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LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p data-bbox="132 289 972 324">3.2.7 <u>REACTOR COOLANT SYSTEM ISOLATION VALVES</u></p> <p data-bbox="254 362 432 397"><u>Applicability:</u></p> <p data-bbox="254 430 936 532">Applies to the operating status of the system of isolation valves on lines connected to the reactor coolant system.</p> <p data-bbox="254 570 399 605"><u>Objective:</u></p> <p data-bbox="254 638 1005 776">To assure the capability of the reactor coolant system isolation valves to minimize reactor coolant loss in the event of a rupture of a line connected to the nuclear steam supply system.</p> <p data-bbox="254 813 447 849"><u>Specification:</u></p> <ol data-bbox="254 881 999 1300" style="list-style-type: none"> <li data-bbox="254 881 999 1057">a. During power operating conditions whenever the reactor head is on, all reactor coolant system isolation valves on lines connected to the reactor coolant system shall be operable except as specified in "b" below. <li data-bbox="254 1089 999 1300">b. In the event any isolation valve becomes inoperable the system shall be considered operable provided at least one valve in each line having an inoperable valve is in the mode corresponding to the isolated condition, except as noted in Specification 3.1.1.e. 	<p data-bbox="1106 280 1946 316">4.2.7 <u>REACTOR COOLANT SYSTEM ISOLATION VALVES</u></p> <p data-bbox="1232 354 1411 389"><u>Applicability:</u></p> <p data-bbox="1232 422 1921 488">Applies to the periodic testing requirement for the reactor coolant system isolation valves.</p> <p data-bbox="1232 561 1373 597"><u>Objective:</u></p> <p data-bbox="1232 630 1980 768">To assure the capability of the reactor coolant system isolation valves to minimize reactor coolant loss in the event of a rupture of a line connected to the nuclear steam supply system.</p> <p data-bbox="1232 805 1421 841"><u>Specification:</u></p> <p data-bbox="1232 873 1934 940">The reactor coolant system isolation valves surveillance shall be performed as indicated below.</p> <ol data-bbox="1232 977 1976 1219" style="list-style-type: none"> <li data-bbox="1232 977 1976 1115">a. <u>At least once per operating cycle</u> the operable automatically initiated power-operated isolation valves shall be tested for automatic initiation and closure times. <li data-bbox="1232 1148 1976 1219">b. Additional surveillances shall be performed as required by Specification 6.17.

6.9.3 Special Reports

Special reports shall be submitted in accordance with 10 CFR 50.4 Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Reactor Vessel Material Surveillance Specimen Examination, Specification 4.2.2(b) (12 months).
- b. (Deleted)
- c. (Deleted)
- d. (Deleted)
- e. (Deleted)
- f. (Deleted)
- g. Sealed Source Leakage In Excess Of Limits, Specification 3.6.5.2 (Three months).
- h. Calculate Dose from Liquid Effluent in Excess of Limits, Specification 3.6.15.a(2)(b) (30 days from the end of the affected calendar quarter).
- i. Calculate Air Dose from Noble Gases Effluent in Excess of Limits, Specification 3.6.15.b(2)(b) (30 days from the end of the affected calendar quarter).
- j. Calculate Dose from I-131, H-3 and Radioactive Particulates with half lives greater than eight days in Excess of Limits, Specification 3.6.15.b(3)(b) (30 days from the end of the affected calendar quarter).
- k. Calculated Doses from Uranium Fuel Cycle Source in Excess of Limits, Specification 3.6.15.d (30 days from the end of the affected calendar year).
- l. Inoperable Gaseous Radwaste Treatment System, Specification 3.6.16.b (30 days from the event).
- m. Environmental Radiological Reports. With the level of radioactivity (as the result of plant effluents) in an environmental sampling medium exceeding the reporting level of Table 6.9.3-1, when averaged over any calendar quarter, in lieu of a Licensee Event Report, prepare and submit to the Commission within thirty (30) days from the end of the calendar quarter a special report identifying the cause(s) for exceeding the limits, and define the corrective action to be taken.

4. The combined Local Leak Rate Test (Type B & C Tests including airlocks) acceptance criteria is less than $0.6 L_a$, calculated on a minimum pathway basis, at all times when containment integrity is required.

The provisions of Specification 4.0.1 do not apply to the test frequencies specified in the 10 CFR 50 Appendix J Testing Program Plan.

6.17 Inservice Testing Program

This program provides controls for inservice testing of Quality Group A, B, and C pumps and valves.

- a. Inservice testing of Quality Group A, B, and C pumps and valves shall be performed in accordance with requirements for American Society of Mechanical Engineers (ASME) Code Class 1, 2, and 3 components specified in Section XI of the applicable ASME Boiler and Pressure Vessel Code Edition and Addenda, subject to the applicable provisions of 10CFR50.55a;
- b. The provisions of Specification 4.0.1 are applicable to the normal and accelerated testing frequencies for performing inservice testing activities;
- c. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.