SEP 12 1984

Docket Nos. 50-282
and 50-306

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Mr. D. M. Musolf
Nuclear Support Services Department
Northern States Power Company
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

The Commission has issued the enclosed Amendment Nos. 70 and 64to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2 in response to your application dated April 10, 1984 as supplemented by letter dated July 9, 1984.

The amendments are in response to the matters listed below.

1. Administrative requirements related to Post Accident Sampling (NUREG-0737 Item II.B.3 and II.F.1.2).

2. Radiation Environmental Monitoring Program dealing with maximum lower level of detection for Cs 134 and 137 (TS 4.10, Table TS 4.10-2, page 1 of 2).

3. Radiation Environmental Monitoring Program deletion of Figure TS 4.10-1 and TS 4.10-2.

4. Administrative Control Organization title change (Figure TS 6.1-2).

5. Wording change involving operator action related to the number of operable instrumentation channels associated with containment ventilation isolation system (Table TS 3.5-4).

6. Typographical errors and misspelled word corrections.

By your letter dated July 9, 1984 you withdrew the TS change request related to the toxic gaseous detection system because of a new study concluding that these gaseous detection systems are no longer necessary. You are requested to maintain the toxic gaseous detection systems in an operable condition until we have completed our evaluation of your submittal.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by:

Dominic C. DiIanni, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 70 to DPR-42
2. Amendment No. 64 to DPR-60
3. Safety Evaluation

cc w/enclosures:
See next page

Concur subject to (license note)

ORB#3:DL	~~ORB#3:DL~~	ORB#3:DL	OELD	AD:OR:DL	RAB:DSI	Items 2+3
PMKreutzer	DCDilanni/pn	JRMiller		GCLainas	FJC	only
8/ /84	8/21/84	8/ /84	8/ /84	8/13/84	9/4/84	FJC

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated April 10, 1984 as supplemented July 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (1) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 70, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 12, 1984.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated April 10, 1984 as supplemented July 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (1) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 64 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 12, 1984

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NOS. 70 AND 64 TO FACILITY OPERATING LICENSE

NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of changes.

Remove	Insert
TS-iv	TS-iv
Table TS.3.5-4	Table TS.3.5-4
Table TS.4.2-1	Table TS.4.2-1
Table TS.4.4-1 (page 4 of 5)	Table TS.4.4-1 (page 4 of 5)
Table TS.4.10-2 (page 1 of 2)	Table TS.4.10-2 (page 1 of 2)
Figure TS.4.10-1	-
Figure TS.4.10-2	-
Figure TS.6.1-2	Figure TS.6.1-2
TS.6.5-2	TS.6.5-2
TS.6.5-3	TS.6.5-3
TS.6.7-3	TS.6.7-3

APPENDIX A TECHNICAL SPECIFICATIONS

LIST OF FIGURES

Unit 1 -- Amendment No. 40, 70
Unit 2 -- Amendment No. 40, 64

TABLE TS.3.5-4

INSTRUMENT OPERATING CONDITIONS FOR ISOLATION FUNCTIONS

FUNCTIONAL UNIT	1 MINIMUM OPERABLE CHANNELS	2 MINIMUM DEGREE OF REDUNDANCY	3 PERMISSIBLE BYPASS CONDITIONS	4 OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET
1. CONTAINMENT ISOLATION				
a. Safety Injection	(See Item No. 1 of Table TS.3.5-3)			Hot shutdown**
b. Manual	2	1		Hot shutdown
2. CONTAINMENT VENTILATION ISOLATION				
a. Safety Injection	(See Item No. 1 of Table TS.3.5-3)			Maintain Purge and Inservice Purge Valves closed if (1) conditions of a, b, or c cannot be met or (2) if conditions of b and c cannot be met during fuel handling in containment.
b. High Radiation in Exhaust Air	2	1		
c. Manual	2	1		
3. STEAM LINE ISOLATION				
a. Hi-Hi Steam Flow with Safety Injection	2	1		Hot shutdown**
b. Hi Steam Flow and 2 of 4 Low T_{avg} with Safety Injection	2	1		Hot shutdown**
c. Hi Containment Pressure	1/loop	1		Hot shutdown**
d. Manual	1/loop	-		Hot shutdown**
4. EMERGENCY COOLDOWN EQUIPMENT ROOM ISOLATION				
a. High temperature in ventilation system ducts	2	1		Hot shutdown**

**If minimum conditions are not met within 24 hours, steps shall be taken on the affected unit to place the unit in cold shutdown conditions.

SPECIAL INSERVICE INSPECTION REQUIREMENTS

Component	Method of Examination	Extent and Frequency
REACTOR COOLANT PUMPS		
1. Pump Flywheel	U.T.	An in-place ultrasonic volumetric examination of the areas of higher stress concentration at the bore and key way at approx. 3 year intervals, during the refueling or maintenance shutdown coinciding with the in-service inspection schedule as required by the ASME B & PV Code Section XI.
	M.T. or P.T. U.T.	A surface examination of all exposed surfaces and complete ultrasonic volumetric examination at approx. 10 year intervals, during the plant shutdown coinciding with the in-service inspection schedule as required by the ASME B & PV Code Section XI. Removal of the flywheel is not required to perform these examinations.

Notes:

1. The following definitions shall apply to the inspection methods employed in Table TS.4.2-1.

   a. U.T. - Ultrasonic examination per IWA-2230.
   b. P.T. - Liquid Penetrant examination per IWA-2220.
   c. M.T. - Magnetic Particle examination per IWA-2220.

UNIT 1 AND UNIT 2 PENETRATION DESIGNATION FOR LEAKAGE TESTS

Penetration Number	Penetration Description	Penetration Designation (Note 3)	Type of Test
42B (53 in Unit 2)	Inservice Purge Supply Valves (6)	ABSVZ	C
42B (53 in Unit 2)	*Inservice Purge Supply Blind Flange(4)	Annulus	B
42C (54 in Unit 2)	Containment Heating Steam (4)	ABSVZ	B
42D, 42E	Spare		-
42F-1 (42E-1 in Unit 2)	Heating Steam Condensate Return(4)	ABSVZ	B
42F-2 (42E-2 in Unit 2)	Heating Steam Return Vent(4)	ABSVZ	B
42G	Spare		
43A (52 in Unit 2)	Inservice Purge Exhaust Valves(6)	ABSVZ	C
43A (52 in Unit 2)	*Inservice Purge Exhaust Blind Flange(4)	Annulus	B
43B,C,D	Spares		
44	Containment Vessel Pressurization (4)	ABSVZ	B
45	Reactor Makeup to Pressurizer Relief Tank	ABSVZ	C
46A, 46B (46C, 46D in Unit 2)	Auxiliary Feedwater	Note (2)	-
47	Electrical Penetration	Annulus	B
48	Low Head SI	Note (5)	-
49A	Instrumentation	Note (1)	-
49B (55 in Unit 2)	Demineralized Water (4)	ABSV	B

*Testing required following modification to inservice purge system of each unit during 1983 refueling outages.

TABLE TS.4.10-2

MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION (LLD) [a,e]

Analysis	Water (pCi/l)	Airborne Particulate or Gas (pCi/m^3)	Fish (pCi/kg, wet)	Milk (pCi/l)	Food Products (pCi/kg, wet)	Sediment (pCi/kg, dry)
gross beta	4[b]	1×10^{-2}				
^{3}H	2000(1000[b])					
^{54}Mn	15		130			
^{59}Fe	30		260			
$^{58,60}Co$	15		130			
^{65}Zn	30		260			
^{95}Zr-Nb	15[c]					
^{131}I	1[b,d]	7×10^{-2}		1[d]	60	
$^{134,137}Cs$	15(10[b]), 18	1×10^{-2}	130	15	60	150
^{140}Ba-La	15[c]			15[c]		

TABLE TS.4.10-2 (Page 1 of 2)

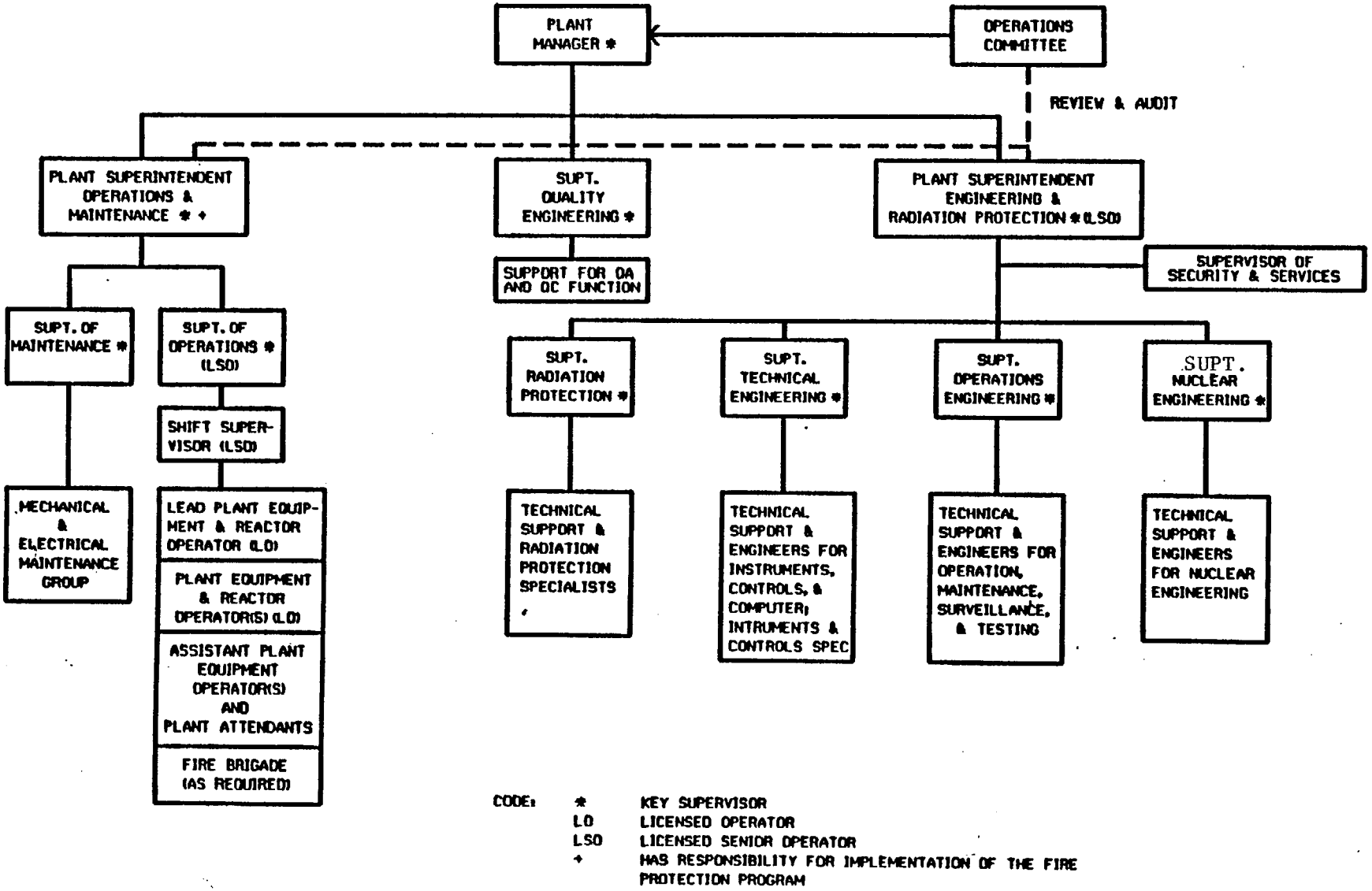

FIGURE TS.6.1-2 PRAIRIE ISLAND NUCLEAR GENERATING PLANT
FUNCTIONAL ORGANIZATION FOR ON-SITE GROUP

FIGURE TS.6.1-2

1. a. Paragraph 20.203 "Caution signs, labels, signals and controls". In lieu of the "Control device" or alarm signal required by paragraph 20.203(c)(2), each high radiation area in which the intensity of radiation is 1000 mRem/hr or less shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (or continuous escort by a qualified person for the purpose of making a radiation survey) and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

   b. The above procedure shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mRem/hr, except that doors shall be locked or attended to prevent unauthorized entry into these areas and the keys or key devices for locked doors shall be maintained under the administrative control of the Plant Manager.

2. A program shall be implemented to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

   a. Provisions establishing preventive maintenance and periodic visual inspection requirements, and

   b. Integrated leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

   A program acceptable to the Commission was described in letters from L.O. Mayer, NSP, to Director of Nuclear Reactor Regulation, dated December 31, 1979 "Lessons Learned Implementation" and March 13, 1980, "1/1/80 Lessons Learned Implementation Additional Information".

3. A program shall be implemented which will ensure the capability to accurately determine the airborne iodine concentration in essential plant areas under accident conditions. This program shall include the following:

   a. Training of personnel,

   b. Procedures for monitoring, and

   c. Provisions for maintenance of sampling and analysis equipment.

   A program acceptable to the Commission was described in letters from L.O. Mayer, NSP, to Director of Nuclear Reactor Regulation, dated December 31, 1979 "Lessons Learned Implementation" and March 13, 1980, "1/1/80 Lessons Learned Implementation Additional Information".

4. A program shall be implemented which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant

gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

a. Training of personnel,

b. Procedures for sampling and analysis,

c. Provisions for maintenance of sampling and analysis equipment.

C. Maintenance and Test

The following maintenance and test procedures will be developed to satisfy routine inspection, preventive maintenance programs, and operating license requirements.

1. Routine testing of Engineered Safeguards and equipment as required by the facility License and the Technical Specifications.

2. Routine testing of standby and redundant equipment.

3. Preventive or corrective maintenance of plant equipment and systems that could have an effect on nuclear safety.

4. Calibration and preventive maintenance of instrumentation that could affect the nuclear safety of the plant.

5. Special testing of equipment for proposed changes to operational procedures or proposed system design changes.

D. Process Control Program (PCP)

The PCP shall be approved by the Commission prior to initial implementation. Changes to the PCP shall satisfy the following requirements:

1. A description of changes shall be submitted to the Commission with the Semi-Annual Radioactive Effluent Release Report for the period in which the change(s) were made. This submittal shall contain:

   a. sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;

   b. a determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and

   c. documentation of the fact that the change has been reviewed and found acceptable by the Operations Committee.

2. Shall become effective upon review and acceptance by the Operations Committee.

5. Semiannual Radioactive Effluent Release Report. Routine radioactive effluent release reports covering the operation of the unit during the previous six months of operation shall be submitted within 60 days after January 1st and July 1st of each year.

   The radioactive effluent release reports shall include a summary of the quantities of radioactive liquid and gaseous effluents as outlined in Appendix B of Regulatory Guide 1.21, Revision 1, June, 1974, with data summarized on a quarterly basis.

   The report to be submitted 60 days after January 1 of each year shall include an assessment of the radiation doses from radioactive effluents released from the plant during the previous calendar year. This same report shall also include an assessment of the radiation doses from radioactive liquid and gaseous effluents to individuals due to their activities inside the site boundary (Figures 3.9-1 and 3.9-2) during the report period. All assumptions used in making these assessments (i.e., specific activity, exposure time and location) shall be included in these reports. The assessment of radiation doses shall be performed in accordance with the Offsite Dose Calculation Manual (ODCM) or standard NRC computer codes.

   The report to be submitted 60 days after January 1 of each year shall also include an assessment of radiation doses to the likely most exposed member of the general public from reactor releases and other nearby uranium fuel cycle sources (including doses from primary effluent pathways and direct radiation) for the previous 12 consecutive months to show conformance with 40 CFR 190, Environmental Radiation Protection Standards for Nuclear Power Operation.

   The radioactive effluent release reports shall include the following information for solid waste shipped offsite during the report period.

   a. container volume,
   b. total curie quantity (specify whether determined by measurement or estimate).
   c. principal radionuclides (specify whether determined by measurement or estimate),
   d. type of waste (e.g., spent resin, compacted dry waste, evaporator bottoms),
   e. type of container (e.g., LSA, Type A, Type B, Large Quantity), and
   f. solidification agent (e.g., cement, urea formaldehyde).

   The radioactive effluent releases reports shall include unplanned releases from the site of radioactive materials in gaseous and liquid effluents on a quarterly basis, changes to the ODCM, a description of changes to the PCP, a report of when milk or vegetable samples cannot be obtained as required by Table 4.10-1, and changes in land use resulting in significant increases in calculated doses.

6. Annual Summaries of Meteorological Data. An annual summary of meteorological data shall be submitted for the previous calendar year in the form of joint frequency distributions of wind speed, wind direction, and atmospheric stability at the request of the Commission.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 70 AND 64

TO FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

Introduction

By letter dated April 10, 1984 as supplemented July 9, 1984, Northern States Power Company (NSP), the licensee, requested amendments to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2 (PINGP). The requested amendments proposed changes to the Technical Specifications (TS) in the following areas.

1. TS 6.5.B.4 - Administrative requirements related to Post Accident Sampling (NUREG-0737 Items II.B.3 and II.F.1.2) (Pages TS.6.5-2 and 6.5-3).

2. TS.4-10 - Radiation Environmental Monitoring Program - Change the maximum lower limit of detection for Cs 134 and 137 in food products (Table TS.4.10-2; page 1 of 2).

3. TS.4.10 - Radiation Environmental Monitoring Program - Delete Figures TS.4.10-1 and TS.4.10-2.

4. TS.6.1 - Administrative Controls Organization - Change the title of the Nuclear Engineering department head from Senior Nuclear Engineer to Superintendent Nuclear Engineering appearing in the plant organization chart of the technical specification (Figure TS.6.1-2).

5. TS.3.5 - A wording change involving operator action related to the number of operable instrumentation channels associated with the containment ventilation isolation system. The change clarifies the purge valve position during fuel handling while the plant is shutdown having no effect when the plant is above cold shutdown (Table TS.3.5-4).

6. The licensee reviewed the TS for typographical and spelling errors. The following corrections were proposed based on the review.

   a. Correct typo on page TS-iv line 6.1-1 by changing "On-site to "On-Site".

   b. Correct typo in TS 3.1.A.4.c by changing "that" to "than".

c. Correct typo in TS 3.1.A.4.g by changing "blocks" to "block".

d. Correct typo in second paragraph on page TS.3.5-3 by changing "exhause" to "exhaust".

e. Correct typo in TS 3.14.F by changing "Hydant" to "Hydrant".

f. Correct typo in TS 3.14.F.1.e by changing "house" to "House".

g. Correct typo in Table TS.4.2-1 by changing "ASME B & PB" to "ASME B & PV".

h. Correct typo in note at the bottom of Table TS.4.4-1 (pg 4 of 5) by changing "refuleing" to "refueling".

i. Correct typos on page TS.4.5-4 by changing "Deviaitons" to "Deviations", "throughtout" to "throughout", "presure" to "pressure" and "perfformance" to performance".

j. Correct typo in TS.6.5.B.3 by changing "March 31, 1979" to "March 13, 1980".

k. Correct typo in TS.6.7.A.5 by changing "Table 4.16.1" to "Table 4.10-1".

l. Correct typo in TS.6-6 by changing "zirconium reactors" to "zirconium reaction".

Items 1 and 2 above fall into the category where the licensee proposes to expand the scope of limited conditions for plant operation and expand the maintenance surveillance of plant equipment due to the NRC staff imposing additional plant requirements. Items 3 thru 6 above fall in the category of administrative changes related to eliminating areas in the TS that could lead to confusion and inaccuracies. By letter dated July 9, 1984 the licensee has withdrawn the proposed TS related to the surveillance of the toxic gas monitors (i.e., ammonia, formaldehyde and hydrochloric acid).

1. TS 6.5.B.4 - Administrative requirements for Post Accident Sampling - NUREG-0737 Items II.B.3 and II.F.1.2

The licensee has provided a post accident sampling system (PASS) that was designed based on the criteria guidelines of NUREG-0737 Item II.B.3. PASS permits the licensee to promptly obtain reactor coolant and containment atmosphere samples under accident conditions. The samples are then analyzed for certain radionuclides (i.e., noble gases, iodine, cesiums and nonvolatile isotopes) to determine the degree of core damage. By letter dated November 23, 1983, the NRC staff issued a safety evaluation concluding that the licensee met four of the five criteria and the last remaining criterion dealing with estimating the extent of core damage was found acceptable on an interim basis. By letter dated December 22, 1983, the licensee responded to areas in the safety evaluation requiring clarification. The staff finds the licensee's response to the clarifications acceptable. The procedure for estimating core damage, based on the Westinghouse Owners Group Generic criterion, has been

submitted by the licensee and is under staff review. This is the only remaining open issue related to PASS and has no effect on the proposed administrative requirement associated with PASS that would appear in the TS. The licensee used the staff's model standard technical specifications (STS) as guidance in preparing proposed administrative requirements for PASS. The staff's review of the licensee's proposed TS change shows that the proposed administrative requirements of the proposed TS meet the guidance of the model STS. On this basis, the proposed TSs associated with PASS are acceptable.

The licensee has also proposed TS to sample and analyze plant effluents, NUREG-0737, Item II.F.1.2. The proposed TS are consistent with the guidance of the model STS and are, therefore, acceptable.

2. TS.4.10 - Radiation Environmental Monitoring Program changing the maximum lower limit of detection (LLD) for Cs 134 and 137 (Table TS.4.10-2 Page 1 of 2)

The licensee proposed a change to Table TS.4.10-2 dealing with the maximum lower limit of detection (LLD) for Cesium 134 and 137 in analyzing for the presence of these nuclides in food products. Amendment Nos. 59 and 53 issued by letter dated October 21, 1982 contained an error regarding the maximum LLD requirement for the radioanalysis of Cesium 134 and 137. The change would lower the maximum LLD from the existing 80 to 60 PCi/Kg wet for Cs 134 and Cs 137 which would agree with the licensee's existing plant procedures. The proposed change would be more restrictive than the existing requirement, thus increasing the level of plant safety. On this basis, the proposed change to Table TS 4.10-2 is acceptable.

3. TS.4.10 - Radiation Environmental Program Figures TS.4.10-1 and TS.4.10-2

The licensee proposed to correct an error related to Amendment Nos. 59 and 53 issued by our letter dated October 21, 1982 which consisted of deleting Figure TS.4.10-1 and TS.4.10-2. This is considered an administrative error in that the deletion of Figures TS.4.10-1 and TS.4.10-2 were to be part of Amendment Nos. 59 and 53 but were inadvertently omitted. Figures TS.4.10-1 and TS.4.10-2 show the locations of the radiation environmental monitors in the surrounding area of the plant site. The locations of these radiation environmental monitors are identified in greater detail in Table 4.10-1. Therefore, Figures TS.4.10-1 and TS.4.10-2 are no longer needed and serve no useful purpose in the TS. On this basis, the staff finds the proposed change of deleting these figures to be acceptable.

4. TS.6-1 - Administrative Controls Plant Site Organization

The licensee proposed to change the title from "Senior Nuclear Engineering" to "Superintendent Nuclear Engineering" in the title block of TS Figure TS.6.1-2. The title change would make the title appearing in the TS figure agree with the title existing in the actual plant organization. This change is administrative in nature and has no effect on the management function, the

authority or the responsibility of the position identified in the figure. On this basis, the staff concludes that the change will not reduce the level of the plant safety and therefore is acceptable.

5. TS.3.5 - Instrument Operating Conditions for Isolation Functions (Table TS.3.5-4)

The licensee proposes to modify the action statement for the containment ventilation isolation system when a certain minimum number of operable channels cannot be met during the refueling outages. The proposed change would affect the wording appearing in the action column for Item 2, Containment Ventilation Isolation, in Table TS.3.5.4. The present wording requires that the inservice containment purge and inservice purge valves be closed when one train of the containment ventilation isolation is taken out of service for maintenance. The maintenance of this ventilation system is normally performed during a refueling outage. The proposed change clarifies the requirement so that the main purge and inservice purge valve would be closed only during the fuel handling phase of the refueling outage if the minimum number of operable channels of the containment ventilation isolation system are not met. It was never the intent of the staff to have the action statement effective during other periods of the refueling outage since the containment atmosphere is susceptible to contamination only during the fuel handling phase of the outage. The staff agrees with the licensee that it is undesirable to close the purge and inservice purge valves because eventually such a condition would lead to a degradation of the containment atmosphere that could be detrimental to operating personnel. The staff considers the proposed change a clarification of the purge and inservice purge valve position during refueling outages which has no effect when the plant is above cold shutdown and therefore does not change the level of plant safety. On this basis, the staff finds the proposed change acceptable.

6. Typographical corrections and misspelled words throughout TSs

The licensee requested change throughout the TS involving typographical correction and misspelled words which was the result of a review of the TS the purpose of which was to uncover such errors. The findings resulting from the review are as follows.

a. Correct typo on page TS-iv line 6.1-1 by changing "On-site to "On-Site".

b. Correct typo in TS 3.1.A.4.c by changing "that" to "than".

c. Correct typo in TS 3.1.A.4.g by changing "blocks" to "block".

d. Correct typo in second paragraph on page TS.3.5-3 by changing "exhause" to "exhaust".

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f. Correct typo in TS 3.14.F.1.e by changing "house" to "House".

g. Correct typo in Table TS.4.2-1 by changing "ASME B & PB" to "ASME B & PV".

h. Correct typo in note at the bottom of Table TS.4.4-1 (pg 4 of 5) by changing "refuleing" to "refueling".

i. Correct typos on page TS.4.5-4 by changing "Deviaitons" to "Deviations", "throughtout" to "throughout", "presure" to "pressure" and "perfformance" to performance".

j. Correct typo in TS.6.5.B.3 by changing "March 31, 1979" to "March 13, 1980".

k. Correct typo in TS.6.7.A.5 by changing "Table 4.16.1" to "Table 4.10-1".

l. Correct typo in TS.6-6 by changing "zirconium reactors" to "zirconium reaction".

The proposed typographical changes b, c, d, e, and i listed above do not appear in the TSs located at our office and therefore these corrections would apply only to the TSs located at the licensee's offices. All other corrections listed above would be made to the TSs located at NRC and the licensee's offices. Proposed change item l above has been added to the list by the staff. The staff has reviewed all of these proposed changes and agrees with the licensee that these changes are merely corrections of misspelled words and typographical errors. These corrections have no effect on the TS requirements nor do they change the intent of the TS. On this basis, the staff finds the proposed changes acceptable.

NUREG-0737 Item III.D.3.4, Control Room Habitability, requires licensees to assure that control room operators are adequately protected against the effects of accidental release of toxic and radioactive gases. By letter of April 9, 1982, the staff found the licensee's response to Item III.D.3.4 in NUREG-0737 acceptable. The staff's acceptance of these requirements was predicated on the proper installation and maintenance of the toxic gaseous detection system that monitors ammonia,formaldehyde and hydrochloric acid. By letter dated July 9, 1984 the licensee informed the staff that a new study for estimating incapacitation times following exposure to toxic gases (i.e., ammonia, formaldehyde and hydrochloric acid) indicates that existing detection systems for these toxic gases are no longer necessary. Consequently, the licensee withdrew the proposed technical specifications change request related to the toxic gas monitoring system (i.e., Item 2 of TS change requested by letter dated April 10, 1984). The staff is presently reviewing this study that was performed by Bechtel Corporation under contract to the licensee. The licensee is requested to maintain the toxic gaseous detection systems in an operable condition until such time that the staff completes the evaluation of the licensee's submittal.

Environmental Consideration

The part of the amendment dealing with correcting errors in the radiation environmental monitoring program technical specifications involves an

administrative change. Accordingly, this part of the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). The remaining parts of the amendments involve a change in the installation or use of a facility component located within the restricted area or a change to a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts of any effluents that may be released and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the remaining parts of the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 12, 1984.

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