed byproduct material allowance
18	13	21 10/6/75	Suppression Pool Temperature Limits
19	14	22 10/30/75	Appendix K and GETAB Limits
20	15 1/22/76 NOTE 4		Reporting Requirements
21	16 2/3/76		CRD Collet Failure Surveillance
22	17 3/16/76		NSP Organization Changes
23	NOTE 3 4/13/76		Adoption of GETAB
24	18 4/14/76		Containment Isolation Valve Testing
25	21 5/20/76		Interim Appendix B, Section 2.4 Tech. Specs.

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<u>NSP Revision (REV) No.</u>	<u>License DPR-22 Amend No. & Date</u>	<u>Major Subject</u>
26	19 5/27/76	Low Steamline Pressure Setpoint and MCPR Changes
27	20 6/18/76	APLHGR, LHGR, MCPR Limits
28	22 7/13/76	Correction of Errors and Environmental Reporting
29	23 9/27/76	Standby Gas Treatment System Surveillance
30	24 10/15/76	CRD Test Frequency
31	25 10/27/76	Snubber Testing Changes
32	26 4/1/77	APRS Test Method
33	27 5/24/77	MAPLHGR Clamp at Reduced Flow
34	28 6/10/77	Radiation Protection Supervisor Qualification
35	29 9/16/77	REMP Changes
36	30 9/28/77	More Restrictive MCPR
37	31 10/14/77	Inservice Inspection Changes
38	32 12/9/77	Reporting Requirements
39	33 1/25/78	Fire Protection Requirements
NOTE 1	34 4/14/78	Increase in spent fuel storage capacity
40	35 9/15/78	RPT Requirements
41	36 10/30/78	Suppression Pool Surveillance
42	37 11/6/78	8x8R Authorization, MCPR Limits & SRV Setpoints
43	NOTE 3 11/24/78	Corrected Downcomer Submergence
44	38 3/15/79	Incorporation of Physical Security Plan into License
45	39 5/15/79	Revised LPCI Flow Capability
46	40 6/5/79	Respiratory Protection Program Changes
47	41 8/29/79	Fire Protection Safety Evaluation Report
48	42 12/28/79	MAPLHGR vs. Exposure Table
49	43 2/12/80	MCPR & MAPLHGR Changes for Cycle 8 and Extended Core Burnup
50	44 2/29/80	ILRT Requirements
NOTE 1	- 8/29/80	Order for Modification of License-Environmental Qualification
NOTE 1	- 9/19/80	Revised Order for Modification of License-Environmental Qualification
51	- 10/24/80	Order for Modification of License-Environmental Qualification Records
52	- 1/9/81	Issuance of Facility Operating License (FTOL)
NOTE 1	- 1/9/81	Order for Modification of License Concerning BWR Scram Discharge Systems (License conditions removed per Amendment No. 11 dated 10/8/82)

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<u>NSP Revision (REV) No.</u>	<u>License DPR-22 Amend No. & Date</u>	<u>Major Subject</u>
NOTE 1	- 1/13/81	Order for Modification Mark I Containment
-	1 2/12/81	Revision of License Conditions Relating to Fire Protection Modifications
53	2 3/2/81	TMI Lessons Learned & Safety - Related Hydraulic Snubber Additions
54	3 3/27/81	Low voltage protection, organization and miscellaneous
NOTE 1	4 3/27/81	Incorporation of Safeguards Contingency Plan and Security Force Qualification and Training Plan into License
55	5 5/4/81	Cycle 9 - OLYN Changes, New MAPLHGR's, RPS Response time change
56	6 6/3/81	Inservice Inspection Program
57	7 6/30/81	Fire Protection Technical Specification Changes
58	8 11/5/81	Mark I Containment Modifications
59	9 12/28/81	Inservice Surveillance Requirements for Snubbers
NOTE 1	- 1/19/82	Revised Order for Modification Mark I Containment
60	10 5/20/82	Scram Discharge Volume
61	11 10/8/82	New Scram Discharge Volumes
62	12 11/30/82	RPS Power Monitor
63	13 12/6/82	Cycle 10
64	14 12/10/82	Recirc Piping and Coolant Leak Detection
65	15 12/17/82	Appendix I Technical Specifications (removed App. B)
66	16 4/18/83	Organizational Changes
67	17 4/17/83	Miscellaneous Changes
68	18 11/28/83	Steam Line Temperature Switch Setpoint
69	19 12/30/83	Radiation Protection Program
70	20 1/16/84	SRM Count Rate
71	21 1/23/84	Definition of Operability
72	22 2/2/84	Miscellaneous Technical Specification Changes
73	23 4/3/84	RPS Electrical Protection Assembly Time Delay
74	24 5/1/84	Scram Discharge Volume Vent and Drain Valves
75	25 8/15/84	Miscellaneous Technical Specification Changes
76	26 9/24/84	Cycle 11
77	27 10/31/84	RHR Intertie Line Addition
78	28 11/2/84	Hybrid I Control Rod Assembly
79	29 11/16/84	ARTS
80	30 11/16/84	Low Low Set Logic
81	31 11/27/84	Degraded Voltage Protection Logic

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<u>NSP Revision (REV) No.</u>	<u>License DPR-22 Amend No. & Date</u>	<u>Major Subject</u>
82	32 5/28/85	Surveillance Requirements
83	33 10/7/85	Screen Wash/Fire Pump (Partial)
84	34 10/8/85	Fuel Enrichment Limits
85	35 12/3/85	Combustible Gas Control System
86	36 12/23/85	Vacuum Breaker Cycling
87	37 1/22/86	NUREG-0737 Technical Specifications
88	38 2/12/86	Environmental Technical Specifications
89	39 3/13/86	Administrative Changes
90	40 3/18/86	Clarification of Radiation Monitor Requirements
91	41 3/24/86	250 Volt Battery
92	42 3/27/86	Jet Pump Surveillance
93	43 4/8/86	Simmer Margin Improvement
94	44 5/27/86	Cycle 12 Operation
95	45 7/1/86	Miscellaneous Changes
96	46 7/1/86	LER Reporting and Miscellaneous Changes
97	47 10/22/86	Single Loop Operation
98	48 12/1/86	Offgas System Trip
99	49 8/26/87	Rod Block Monitor
100	50 8/26/87	APRM and IRM Scram Requirements
101	51 10/16/87	2R Transformer
102	52 11/18/87	Surveillance Intervals - ILRT Schedule
103	53 11/19/87	Extension of Operating License
104	54 11/25/87	Cycle 13 and Misc Changes
105	55 11/25/87	Appendix J Testing
106	56 12/11/87	ATWS - Enriched Boron
107	57 9/23/88	Increased Boron Enrichment
108	58 12/13/88	Physical Security Plan
109	59 2/16/89	Miscellaneous Administrative Changes
110	60 2/28/89	Miscellaneous Administrative Changes
111	61 3/29/89	Fire Protection and Detection System
112	62 3/31/89	ADS Logic and S/RV Discharge Pipe Pressure
113	63 4/18/89	Miscellaneous Technical Specification Improvements
114	64 5/10/89	Containment Vent and Purge Valves
115	65 5/30/89	NUREG-0737 - Generic Letter 83-36
116	66 5/30/89	Reactor Vessel Level Instrumentation

MONTICELLO NUCLEAR GENERATING PLANT
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<u>NSP Revision (REV) No.</u>	<u>License DPR-22 Amend No. & Date</u>	<u>Major Subject</u>
117	67 6/19/89	Extension of MAPLHGR. Exposure for One Fuel Type
118	68 7/14/89	SRO Requirements & Organization Chart Removal
119	69 9/12/89	Operations Committee Quorum Requirements
120	70 9/28/89	Relocation of Cycle-Specific Thermal-Hydraulic Limits
121	71 10/19/89	Deletion of Primary Containment Isolation Valve Table
122	72 11/2/89	RG 1.99, Rev 2, ISI & ILRT
123	73 5/1/90	Combined STA/LSO Position
124	74 6/5/90	Removal of WRGM Automatic ESF Actuation
125	75 10/12/90	Diesel Fuel Oil Storage
126	76 12/20/90	Miscellaneous Administrative Changes
127	77 2/15/91	Redundant and IST Testing
128	78 3/28/91	Alarming Dosimetry
125	79 4/9/91	SAFER/GESTR
130	80 8/12/91	Torus Vacuum Breaker Test Switch/EDG Fuel Oil Tank Level
131	81 4/16/92	Surveillance Test Interval Extension - Part I
132	82 7/15/92	Alternate Snubber Visual Inspection Intervals
133	83 8/18/92	Revisions to Reactor Protection System Tech Specs
134	84 1/27/93	MELLIA and Increase Core Flow
135	85 6/29/93	Revision to Diesel Fire Pump Fuel Oil Sampling Requirements
136	86 7/12/93	Revisions to Control Rod Drive Testing Requirements
137	87 4/15/94	Revised Coolant Leakage Monitoring Frequency
138	88 6/30/94	Average Planar Linear Heat Generation Rate (APLHGR) Specification & Minimum Critical Power Ratio Bases Revisions
139	89 8/25/94	Removal of Chlorine Detection Requirements and Changes to Control Room Ventilation System Requirements
140	90 9/7/94	Revisions to Radiological Effluent Specifications
141	91 9/9/94	Secondary Containment System and Standby Gas Treatment System Water Level Setpoint Change
142	92 9/15/94	Change in Safety Relief Valves Testing Requirements
143	93 7/12/95	Revised Core Spray Pump Flow
144	94 10/2/95	Standby Gas Treatment and Secondary Containment Systems
145	95 4/3/96	MSIV Combined Leakrate, and Appendix J, Option B
146	96 4/9/96	Purge and Vent Valve Seal Replacement Interval
147	97 9/17/96	Implementation of BRWOG Option I-D core Stability Solution and re-issue of pages 11, 12, 82 and 231 to reflect pages issued by NRC amendments.

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NSP Revision (REV) No.	License DPR-22 Amend No. & Date	<u>Major Subject</u>
148	98 7/25/97	Bases changes on containment overpressure and number of RHR pumps required to be operable. Reissue pages 207, 209, 219, 229k, 229p, 230, 245 to reflect pages issued by NRC amendments.
149	99 10/29/97	SLMCPR for Cycle 18 and reissue pages vi, 155, 202, 207, 219, 229u
NOTE 5	11/25/97	Reissue pages a, b, g, iii, vi, 14, 25a, 155, 198y, 198z, 202, 207, 209, 219, 229k, 229p, 229r, 229u, 230, 245
150	100 4/20/98	SLMCPR for Cycle 19
NOTE 6	100a 4/30/98	Reissue all pages.
	101 08/28/98	Reactor Coolant Equivalent Radioiodine Concentration and Control Room Habitability
	102 09/16/98	Monticello Power Rerate
	103 12/23/98	Surveillance Test Interval/Allowed Outage Time Extension Program - Part 2
	104 12/24/98	Revision of Statement on Shift Length & other Misc Changes
	105 03/19/99	CST Low Level HPCI/RCIC Suction Transfer
	106 10/12/99	Revised RPV-PT Curves & remove SBLC RV setpoint
	107 11/24/99	Reactor Pressure Vessel Hydrostatic and Leakage Testing
	108 12/8/99	Testing Requirements for Control Room EFT Filters
	109 02/16/00	Safety Limit Minimum Critical Power Ratio for Cycle 20
	110 08/07/00	Transfer of Operating Authority from NSP to NMC
	111 08/18/00	Transfer of Operating License from NSP to a New Utility Operating Company
	112 08/18/00	Emergency Filtration Train Testing Exceptions and Technical Specification Revisions
	113 10/02/00	Alternate Shutdown System Operability Requirements
	114 11/30/00	Safety/Relief Valve Bellows Leak Detection System Test Frequency
	115 12/21/00	Administrative Controls and Other Miscellaneous Changes
	115a 02/13/01	Bases Change to Reflect Modification 98Q145 Installed Control Room Toxic Gas Air Supply
	116 03/01/01	Relocation of Inservice Inspection Requirements to a Licensee Program
	117 03/07/01	Reactor Water Cleanup (RWCU) System Automatic Isolation and Miscellaneous Instrumentation System Changes
	118 03/09/01	Revision of Standby Liquid Control System Surveillance Requirements
	118a 05/10/01	Bases Change - 50°F Loop Temperature, Bus Transfer & Rerate Correction

**MONTICELLO NUCLEAR GENERATING PLANT
RECORD OF TECHNICAL SPECIFICATION CHANGES AND LICENSE AMENDMENTS**

NSP Revision (REV) No.	License DPR-22 Amend No. & Date	Major Subject
	119 04/05/01	Fire Protection Technical Specification Changes
	119a 06/28/01	Bases Change - Added information on cooldown rate
	120 07/24/01	Relocation of Radiological Effluent Technical Specifications to a Licensee-Controlled Program
	121 07/25/01	Clarify air ejector offgas activity sample point and operability requirements
	122 08/01/01	Relocation of Inservice Testing Requirements to a Licensee-Controlled Program
	122a 10/22/01	Bases Change - Remove scram setpoints sentence and correct typo
	123 10/26/01	Control Rod Drive and Core Monitoring Technical Specification Changes
	123a 10/25/01	Bases Change - Change to reflect new operation of drywell to suppression chamber vacuum breaker valve position indicating lights
	124 10/30/01	Relocation of Technical Specification Administrative Controls Related to Quality Assurance Plan
	124a 12/05/01	Bases Change - Change to reflect revised Technical Specification definition of a containment spray/cooling subsystem
	125 12/06/01	Safety Limit Minimum Critical Power Ratio for Cycle 21
	126 01/18/02	Elimination of Local Suppression Pool Temperature Limits
	126a 02/15/02	Bases Change - Change reflects relocation of sample point for the offgas radiation monitor
	127 05/31/02	Missed Surveillance Requirement Technical Specification Changes
	128 06/11/02	Changes to the Technical Specifications Revised Reference Point for Reactor Vessel Level Setpoints, Simplification of Safety Limits, and Improvement to the Bases
	128a 07/11/02	Bases Change - Correct Drywell to Suppression Chamber Vacuum Breaker Indicating Light Description

1. License Amendment or Order for Modification of License not affecting Technical Specifications.
2. Technical Specification change issued prior to 10 CFR revisions which require issuance of Technical Specification changes as License Amendments.
3. Modification to Bases. No Technical Specification change or License Amendment issued.
4. Technical Specification change numbers no longer assigned beginning with Amendment 15.
5. Pages reissued 11/25/97 to conform with NRC version. Revision number of effected pages not changed.
6. All pages reissued using INTERLEAF in different font. Using NRC Amendment Nos. and issue date. For Bases and Table of Contents, spelling errors corrected and editorial corrections made and all Amendment Nos. changed to 100a. For remaining Tech Spec pages, no other changes made and current Amendment Nos. used.