

DEC 4 1981

Docket Nos. 50-282
and 50-306

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Mr. L. O. Mayer, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Dear Mr. Mayer:

The Commission has issued the enclosed Amendments Nos. 52 and 46 to Facility Operating Licenses Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, respectively, in response to your application dated November 12, 1979.

The amendments revise the common Technical Specifications for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2 to limit conditions for operation and establish surveillance requirements of the reactor coolant system and secondary coolant activities.

Copies of the related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

Dominic C. DiIanni, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 52 to DPR-42
2. Amendment No. 46 to DPR-60
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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No legal objection to form of amendment M.B.

OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:OR:DL	OELD	
SURNAME	PMKreutzer	DDianni/pn	RAClark	TMNovak	M BLUME	WAYNE Houston
DATE	11/5/81	11/9/81	11/9/81	11/11/81	11/13/81	11/16/81

Northern States Power Company

cc:

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Representative
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Chicago, Illinois 60604



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated November 12, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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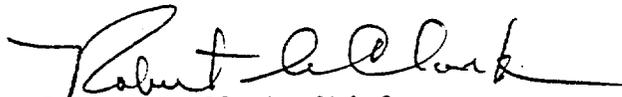
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 4, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated November 12, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 46, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 4, 1981

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-42

AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment Number and contain vertical lines indicating the area of change.

Remove Pages

TS-iv
TS.3.1-11
TS.3.1-12
TS.3.1-13
Figure TS.3.1-5
TS.3.4-2
TS.3.4-3
Table TS.4.1-2B

Insert Pages

TS-iv
TS.3.1-11
TS.3.1-12
TS.3.1-13
Figure TS.3.1-5
TS.3.4-2
TS.3.4-3
Table TS.4.1-2B
pages 1 and 2

APPENDIX A TECHNICAL SPECIFICATIONSLIST OF FIGURES

<u>TS FIGURE</u>	<u>TITLE</u>
2.1-1	Safety Limits, Reactor Core, Thermal and Hydraulic Two Loop Operation
3.1-1	Unit 1 and Unit 2 Reactor Coolant System Heatup Limitations
3.1-2	Unit 1 and Unit 2 Reactor Coolant System Cooldown Limitations
3.1-3	Effect of Fluence and Copper Content on Shift of RT_{NDT} for Reactor Vessel Steels Exposed to 550° Temperature
3.1-4	Fast Neutron Fluence ($E > 1$ MeV) as a Function of Full Power Service Life
3.1-5	DOSE EQUIVALENT I-131 Primary Coolant Specific Activity Limit Versus Percent of RATED THERMAL POWER with Primary Coolant Specific Activity > 1.0 μ Ci/gram DOSE EQUIVALENT I-131
3.10-1	Required Shutdown Reactivity Vs Reactor Boron Concentration
3.10-2	Control Bank Insertion Limits
3.10-3	Insertion Limits 100 Step Overlap with One Bottomed Rod
3.10-4	Insertion Limits 100 Step Overlap with One Inoperable Rod
3.10-5	Hot Channel Factor Normalized Operating Envelope for $F_0 = 2.21$
3.10-6	Deviation from Target Flux Difference as a Function of $Q_{Thermal}$ Power
3.10-7	Normalized Exposure Dependent Function $BU(E_j)$ for Exxon Nuclear Company Fuel
3.10-8	$V(Z)$ as a function of core height
4.4-1	Shield Building Design In-Leakage Rate
4.10-1	Prairie Island Nuclear Generating Plant Radiation Environmental Monitoring Program (Sample Location Map)
4.10-2	Prairie Island Nuclear Generating Plant Radiation Environmental Monitoring Program (Sample Location Map)
6.1-1	NSP Corporate Organizational Relationship to On-site Operating Organization
6.1-2	Prairie Island Nuclear Generating Plant Functional Organization for On-site Operating Group

Prairie Island Unit 1 - Amendment No. 33, 44, 52
 Prairie Island Unit 2 - Amendment No. 29, 38, 46

D. MAXIMUM COOLANT ACTIVITY

Specification:

1. The specific activity of the primary coolant shall be limited to:
 - (a) Less than or equal to 1.0 microcuries per gram DOSE EQUIVALENT I-131, and
 - (b) Less than or equal to $100/\bar{E}$ microcuries per gram.
2. In Specification 3.1.D.1 the following definitions apply:
 - (a) DOSE EQUIVALENT I-131 is that concentration of I-131 (uCi/gram) which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in Table III of TID-14844, "Calculation of Distance Factors for Power and Test Reactor Sites."
 - (b) \bar{E} shall be the average (weighted in proportion to the concentration of each radionuclide in the reactor coolant at the time of sampling) of the sum of the average beta and gamma energies per disintegration (in MeV) for isotopes, other than iodines, with half lives greater than 15 minutes, making up at least 95% of the total non-iodine activity in the coolant.
3. If a reactor is above hot shutdown and RCS temperature is greater than or equal to 500°F:
 - (a) With the specific activity of the primary coolant greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 but within the allowable limit (below and to the left of the line) shown on Figure TS.3.1-5, operation may continue for up to 48 hours provided that the cumulative operating time under these circumstances does not exceed 800 hours in any consecutive 12-month period. With the total cumulative operating time at a primary coolant specific activity greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 exceeding 500 hours in any consecutive 6-month period, a special report to the Commission shall be submitted within 30 days indicating the number of hours above this limit.
 - (b) With the specific activity of the primary coolant greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 for more than 48 hours during one continuous time interval or exceeding the limit line shown on Figure TS.3.1-5, the affected reactor shall be shutdown and RCS temperature cooled to 500°F or less within 6 hours.
 - (c) With the specific activity of the primary coolant greater than $100/\bar{E}$ microcurie per gram, the affected reactor shall be shutdown and RCS temperature cooled to 500°F or less within 6 hours of detection.

4. If a reactor is at or above cold shutdown:

- (a) With the specific activity of the primary coolant greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 or greater than $100/\bar{E}$ microcuries per gram, perform the sampling and analysis requirements of item 4a of Table 4.1-2B until the specific activity of the primary coolant is restored to within its limits. A reportable occurrence report shall be submitted to the Commission within 30 days. This report shall contain the results of the specific activity analyses together with the following information:
1. Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded,
 2. Fuel burnup by core region,
 3. Clean-up flow history starting 48 hours prior to the first sample in which the limit was exceeded,
 4. History of de-gassing operations, if any, starting 48 hours prior to the first sample in which the limit was exceeded, and
 5. The time duration when the specific activity of the primary coolant exceeded 1.0 microcurie per gram DOSE EQUIVALENT I-131.

Basis

The limitations on the specific activity of the primary coolant ensure that the resulting 2 hour doses at the site boundary will not exceed an appropriately small fraction of Part 100 limits following a steam generator tube rupture accident in conjunction with an assumed steady state primary-to-secondary steam generator leakage rate of 1.0 GPM. The values for the limits on specific activity represent limits based upon a parametric evaluation by the NRC of typical site locations. These values are conservative in that specific site parameters of the Prairie Island site, such as site boundary location and meteorological conditions, were not considered in this evaluation.

Specification 3.1.D.2, permitting power operation to continue for limited time periods with the primary coolant's specific activity greater than 1.0 microcuries/gram DOSE EQUIVALENT I-131, but within the allowable limit shown on Figure TS.3.1-5, accommodates possible iodine spiking phenomenon which may occur following changes in thermal power. Operation with specific activity levels exceeding 1.0 microcuries/gram DOSE EQUIVALENT I-131 but within the limits shown on Figure TS.3.1-5 must be restricted to no more than 800 hours per year (approximately 10 percent of the unit's yearly operating time) since the activity levels allowed by

Figure TS.3.1-5 increase the 2 hours thyroid dose at the site boundary by a factor of up to 20 following a postulated steam generator tube rupture. The reporting of cumulative operating time over 500 hours in any 6 month consecutive period with greater than 1.0 microcuries/gram DOSE EQUIVALENT I-131 will allow sufficient time for Commission evaluation of the circumstances prior to reaching the 800 hour limit.

Reducing RCS temperature to less than 500°F prevents the release of activity should a steam generator tube rupture since the saturation pressure of the primary coolant is below the lift pressure of the atmospheric steam relief valves. The surveillance requirements in Table TS.4.1-2B provide adequate assurance that excessive specific activity levels in the primary coolant will be detected in sufficient time to take corrective action. Information obtained on iodine spiking will be used to assess the parameters associated with spiking phenomena. A reduction in frequency of isotopic analyses following power changes may be permissible if justified by the data obtained.

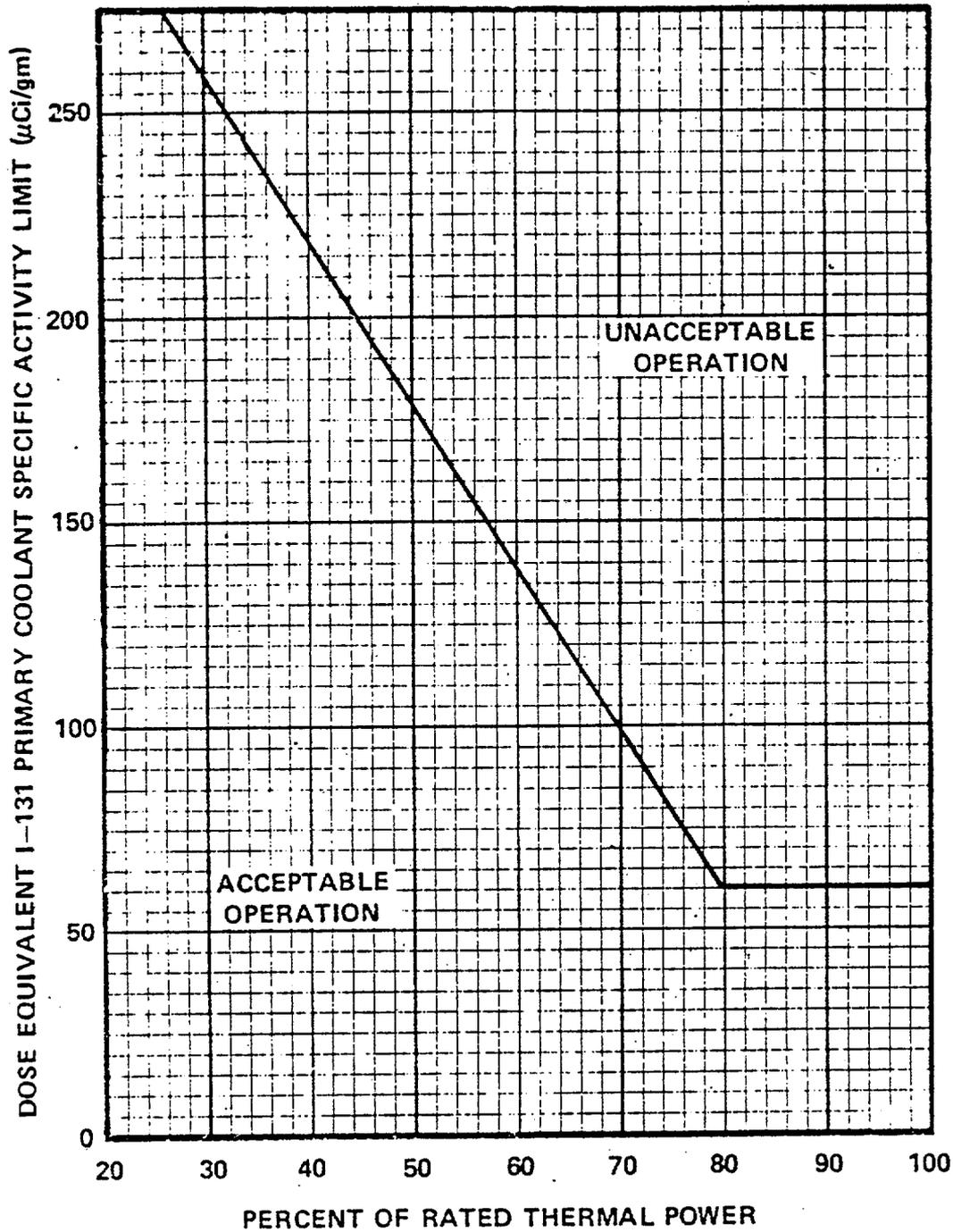


FIGURE 3.1-5

DOSE EQUIVALENT I-131 Primary Coolant Specific Activity Limit Versus Percent of RATED THERMAL POWER with the Primary Coolant Specific Activity > 1.0 µCi/gram Dose Equivalent I-131

Prairie Island Unit 1 - Amendment No. 52
 Prairie Island Unit 2 - Amendment No. 48

- e. For Unit 1 operation motor operated valves MV32242 and MV32243 shall have valve position monitor lights operable and shall be locked in the open position by having the motor control center supply breakers manually locked open. For Unit 2, corresponding valve conditions shall exist.
- f. Essential features including system piping, valves, and interlocks directly associated with the above components are operable.
- g. Manual valves in the above systems that could (if one is improperly positioned) reduce flow below that assumed for accident analysis shall be locked in the proper position for emergency use. During power operation, changes in valve position will be under direct administrative control.

3. Steam Exclusion System

Both isolation dampers in each ventilation duct that penetrates rooms containing equipment required for a high energy line rupture outside of containment shall be operable or at least one damper in each duct shall be closed.

4. Radiochemistry

The specific activity of the secondary coolant system for that reactor shall be ≤ 0.10 uCi/gm DOSE EQUIVALENT I-131.

- B. If, during startup operation or power operation any of the conditions of Specification 3.4.A., except as noted below for 2.a, 2.b or 4 cannot be met, startup operations shall be discontinued and if operability cannot be restored within 48 hours, the affected reactor shall be placed in the cold shutdown condition using normal operating procedures.

With regard to Specifications 2a or 2b, if a turbine driven AFW pump is not operable, that AFW pump shall be restored to operable status within 72 hours or the affected reactor shall be cooled to less than 350°F within the next 12 hours. If a motor driven AFW pump is not operable, that AFW pump shall be restored to operable status within 72 hours or one unit shall be cooled to less than 350°F within the next 12 hours.

If 4. is not met, the affected reactor shall be placed in hot standby within 6 hours and cold shutdown within the following 30 hours.

Basis

A reactor shutdown from power requires removal of decay heat. Decay heat removal requirements are normally satisfied by the steam bypass to the condenser and by continued feedwater flow to the steam generators. Normal feedwater flow to the steam generators is provided by operation of the turbine-cycle feedwater system.

Prairie Island Unit 1 - Amendment No. 17, 46, 52
 Prairie Island Unit 2 - Amendment No. 11, 40, 46

The ten main steam safety valves have a total combined rated capability of 7,745,000 lbs/hr. The total full power steam flow is 7,094,000 lbs/hr; therefore, the ten main steam safety valves will be able to relieve the total steam flow if necessary. (1)

In the unlikely event of complete loss of offsite electrical power to either or both reactors, continued removal of decay heat would be assured by availability of either the steam-driven auxiliary feedwater pump or the motor-driven auxiliary feedwater pump associated with each reactor, and by steam discharge to the atmosphere through the main steam safety valves. One auxiliary feedwater pump can supply sufficient feedwater for removal of decay heat from one reactor. The motor-driven auxiliary feedwater pump for each reactor can be made available to the other reactor.

The minimum amount of water specified for the condensate storage tanks is sufficient to remove the decay heat generated by one reactor in the first 24 hours of shutdown. Essentially unlimited replenishment of the condensate storage supply is available from the intake structures through the cooling water system.

The two power-operated relief valves located upstream of the main steam isolation valves are required to remove decay heat and cool the reactor down following a high energy line rupture outside containment. (2) Isolation dampers are required in ventilation ducts that penetrate those rooms containing equipment needed for the accident.

The limitations on secondary system specific activity ensure that the resultant off-site radiation dose will be limited to a small fraction of 10 CFR Part 100 limits in the event of a steam line rupture. This dose also includes the effects of a coincident 1.0 GPM primary to secondary tube leak in the steam generator of the affected steam line. These values are consistent with the assumptions used in the accident analyses.

References

- (1) FSAR, Section 10.4
- (2) FSAR, Appendix I

TABLE TS.4.1-2B

MINIMUM FREQUENCIES FOR SAMPLING TESTS

<u>TEST</u>	<u>FREQUENCY</u>	<u>FSAR Section Reference</u>
1. RCS Gross Activity Determination	5/week	
2. RCS Isotopic Analysis for DOSE EQUIVALENT I-131 Concentration	1/14 days (when at power)	
3. RCS Radiochemistry \bar{E} determination	1/6 months(1) (when at power)	
4. RCS Isotopic Analysis for Iodine Including I-131, I-133, and I-135	a) Once per 4 hours, whenever the specific activity exceeds 1.0 uCi/gram DOSE EQUIVALENT I-131 or 100/ \bar{E} uCi/gram (at or above cold shutdown), and b) One sample between 2 and 6 hours following a THERMAL POWER change exceeding 15 percent of the RATED THERMAL POWER within a one hour period (above hot shutdown)	
5. RCS Radiochemistry (2)	Monthly	
6. RCS Tritium Activity	Weekly	
7. RCS Chemistry (Cl*, F*, O ₂)	5/Week	
8. RCS Boron Concentration*(3)	2/Week (4)	9.2
9. RWST Boron Concentration	Weekly	
10. Boric Acid Tanks Boron Concentration	2/Week	
11. Caustic Standpipe NaOH Concentration	Monthly	6.4
12. Accumulator Boron Concentration	Monthly	6
13. Spent Fuel Pit Boron Concentration	Monthly	9.5.5

Prairie Island Unit 1 - Amendment No. 25, 52
Prairie Island Unit 2 - Amendment No. 19, 46

TABLE TS.4.1-2B

MINIMUM FREQUENCIES FOR SAMPLING TESTS

<u>TEST</u>	<u>FREQUENCY</u>	<u>FSAR Section Reference</u>
14. Secondary Coolant Gross Beta-Gamma activity	Weekly	
15. Secondary Coolant Isotopic Analysis for DOSE EQUIVALENT I-131 concentration	1/6 months (5)	
16. Secondary Coolant Chemistry		
pH	5/week (6)	
Ammonia	5/week (6)	
Sodium	5/week (6)	

NOTES:

1. Sample to be taken after a minimum of 2 EFPD and 20 days of power operation have elapsed since reactor was last subcritical for 48 hours or longer.
 2. To determine activity of corrosion products having a half-life greater than 30 minutes.
 3. See Specification 3.8 for requirements during refueling.
 4. The maximum interval between analyses shall not exceed 5 days.
 5. If activity of the samples is greater than 10% of the limit in Specification 3.4.A.4, the frequency shall be once per month.
 6. The maximum interval between analyses shall not exceed 3 days.
- * See Specification 4.1.D.

Prairie Island Unit 1 - Amendment No. 23, 31, 52
Prairie Island Unit 2 - Amendment No. 19, 43, 46



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-42
AND AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NO. DPR-60
NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-282 AND 50-306

INTRODUCTION

By letter dated November 12, 1979 Northern States Power Company (the licensee) requested a change to the Technical Specifications (TS) of Appendix A of operating license Nos. DPR-42 and DPR-60. The proposed changes which were initiated at our request, would limit conditions for operation and establish surveillance requirements of the reactor coolant and the secondary coolant activities.

DISCUSSION AND EVALUATION

The staff requested that the licensee propose changes to Appendix A TS that would modify the RCS activity specification in order to comply with requirements set forth in the standardized Technical Specifications. In addition these proposed changes would impose new requirements consistent with NRC regulatory guidance ensuring that the resulting two hour dose at the site boundary will not exceed an appropriately small fraction of 10 CFR Part 100 limits following a steam generator tube rupture accident.

These limits imposed by the new requirements on [13] at the site boundary would be reduced by approximately a factor of 5 below those now existing in Appendix A which are already considerably below the 10 CFR Part 100 limits. By letter dated November 12, 1979 the licensee submitted proposed changes limiting conditions for operation and imposing surveillance requirements of the reactor coolant and secondary coolant activity. Based on our review, we concluded that the licensee's initial submittal conformed with all of the requirements of the standardized Technical Specifications except for monitoring activities at cold shutdown and during thermal power changes. The licensee's submittal was modified to include requirements acceptable to the staff. These modifications were discussed with and agreed to by the licensee. The licensee's proposed change as modified will further ensure that the reactor coolant and secondary coolant activities will be maintained at low concentrations so that in the event of a steam generator tube rupture or a steam line rupture the off-site doses will be a small fraction of the exposure guidelines of 10 CFR Part 100. On this basis we find the proposed changes to the TS as modified are acceptable.

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PDR

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 4, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKET NOS. 50-282 AND 50-306
NORTHERN STATES POWER COMPANY
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 52 and 46 to Facility Operating License Nos. DPR-42 and DPR-60 issued to Northern States Power Company (the licensee), which revised Technical Specifications for operation of Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2 (the facilities) located in Goodhue County, Minnesota. The amendments are effective as of the date of issuance.

The amendments revise the common Technical Specifications for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2 to limit conditions for operation and establish surveillance requirements of the reactor coolant system and secondary coolant activities.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

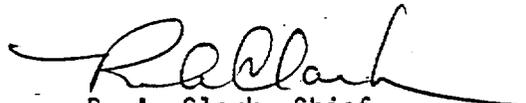
- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated November 12, 1981, (2) Amendment Nos. 52 and 46 to License Nos. DPR-42 and DPR-60, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental Conservation Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 4th day of December, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


R. A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DISTIRBUTION:
Docket File
ORB#3 Rdg
PMKreutzer

Docket No. 50-282/50-306

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: NORTHERN STATES POWER COMPANY, Prairie Island Nuclear Generating
Plant, Unit Nos. 1 and 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment Nos. 52 and 46
Referenced documents have been provided.PDR.

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#3:DL				
SURNAME →	PMKreutzer/pm				
DATE →	12/7/81				