

Docket Nos. 50-282
and 50-306

AUGUST 2 1979

Mr. L. G. Mayer, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Hall - 8th Floor
Minneapolis, Minnesota 55401

Distribution

Docket Files 50-282 I&E (5)
and 50-306

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Dear Mr. Mayer:

The Commission has issued the enclosed Amendment No. 37 to Facility Operating License No. DPR-42 and Amendment No. 31 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated January 10, 1978, as supplemented March 15, 1979.

These amendments change the surveillance frequencies of the nuclear instrumentation system low power level reactor trip functions in both units. In addition, an administrative error in Amendments 28 and 32 dated March 28, 1978 is corrected.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 37 to DPR-42
2. Amendment No. 31 to DPR-60
3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 2, 1979

Docket Nos. 50-282
and 50-306

Mr. L. O. Mayer, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Dear Mr. Mayer:

The Commission has issued the enclosed Amendment No. 37 to Facility Operating License No. DPR-42 and Amendment No. 31 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated January 10, 1978, as supplemented March 15, 1979.

These amendments change the surveillance frequencies of the nuclear instrumentation system low power level reactor trip functions in both units. In addition, an administrative error in Amendments 28 and 32 dated March 28, 1978 is corrected.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer", is written over the typed name.

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1. Amendment No. 37 to DPR-42
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3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

Mr. L. O. Mayer
Northern States Power Company

- 2 -

August 2, 1979

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Crystal Mall #2
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U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
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230 South Dearborn Street
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Attorney General
102 State Capitol
St. Paul, Minnesota 55155



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 10, 1978 supplemented March 15, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 2, 1979



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated January 10, 1978 supplemented March 15, 1979, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 31, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 2, 1979

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-42

AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NOS. 50-282 AND 50-306

Revise Appendix A as follows:

Remove Pages

TS 2.3-1

Table TS 4.1-1 (page 1 of 4)
Table TS 4.1-1 (page 2 of 4)
Table TS 4.1-1 (page 3 of 4)
Table TS 4.1-1 (page 4 of 4)

Insert Pages

TS 2.3-1

Table TS 4.1-1 (page 1 of 5)
Table TS 4.1-1 (page 2 of 5)
Table TS 4.1-1 (page 3 of 5)
Table TS 4.1-1 (page 4 of 5)
Table TS 4.1-1 (page 5 of 5)

2.3 LIMITING SAFETY SYSTEM SETTINGS, PROTECTIVE INSTRUMENTATION

Applicability

Applies to trip settings for instruments monitoring reactor power and reactor coolant pressure, temperature, flow, and pressurizer level.

Objective

To provide for automatic protective action in the event that the principal process variables approach a safety limit.

Specification

A. Protective instrumentation settings for reactor trip shall be as follows:

1. Startup protection

- a. High flux, intermediate range (high set point) - current equivalent to $\leq 40\%$ of full power.
- b. High flux, power range (low set point) - $\leq 25\%$ of rated power.
- c. High flux, source range - neutron flux $\leq 10^6$ counts/second

2. Core protection

- a. High flux, power range (high set point) - $\leq 108\%$ of rated power.
- b. High pressurizer pressure - ≤ 2385 psig.
- c. Low pressurizer pressure - ≥ 1815 psig.
- d. Overtemperature ΔT

$$\Delta T_t \leq \Delta T_o \left[K_1 - K_2 (T - T') \left(\frac{1 + \tau_1 s}{1 + \tau_2 s} \right) + K_3 (P - P') - f(\Delta I) \right]$$

where

ΔT_o	=	Indicated ΔT at rated power
T_o	=	Average temperature, $^{\circ}F$
T'	=	$567.3^{\circ}F$
P	=	Pressurizer pressure, psig
P'	=	psig 2235
K_1	\leq	1.11
K_2	=	0.0090
K_3	=	0.000566
τ_1	=	30 sec.
τ_2	=	4 sec.

TABLE TS.4.1-1
(Page 1 of 5)
MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND
TEST OF INSTRUMENT CHANNELS

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Functional Response</u>		<u>Remarks</u>
			<u>Test</u>	<u>Test</u>	
1. Nuclear Power Range	S(1) M(1)	D(2) Q(4)	M(3) M(5) M(6) P(7)	R	1) Once/shift when in service 2) Heat balance 3) Signal to ΔT : bistable action (permissive, rod stop, trips), with the exception of the items covered in Remark #7. 4) Upper and lower chambers for axial off-set using in-core detectors 5) Simulated signal for testing positive and negative rate bistable action 6) Quadrant Power Tilt Monitor 7) P8 and P10 permissives and the 25% High Flux Low Setpoint Trip.
2. Nuclear Intermediate Range	*S(1)	NA	T(2)	R	1) Once/shift when in service 2) Log level; bistable action (permissive, rod stop, trips)
3. Nuclear Source Range	*S(1)	NA	T(2)	R	1) Once/shift when in service 2) Bistable action (alarm, trips)
4. Reactor Coolant Temperature	*S(1,2)	R(1,2,3)	M(1) M(2) T(3)	R(1) R(2)	1) Overtemperature ΔT 2) Overpower ΔT 3) Control Rod Bank Insertion Limit Monitor
5. Reactor Coolant Flow	S	R	M	NA	
6. Pressurizer Water Level	S	R	M	NA	
7. Pressurizer Pressure	S	R	M	NA	

TABLE TS.4.1-1 (page 1 of 5)

TABLE TS.4.1-1
(Page 2 of 5)
MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND
TEST OF INSTRUMENT CHANNELS

Channel Description	Check	Calibrate	Functional	Response	Remarks
			Test	Test	
8. 4KV Voltage & Frequency	NA	R	M	NA	Reactor protection circuits only
8a. RCP Breakers	NA	R	T	NA	
9. Analog Rod Position	S(1) M(2)	R	T(2)	NA	1) With step counters 2) Rod Position Deviation Monitor Tested by updating computer bank count and comparing with analog rod position test signal.

TABLE TS.4.1-1 (page 2 of 5)

TABLE TS.4.1-1

(Page 3 of 5)

Channel Description	Check	Functional Response			Remarks
		Calibrate	Test	Test	
10. Rod Position Bank Counters	S(1,2) M(3)	NA	T(3)	NA	1) With analog rod position 2) Following rod motion in excess of six inches when the computer is out of service 3) Control rod banks insertion limit monitor and control rod position deviation monitors.
11. Steam Generator Level	S	R	M	NA	
12. Steam Generator Flow Mismatch	S	R	M	NA	
13. Charging Flow	S	R	NA	NA	
14. Residual Heat Removal Pump Flow	S(1)	R	NA	NA	1) When in operation
15. Boric Acid Tank Level	D	R(1)	M(1)	NA	1) Transfer logic to Refueling Water Storage Tank
16. Refueling Water Storage Tank Level	W	R	M(1)	NA	1) Functional test can be performed by bleeding transmitter.
17. Volume Control Tank Level	S	R	NA	NA	
18a. Containment Pressure SI Signal	S	R	M(1)	NA	Wide Range Containment Pressure 1) Isolation Valve Signal
18b. Containment Pressure Steam Line Isolation	S	R	M	NA	Narrow Range Containment Pressure
18c. Containment Pressure Containment Spray	S	R	M	NA	
18d. Annulus Pressure (Vacuum Breaker)	NA	R	R	NA	

TABLE TS.4.1-1

(Page 4 of 5)

	<u>Channel Description</u>	<u>Check</u>	<u>Functional Response</u>			<u>Remarks</u>
			<u>Calibrate</u>	<u>Test</u>	<u>Test</u>	
*19.	Radiation Monitoring	*D	R	M	NA	Includes all channels used for leak detection per Spec. 3.1 D. and effluent release monitoring per Spec. 3.9 and 5.5.
20.	Boric Acid Make-up Flow Channel	NA	R	NA	NA	
21.	Containment Sump Level	NA	R	R	NA	Includes Sumps A, B, and C
22.	Accumulator Level and Pressure	S	R	R	NA	
23.	Steam Generator Pressure	S	R	M	NA	
24.	Turbine First Stage Pressure	S	R	M	NA	
25.	Emergency Plan Radiation Instruments	*M	R	M	NA	Includes those named in the emergency procedure (referenced in Spec. 6.5 A.6.)
26.	Protection Systems Logic Channel Testing	NA	NA	M	NA	Includes auto load sequencers
27.	Turbine Overspeed Protection Trip Channel	NA	R	M	NA	
28.	Containment Shell Temperature	M	NA	NA	NA	Includes those used per Spec. 3.6 D.
29.	Containment Air Temperature	M	NA	NA	NA	Includes those used per Spec. 3.6 C.
30.	Environmental Monitors	M	NA	NA	NA	Includes those used per Spec. 4.10
31.	Seismic Monitors	R	R	NA	NA	Includes those reported in Item 4 of Table TS.6.7-1
32.	Coolant Flow - RTD Bypass Flowmeter	S	R	M	NA	
33.	CRDM Cooling Shroud Exhaust Air Temperature	S	NA	R	NA	FSAR page 3.2-56
34.	Reactor Gap Exhaust Air Temperature	S	NA	R	NA	FSAR page 5.4-2

Amendment No. 37, Unit 1
Amendment No. 31, Unit 2

TABLE TS.4.1-1

(Page 5 of 5)

	<u>Channel Description</u>	<u>Check</u>	<u>Functional</u>		<u>Response</u>	<u>Remarks</u>
			<u>Calibrate</u>	<u>Test</u>	<u>Test</u>	
35.	Post-Accident Monitoring Instruments	M	NA	NA	NA	Includes all those in FSAR Table 7.7-2 that are not itemized in Table TS.4.1-1.
36.	Steam Exclusion Actuation System	W	R	M		See FSAR Appendix I, Section I.14.6

S - Each Shift
 D - Daily
 W - Weekly
 M - Monthly
 Q - Quarterly
 R - Each refueling shutdown
 P - Prior to each startup if not done previous week
 T - Prior to each startup following shutdown in excess of 2 days if not done the previous 30 days.
 NA - Not applicable
 * - See Spec. 4.1.D.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-42
AND AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-60
NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-282 AND 50-306

Introduction

By letter dated January 10, 1978¹ supplemented on March 15, 1979³, Northern States Power Company (the licensee) submitted a proposed amendment to the Technical Specifications for Prairie Island Nuclear Generating Plant, Units 1 and 2. The amendment changes the surveillance frequencies of the nuclear instrumentation system low power level reactor trip functions. The licensee has provided an analysis for our review indicating that the proposed change would not degrade safe operation of the plant.

In addition, we have corrected T' to read 567.3°F in 2.3.A.2.d on page TS 2.3-1. In amendment requests dated November 4, 1976 and January 4, 1977 T' was proposed by the licensee to be 566.1°F. A January 28, 1977 licensee withdrew that request and requested that T' remain at 567.3°F. T' was inadvertently changed to 566.1°F in Amendments 28 and 22 issued March 28, 1978. Since the plant normally operates at around 560°F, that error was of no safety significance. It is appropriate at this time to revert to the original value of 567.3°F in the Technical Specifications.

The effect of the requested change in surveillance frequencies would be to require functional testing of the nuclear source and intermediate range power level monitoring instrumentation channels prior to each startup following a shutdown which lasts longer than two days, provided the functional testing has not been performed in the previous 30 days. The present specification requires the functional test prior to each startup if not done the previous week.

Background

Functional testing of a channel consists of electrical simulation of a signal as close as practicable to the channel sensor. The level of the simulated signal is varied such that the operability and setpoints of the various bistables are checked. Thus, "functional testing" should not be confused with "calibration." Functional testing is a check of the electronic analog and logic sections of the channel.

Source Range Channels

The source range channels are used by the operator to monitor neutron flux levels and rates of increase during reactor startup. The two channels also operate an alarm and a trip in a one-out-of-two logic. The analyses of the startup accidents do not take credit for this trip, but instead conservatively assume failure of both source range channels, both intermediate range channels, and two of the four power range channels. With such high redundancy, an extended functional test interval would be acceptable if trip failure were the only concern. However, the functional test also includes a checkout of the analog circuitry, which in addition to outright failure, is subject to drift.

The Technical Specifications require that the source range neutron flux reactor trip setpoint be $\leq 10^6$ counts per second (CPS). This is the limit of the range of allowable values: if the setpoint were discovered above this value, it would constitute a Technical Specification violation. Unlike the Standard Technical Specifications², the Prairie Island Technical Specifications do not specify a target value for the operator. It is the responsibility of the licensee to specify values in his procedures which have sufficient margin to the 10^6 CPS limit to allow for instrument drift. The licensee stated³ that current practice is to set the trip setpoint at 10^5 CPS. The source range channels read out on a logarithmic scale, so the margin is not as large as it would first appear.

The licensee has stated³ that in the past five years, the RMS deviation observed has been equivalent to 4000 CPS in a six month period, with a maximum deviation equivalent to 5000 CPS. This implies that in an eighteen month fuel cycle, 95% of the time the actual setpoint will be $\leq 1.20 \times 10^5$ CPS, which (on a log scale) is less than 1/3 of the margin to the Technical Specification limit. This is acceptable.

The alarm setpoint drift need not be considered here because the only direct safety function of the alarm is to warn plant personnel of positive reactivity insertions during core alterations. This is a shutdown function which is not addressed by the particular specification under consideration.

Finally, the indicating function to the operator during startup, which is used to calculate periods and other relative rather than absolute observations, should not be impaired by a 20% drift, particularly since the operator has access to both channels. Therefore, we find the longer test interval to be acceptable.

Intermediate Range Channels

In addition to indicating neutron flux levels to the operator during startup, the intermediate range channels feed the P-6 permissive, a rod stop, and a trip through a one-out-of-two logic. The trip is backed up by the source and power range trips, as discussed previously. Once again, an extended functional test interval would be acceptable based only on trip failure considerations. However, the analog circuitry testing must also be considered.

The Technical Specifications require that the intermediate range neutron flux reactor trip setpoint be less than or equal to the equivalent of 40% of rated core thermal power (not detector current). This is in contrast to the source range trip setpoint, which is stated directly in terms of detector output (counts per second). Thus, there are two contributions to intermediate range setpoint drift: deviations in the channel circuitry and changes in the relationship between neutron flux at the detector and core thermal power. The licensee has stated previously that total setpoint drift can be as high as 7% over a fuel cycle⁴. The functional test will check the channel circuitry contribution only. No check of detector-flux-to-power ratio is required by the Technical Specification, although data taken later in the startup sequence can be used to determine whether a violation has taken place. The licensee uses 30% of rated thermal power as a target value, thus allowing 10% margin to the 40% rated thermal power limit in the Technical Specifications.

The licensee has stated³ that the RMS instrument deviation in an average seven and one-half month period has been equivalent to 1.3% power. For an eighteen month refueling cycle, this would correspond to 3.12% power RMS, or 5.13% power for a one-sided 95% probability. This would imply that the 7% maximum total drift would be increased to 8.7%. This is still less than the 10% margin to the Technical Specification limit. Moreover, the sole purpose of this 10% margin is to allow for this drift. Other uncertainties are accounted for elsewhere. Therefore, we find a refueling outage interval to be acceptable for functional tests of the intermediate range trip function.

The intermediate range rod stop is not a Technical Specification requirement. Its sole function is to prevent inadvertent trips during startup, and is not required for safety. Therefore, it will not be considered further.

The P-6 permissive allows manual blocking of the source range trip, and de-energizing of the source range channels to prevent detector burnout only when the reactor neutron flux as indicated by the intermediate range channels is above the P-6 setpoint. (Re-instatement of the source range channels is automatic when the intermediate range signal drops below the setpoint.) The purpose of the P-6 permissive is to ensure that the intermediate range channels are on scale before the source range channels are shut off. The Technical Specifications do not address the P-6 permissive directly, but instead require the source range trip to be active whenever the intermediate range detector signal is less than 10^{-10} amperes. (This assures at least one decade of source range to intermediate range overlap.) Thus, the P-6 permissive is a hard-wired backup to the operator's manual action. Downward drifts of the P-6 setpoint would not necessarily lead to a Technical Specification violation. Because of this, and because it is expected that the instrument drift rate of the P-6 setpoint will be of the same order as the other bistables, we find the longer functional test interval to be acceptable for the P-6 circuitry.

Finally, the indicating functions of the intermediate range channels to the operator during startup could drift up to 8.7% instead of 7% in terms of core thermal power. This is not a sufficient increase to cause a problem with range overlap, nor would it impair the ability of the operator to safely maneuver the reactor. Therefore, the longer functional test interval is acceptable for the indicating functions.

Summary

All safety requirements of the source and intermediate range channels have been examined, and in no case will the longer functional test interval significantly affect the reliability of the channels' required safety functions. Therefore, we find the proposed Technical Specification change to be acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 2, 1979

References

1. Letter, L. O. Mayer (NSP) to Director of Nuclear Reactor Regulation (NRC) dated January 10, 1978, enclosing Request for Amendment to Operating License No. DPR-42 and DPR-60, dated January 10, 1978 by L. J. Wachter (NSP).
2. "Standard Technical Specifications for Westinghouse Pressurized Water Reactors," NUREG-0452, June 15, 1978.
3. Letter, L. O. Mayer (NSP) to Director of Nuclear Reactor Regulation (NRC) dated March 15, 1979.
4. Letter, L. O. Mayer (NSP) to D. K. Davis (NRC) dated August 10, 1977.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-282 AND 50-306

NORTHERN STATES POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 37 to Facility Operating License No. DPR-42, and Amendment No. 31 to Facility Operating License No. DPR-60 issued to Northern States Power Company (the licensee), which revised Technical Specifications for operation of Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2 (the facilities) located in Goodhue County, Minnesota. The amendments are effective as of the date of issuance.

The amendments change the surveillance frequencies of the nuclear instrumentation system low power level reactor trip function.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated January 10, 1978, supplemented March 15, 1979, (2) Amendment Nos. 37 and 31 to License Nos. DPR-42 and DPR-60, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental Conservation Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 2nd day of August, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors