Mr. J. A. Price August 7, 2002 Vice President - Nuclear Technical Services - Millstone Dominion Nuclear Connecticut, Inc. c/o Mr. David A. Smith Rope Ferry Road Waterford, Connecticut 06385

SUBJECT: MILLSTONE POWER STATION, UNIT 1 - BASES CHANGES FOR UNIT 1

CENTRAL MONITORING STATION (TAC NO. MB5633)

Dear Mr. Price:

The staff has incorporated the revision of the technical specification (TS) Bases provided by your letter of December 19, 2001. The change for TS 3.1.1, "Fuel Storage Pool Water Level," was made to reflect the change in location for control and monitoring of Millstone Unit No. 1 from the Central Monitoring Station to the Millstone Unit No. 2 Control Room.

The staff has reviewed the change and finds the revision to the Bases to be acceptable.

Sincerely,

/RA/

John B. Hickman, Project Manager, Section 2 Project Directorate IV Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-245

Enclosure: Bases Page B 3.1-1

cc w/encl: See next page

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Project Directorate IV

Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-245 <u>DISTRIBUTION</u>:

PUBLIC

Enclosure: Bases Page B 3.1-1 PDIV-2 Reading

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ACCESSION NO.: ML022190025 EXCEL 1011 NRR-106

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Millstone Power Station Unit 1

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B 3.1 DEFUELED SYSTEMS

B 3.1.1 Fuel Storage Pool Water Level

BASES

BACKGROUND

The minimum water level in the spent fuel storage pool meets the assumptions of iodine decontamination factors following a fuel handling accident. A general description of the spent fuel storage pool design is found in Chapter 3 of the DSAR, (Ref. 1). The assumptions of the fuel handling accident are found in Chapter 5 of the DSAR (Ref. 2).

APPLICABLE SAFETY ANALYSIS

Although the unit is permanently shutdown and defueled, fuel handling accidents in the fuel storage pool are still possible.

A bounding calculation of the radiological consequences of such an accident in the spent fuel pool was performed, based on the following:

- Actual source term radioactive decay since shutdown credited
- Failure of four assemblies 248 fuel rods in four 8 x 8 assemblies
- Unfiltered ground release no credit for secondary containment or standby gas treatment

The analysis concluded that 1) calculated doses at the exclusion area boundary and the low population zone are within 10CFR100 limits; and 2) calculated doses to the Millstone Units 2 and 3 Control Rooms are within the limits set in GDC-19.

(continued)