<b>JAN 2 &amp; 1982</b>	DISTRIBUTION: Docket File NRC PDR L PDR NSIC	
Docket Nos. 50-282 and 50-306	TERA ORB#3 Rdg DEisenhut DDiIanni PMKreutzer-3	
Mr. L. O. Mayer, Manager Nuclear Support Services Northern States Power Company 414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401	RAClark OELD I&E-4 GDeegan-8 DBrinkman ACRS-10	
Dear Mr. Mayer:	Gray File	

On December 30, 1981, the Commission issued Amendment Nos. 53 and 47 to Facility Operating Licenses Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2. On December 4, 1981, Amendment Nos. 52 and 46 were issued. Page 3-4.2 was affected by both amendments and inadvertently the changes made by Amendments 52 and 46 were not carried forward to the new amendments 53 and 47.

Enclosed is a corrected page 3.4-2. Please accept our apologies for any inconvenience this administrative error may have caused.

Sincerely, Original signed by Robert A. Clark

Dominic C. Dilanni, Project Manager Operating Reactors Branch #3 Division of Licensing

Enclosure: TS page 3.4-2

cc: See next page

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NRC FORM 318	(10-80) NRCM 0240		OFFICIAL	RECORD C	OPY	<u></u>	USGPO: 1981-335-960

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## Northern States Power Company

# cc:

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U. S. Environmental Protection Agency Federak Activities Branch Region V Office ATTN: Regional Radiation Representative 230 South Dearborn Street Chicago, Illinois 60604

- f. Essential features including system piping, valves, and interlocks directly associated with the above components are operable.
- g. Manual valves in the above systems that could (if one is improperly positioned) reduce flow below that assumed for accident analysis shall be locked in the proper position for emergency use. During power operation, changes in valve position will be under direct administrative control.

### 3. Steam Exclusion System

Both isolation dampers in each ventilation duct that penetrates rooms containing equipment required for a high energy line rupture outside of containment shall be operable or one damper in each duct with an inoperable isolation damper shall be closed.

#### 4. Radiochemistry

The specific activity of the secondary coolant system for that reactor shall be  $\leq 0.10$  uCi/gm DOSE EQUIVALENT I-131.

B. If, during startup operation or power operation, any of the conditions of Specification 3.4.A., except as noted below for 2.a, 2.b or 4 cannot be met, startup operations shall be discontinued and if operability cannot be restored within 48 hours, the affected reactor shall be placed in the cold shutdown condition using normal operating procedures.

With regard to Specifications 2.a or 2.b, if a turbine driven AFW pump is not operable, that AFW pump shall be restored to operable status within 72 hours or the affected reactor shall be cooled to less than 350°F within the next 12 hours. If a motor driven AFW pump is not operable, that AFW pump shall be restored to operable status within 72 hours or one unit shall be cooled to less than 350°F within the next 12 hours.

If 4. is not met, the affected reactor shall be placed in hot standby within 6 hours and cold shutdown within the following 30 hours.

#### Basis

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A reactor shutdown from power requires removal of decay heat. Decay heat removal requirements are normally satisfied by the steam bypass to the condenser and by continued feedwater flow to the steam generators. Normal feedwater flow to the steam generators is provided by operation of the turbine-cycle feedwater system.

Prairie Island Unit 1 - Amendment No. 17, 46, 52, 53 Prairie Island Unit 2 - Amendment No. 11, 40, 46, 47