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Docket Nos. 50-282
and 50-306

Northern States Power Company
ATTN: Mr. L. O. Mayer
Manager of Nuclear Support Services
414 Nicollet Mall, 8th Floor
Minneapolis, Minnesota 55401

Gentlemen:

In response to your request dated May 26, 1978, supplemented on July 24, 1978, the Commission has issued the enclosed Amendment Nos. 31 and 25 to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2, respectively.

The amendments consist of changes to the Technical Specifications that relate to the inservice inspection of steam generator tubes. During our review of your proposed amendments we found that certain modifications were necessary to meet our requirements. Your staff has agreed to these modifications and they have been incorporated in these amendments.

A copy of the related Notice of Issuance is enclosed.

Sincerely,

Original Signed by
Mary Zeck for

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. 31 to DPR-42
2. Amendment No. 25 to DPR-60
3. Notice of Issuance

cc: w/enclosure
See next page

Const. 1
61

OFFICE ➤	DOR:ORB1	DOR:ORB1	OELD			
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Docket Nos. 50-282
and 50-306

Northern States Power Company
ATTN: Mr. L. O. Mayer
Manager of Nuclear Support Services
414 Nicollet Mall, 8th Floor
Minneapolis, Minnesota 55401

Gentlemen:

In response to your request dated May 26, 1978, supplemented on July 24, 1978, the Commission has issued the enclosed Amendment Nos. and to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2, respectively.

The amendments consist of changes to the Technical Specifications that relate to the inservice inspection of steam generator tubes that are consistent with Regulatory Guide 1.83, Revision 1, dated July 1975. During our review of your proposed amendments we found that certain modifications were necessary to meet our requirements. Your staff has agreed to these modifications and they have been incorporated in these amendments.

A copy of the related Notice of Issuance is enclosed.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. to DPR-42
2. Amendment No. to DPR-60
3. Notice of Issuance

cc w/encl: See next page

OFFICE➤	DOR:ORB#1	DOR:ORB#1	OELD			
SURNAME➤	MGrotenhuis:lb	ASchwencer				
DATE➤	8/1/78	8/ /78	8/ /78			

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the related Notice of Issuance is enclosed.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. to DPR-42
2. Amendment No. to DPR-60
3. Notice of Issuance

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DATE➤	7/17/78	7/ /78	7/ /78	7/ /78	7/ /78	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-42

AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO., DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NOS. 1 AND 2

DOCKET NOS. 50-282 AND 50-306

Introduction

By letter dated May 26, 1978, supplemented on July 24, 1978, Northern States Power Company (NSP) requested amendments to Facility Operating License Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant Unit Nos. 1 and 2 (PINGP). The proposed amendments would change the Technical Specifications that relate to inservice inspection of steam generator tubes. During our review of the proposed amendments we found that certain modifications were necessary to meet our requirements. These modifications were discussed with the NSP staff and they have agreed to the modifications.

Discussion

The proposed amendments update the PINGP Technical Specifications for inservice inspection of steam generator tubes and put them in essentially the same form and language as the Standard Technical Specifications which have been found acceptable for other PWRs of the PINGP type. The result is an acceptable clarification and simplification of the PINGP Technical Specifications for:

1. Steam generator sample selection for inspection,
2. Steam generator tube sample selection requirement for inspection, (Table 4),
3. Classification of inspection results into categories based on degradation and the percentage of defective tubes,
4. Acceptance criteria for steam generator tubes, and
5. Reporting of results of in-service inspection of steam generator tubes.

These changes in the Technical Specifications have been evaluated by the NRC Staff and have been found to be administrative in nature.

In addition, NSP has proposed that the definition of the plugging limit, which is one of the acceptance criteria, remain at 50%. The NRC Staff's evaluation of this plugging limit follows:

Evaluation

The principal parameter used to determine whether any one steam generator tube is acceptable for continued service is the measured imperfection depth. A tube plugging limit has been established and defined in the Technical Specifications as the imperfection depth beyond which the tube must be removed from service. The plugging limit for the PINGP has been 50% degradation of the nominal tube wall-thickness. In the amendment request NSP has proposed to keep the plugging limit at 50%. This is acceptable because the operating history of the plant supports the continued use of this plugging limit.

The plugging limit is based on (1) the minimum tube wall thickness needed to maintain steam generator tube integrity during the limiting stress loadings associated with a LOCA combined with a Safe Shutdown Earthquake (SSE), and (2) an operational allowance to account for the time interval between inspections. Based on other evaluations made by the NRC staff*, analyses performed by Westinghouse on steam generator tube designs similar to the PINGP tube design, Regulatory Guide 1.121, Bases for Plugging Degraded PWR Steam Generator Tubes, and PINGP operating experience to date**, we conclude that the proposed 50% degradation plugging limit will continue to ensure the required safety factor of three against tube rupture under normal operating conditions and will continue to provide a margin of safety consistent with Section III of the ASME Boiler and Pressure Vessel Code under postulated accident conditions including LOCA and SSE. Furthermore, the proposed plugging limit includes a sufficient thickness degradation allowance to compensate for possible continued degradation between inservice inspections. Based on previous inservice inspection** there is no evidence of tube thinning in the PINGP steam generators. In the

*Supplemental Testimony of James P. Knight before the Atomic Safety and Licensing Appeal Board In the Matter of Northern States Power Company, Docket Nos. 50-282/306.

**Letter, L. O. Mayer to Director, Office of Nuclear Reactor Regulation, dated March 14, 1978.

event that significant tube thinning does occur the Technical Specifications require that NSP will reevaluate the 50% plugging limit. Therefore, we find that the proposed 50% degradation plugging limit is acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmetnal impact, and pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 25, 1978

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated May 26, 1978, supplemented July 24, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility License No. DPR-60 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 31 , are hereby incorporated

in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 25, 1978

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated May 26, 1978, supplemented July 24, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility License No. DPR-42 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 25 , are hereby incorporated in

the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 25, 1978

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-42

AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NOS. 50-282 AND 50-306

Revise Appendix A as follows:

1. Remove the following pages and replace with identically numbered pages.

TS 4.12-1	
TS 4.12-2	TS 6.7-2
TS 4.12-3	
TS 4.12-4	
TS 4.12-5	
TS 4.12-6	
TS 4.12-7	

2. Add the new table.

Table TS 4.12-1

4.12 STEAM GENERATOR TUBE SURVEILLANCE

Applicability

Applies to inservice surveillance of the steam generator tubes.

Objective

To assure the continued integrity of the steam generator tubes that are a part of the primary coolant pressure boundary.

Specification

Steam generator tubes in each unit shall be determined operable by the following:

A. Steam Generator Sample Selection and Inspection-Each steam generator shall be determined operable in accordance with the in-service inspection schedule in Specification 4.12.C. The in-service inspection may be limited to one steam generator on a rotating schedule encompassing 6% of the tubes in the single steam generator, provided the previous inspections indicated that the two steam generators are performing in a like manner.

B. Steam Generator Tube Sample Selection and Inspection-The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table TS.4.12-1. The in-service inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.12.C and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.12.D. The tubes selected for each in-service inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

1. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

2. The first sample of tubes selected for each in-service inpection (subsequent to the preservice inspection) of each steam generator shall include:

   (a) All nonplugged tubes that previously had detectable wall penetrations (>20%).

   (b) Tubes in those areas where experience has indicated potential problems.

   (c) A tube inspection (pursuant to Specification 4.12.D.1.(h)) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.

3. The tubes selected as the second and third samples (if required by Table TS.4.12-1) during each inservice inspection may be subjected to a partial tube inspection provided:

   (a) The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.

   (b) The inspections include those portions of the tubes where imperfections were previously found.

The results of each sample inspection shall be classified into one of the following three catgegories:

Category	Inspection Results
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes must exhibit significant (>10%) further wall penetrations to be included in the above percentage calculations.

C. Inspection Frequencies-The above required in-service inspections of steam generator tubes shall be performed at the following frequencies:

1. In-service inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 categroy or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.

2. If the results of the inservice inspection of a steam generator conducted in accordance with Table TS.4.12-1 at 40 month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.12.C.1; the interval may then be extended to a maximum of once per 40 months.

3. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table TS.4.12-1 during the shutdown subsequent to any of the following conditions.

   (a) Primary-to-secondary tube leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.1.C.6.

   (b) A seismic occurrence greater than the Operating Basis Earthquake.

   (c) A loss-of-coolant accident requiring actuation of the engineered safeguards.

   (d) A main steam line or feedwater line break.

D. Acceptance Criteria

1. As used in this Specification:

(a) Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.

(b) Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube.

(c) Degraded Tube means a tube containing imperfections $\geq$20% of the nominal wall thickness caused by degradation.

(d) % Degradation means the percentage of the tube wall thickness affected or removed by degradation.

(e) Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective.

(f) Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 50% of the nominal tube wall thickness. If significant general tube thinning occurs, this criteria will be reduced to 40% wall penetration.

(g) Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break.

(h) Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg.

2. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table TS.4.12-1.

E. Reports

1. Following each in-service inspection of steam generator tubes, if there are any tubes requiring plugging, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

2. Results of steam generator tube inspections which fall into Category C-3 require notification to the Commission prior to resumption of plant operation, and reporting as a Reportable Occurrence with prompt notification with written followup. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

BASIS

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. In-service inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or in-service conditions that lead to corrosion. In-service inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those parameters found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these parameters, localized corrosion would most likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator leakage between the primary coolant system and the secondary coolant system (primary-to-secondary leakage = 1.0 gpm) Cracks having a primary-to-secondary leakage less than 1.0 gpm during operation will have an adequate margin of safety against failure due to loads imposed by design basis accidents.[1] Operating plants have demonstrated that primary-to-secondary leakage as low as 0.1 gpm will be detected by radation monitors of steam generator blowdown. Leakage in excess of 1.0 gpm will require plant shutdown and an unscheduled eddy current inspection, during which the leaking tubes will be located and plugged.

Wastage-type defects are unlikely with proper chemistry treatment of secondary coolant. However, even if this type of defect occurs it will be found during scheduled in-service steam generator tube inspections Plugging will be required of all tubes with imperfections that could develop defects having less than the minimum acceptable wall thickness prior to the next inservice inspection which, by the definition of Specification 4.12.D.1.(f), is 50% of the tube nominal wall thickness (0.050 inches). Wastage type defects having a wall thickness greater than 0.025 inches will have adequate margins of safety against failure due to loads imposed by normal plant operation and design basis accidents.[1] Steam generator tube inspections of operating plants have

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[1] Testimony of J Knight in the Prairie Island Public Hearing on 1/28/75.

BASIS (Continued)

demonstrated the capability to reliably detect wastage type defects that have penetrated 20% of the original 0.050-inch wall thickness.[1]

Whenever the results of any steam generator tubing in-service inspection fall into Catgegory C-3, these results will be promptly reported to the Commission prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.

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[1] Testimony of L Frank in the Prairie Island Public Hearing on 1/28/75

TABLE TS.4.12-1

STEAM GENERATOR TUBE INSPECTION

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S.G.	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect additional 2S tubes in this S.G.	C-1	None	N/A	N/A
			C-2	Plug defective tubes and inspect additional 4S tubes in this S.G.	C-1	None
					C-2	Plug defective tubes
					C-3	Perform action for C-3 result of first sample
			C-3	Perform action for C-3 result of first sample	N/A	N/A
	C-3	Inspect all tubes in this S.G.,plug defective tubes and inspect 2S tubes in each other S.G. Prompt notification to NRC.	All other S.G.s are C-1	None	N/A	N/A
			Some S.G.s C-2 but no additional S.G. are C-3	Perform action for C-2 result of second sample	N/A	N/A
			Additional S.G. is C-3	Inspect all tubes in each S.G. and plug defective tubes. Promt notification to NRC.	N/A	N/A

$S = \frac{6\%}{n}$ Where n is the number of steam generators inspected during an inspection.

Table TS.4.12-1

2. Occupational Exposure Report.[1/] An annual report of occupational exposure covering the previous calendar year shall be submitted prior to March 1 of each year.

   The report should tabulate on an annual basis the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

3. Monthly Operating Report. A monthly report of operating statistics and shutdown experience covering the previous month shall be submitted by the 15th of the following month to the Office of Management Information and Program Control, U S Nuclear Regulatory Commission, Washington, D.C. 20555

4. Steam Generator Tube Inservice Inspection. The results of steam generator tube inservice inspections shall be reported within 90 days of January 1 for all inspections completed during the previous calendar year. These reports shall include; (1) number and extent of tubes inspected, (2) location and percent of wall-thickness penetration for each indication of an imperfection, and (3) identification of tubes plugged.

B. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

1/ This report supplements the requirements of 10CFR20, Section 20.407. If 10CFR20, Section 20.407 is revised to include such information, this Specification is unnecessary.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-282 AND 50-306

NORTHERN STATES POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 31 and 25 to Facility Operating License Nos. DPR-42 and DPR-60, issued to the Northern States Power Company (the licensee), which revised Technical Specifications for operation of the Unit Nos. 1 and 2 of the Prairie Island Nuclear Generating Plant (the facilities) located in Goodhue County, Minnesota. The amendments will become effective as of the date of their issuance.

The amendments revised the Technical Specifications for the facilities relating to the steam generator tube inservice inspection.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendments dated May 26, 1978, supplemented July 24, 1978, (2) Amendment Nos. 31 and 25 to License Nos. DPR-42 and DPR-60, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Environmental Conservation Library of the Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A single copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 25th day of August, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

Gary G. Zech, Acting Chief
Operating Reactors Branch #1
Division of Operating Reactors

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-282 AND 50-306

NORTHERN STATES POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 31 and 25 to Facility Operating License Nos. DPR-42 and DPR-60, issued to the Northern States Power Company (the licensee), which revised Technical Specifications for operation of the Unit Nos. 1 and 2 of the Prairie Island Nuclear Generating Plant (the facilities) located in Goodhue County, Minnesota. The amendments will become effective as of the date of their issuance.

The amendments revised the Technical Specifications for the facilities relating to the steam generator tube inservice inspection.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendments dated May 26, 1978, supplemented July 24, 1978, (2) Amendment Nos. 31 and 25 to License Nos. DPR-42 and DPR-60, respectively, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Environmental Conservation Library of the Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A single copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 25th day of August, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

Gary G. Zech, Acting Chief
Operating Reactors Branch #1
Division of Operating Reactors

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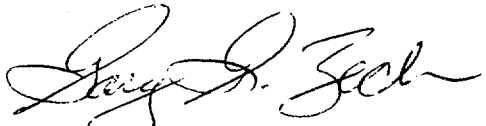

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