

Docket Nos. '50-282
and ~~50-306~~

AUG 11 1976

Northern States Power Company
ATTN: Mr. L. O. Mayer, Manager
Nuclear Support Services
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Gentlemen:

We have reviewed your request dated May 27, 1976, for a temporary exemption from the requirements of Appendix J of 10 CFR Part 50 and Section 4.0 of the Technical Specifications for Unit No. 2 of the Prairie Island Nuclear Generating Plant, License No. DPR-60. The exemption would grant a one-month extension of the containment leak rate test interval. In addition, you requested a Technical Specification change to allow a one-month extension for the maximum interval for refueling outage surveillance tests. Both extensions are for the first operating cycle of Unit No. 2.

Based on the considerations set forth in the enclosed Safety Evaluation, we have concluded that the exemption from the requirements of Appendix J to 10 CFR Part 50 which extends the maximum interval between containment leak rate tests for the first refueling shutdown to 25 months will not endanger life or property or the common defense and security and is otherwise in the public interest. Therefore, the exemption requested in your letter dated May 27, 1976, is granted.

The appropriate changes to your Technical Specifications to reflect the provisions of the above exemption and to include approval of your requested one-month extension for the refueling outage surveillance test interval for Unit No. 2 are contained in the enclosed Amendment No. 9 to Facility License No. DPR-60. During our review of your proposed request we found that certain changes were necessary for clarification. Your staff has agreed to these changes and they have been incorporated. Since Unit Nos. 1 and 2 share a common set of Technical Specifications, we also have issued the enclosed Amendment No. 15 to Facility License No. DPR-42 for Unit No. 1.

AUG 11 1976

A copy of the Notice of Issuance which is being submitted to the Office of the Federal Register for publication also is enclosed.

Sincerely,

Signed by Original Signed by
V. Stello

Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Amendment No. 9
to DPR-60
3. Amendment No. 15
to DPR-42
4. Notice

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 11, 1976

Docket Nos. 50-282
and 50-306

Northern States Power Company
ATTN: Mr. L. O. Mayer, Manager
Nuclear Support Services
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Gentlemen:

We have reviewed your request dated May 27, 1976, for a temporary exemption from the requirements of Appendix J of 10 CFR Part 50 and Section 4.0 of the Technical Specifications for Unit No. 2 of the Prairie Island Nuclear Generating Plant, License No. DPR-60. The exemption would grant a one-month extension of the containment leak rate test interval. In addition, you requested a Technical Specification change to allow a one-month extension for the maximum interval for refueling outage surveillance tests. Both extensions are for the first operating cycle of Unit No. 2.

Based on the considerations set forth in the enclosed Safety Evaluation, we have concluded that the exemption from the requirements of Appendix J to 10 CFR Part 50 which extends the maximum interval between containment leak rate tests for the first refueling shutdown to 25 months will not endanger life or property or the common defense and security and is otherwise in the public interest. Therefore, the exemption requested in your letter dated May 27, 1976, is granted.

The appropriate changes to your Technical Specifications to reflect the provisions of the above exemption and to include approval of your requested one-month extension for the refueling outage surveillance test interval for Unit No. 2 are contained in the enclosed Amendment No. 9 to Facility License No. DPR-60. During our review of your proposed request we found that certain changes were necessary for clarification. Your staff has agreed to these changes and they have been incorporated. Since Unit Nos. 1 and 2 share a common set of Technical Specifications, we also have issued the enclosed Amendment No. 15 to Facility License No. DPR-42 for Unit No. 1.

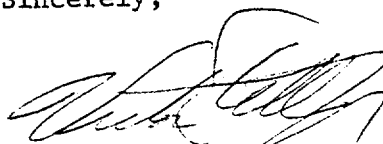
Northern States Power Company

- 2 -

August 11, 1976

A copy of the Notice of Issuance which is being submitted to the Office of the Federal Register for publication also is enclosed.

Sincerely,



Victor Stello, Jr., Director
Division of Operating Reactors
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Amendment No. 9
to DPR-60
3. Amendment No. 15
to DPR-42
4. Notice

cc w/enclosures:
See next page

August 11, 1976

cc w/enclosures:

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Mr. John E. Davidson, Chairman
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Red Wing, Minnesota 55066

cc w/enclosures and copy of NSPCo
filing dtd. 5/27/76

Mr. Norman M. Clapp, Chairman
Public Service Commission of
Wisconsin
Hill Farms State Office Building
Madison, Wisconsin 53702



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 15 AND 9
TO FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

INTRODUCTION

By letter dated May 27, 1976, the Northern States Power Company (NSP) requested a temporary exemption to Appendix J of 10 CFR Part 50, and amendments to Facility Operating Licenses Nos. DPR-42 and DPR-60 for the Prairie Island Nuclear Generating Plant Units 1 and 2 (PINGP). The temporary exemption is for a one-month extension of the interval for the containment leak rate tests required by 10 CFR Part 50 Appendix J. The amendment request involves a one-month extension to the maximum interval for refueling outage surveillance tests set forth in section 4.0 of the Technical Specifications for PINGP. Those tests relate to the first operating cycle of Unit No. 2 of PINGP.

DISCUSSION

The issuance date of DPR-60 was October 29, 1974, and therefore the performance of containment leak rate tests and refueling outage surveillance tests would be due on October 29, 1976. The initial criticality of Unit 2 occurred on December 17, 1974. The first cycle was planned to be terminated in September 1976 but a turbine blade failure caused an unplanned shutdown in December of 1975. The turbine was placed back in service with baffles installed in place of the last three rows of blades in the LP-2 rotor. Since replacement parts will not be available until early December 1976, the refueling outage will be rescheduled to mid-November 1976 to avoid a lengthening of the refueling outage or scheduling an additional outage to repair the turbine. The one month extension of time to perform the tests is necessary to accommodate this schedule change.

There have been no Unit 2 outages during which the tests could have been performed since the turbine was placed back in service on January 20, 1976, and at that time, schedules for the availability for turbine parts were not known. There are no outages scheduled between now and the refueling outage, however, should an outage of 10 days or more occur, the licensee would perform as many of the tests as possible.

The first refueling interval is frequently longer than subsequent fuel cycles because of the startup program which is part of the first cycle and also because unplanned outages are more likely in a new plant. An additional reactor shutdown, cooldown, and depressurization cycle would result in significant costs to NSP and its customers.

During the review of the proposed request we found that certain changes were necessary for clarification. The licensee has agreed to the changes.

EVALUATION

During the leak rate testing of Unit 1, conducted in August 1975, a number of penetrations with leak rates in excess of allowable limits were found. Based on the results of the Unit 1 tests, all suspect penetrations were tested on Unit 2. Only one of the Unit 2 penetrations was found to exceed the allowable leak rate and this penetration was repaired.

The frequency for containment leak rate and refueling outage surveillance tests was selected to coincide with the refueling interval which is normally not more than two years after the first refueling. A shutdown and cooldown specifically and solely for these tests would result in an unnecessary plant thermal cycle. Such thermal cycles are limited by design to minimize the effects of thermal and mechanical stresses. On this basis, it is desirable to combine these tests with some other scheduled shutdown event, such as refueling. To impose an extra shutdown a month prior to refueling would be a poor operating practice as well as being an economic penalty.

On the basis of the foregoing discussion and evaluation, the temporary exemption from the requirements of 10 CFR Part 50 (Appendix J) and the one month extension of the interval between refueling outage surveillance tests for Unit 2 of PINGP is acceptable.

ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) granting the exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest, (2) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (3) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (4) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 11, 1976



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 2

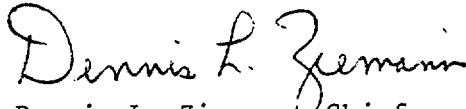
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. DPR-60

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated May 27, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 11, 1976



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT NO. 1

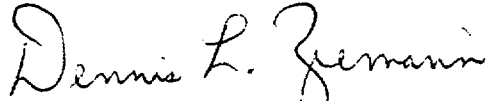
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-42

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated May 27, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 11, 1976

ATTACHMENT TO LICENSE AMENDMENT NOS. 15 AND 9

FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following existing page TS 4.1-1 of the Technical Specifications contained in Appendix A with attached revised page bearing the same number. Changed areas on the page are denoted by marginal lines.

4.0 SURVEILLANCE REQUIREMENTS

Specific time intervals between tests may be adjusted plus or minus 25% to accommodate normal test schedules with the exception that, the intervals between tests scheduled for refueling shutdowns shall not exceed two years.⁽¹⁾

4.1 OPERATIONAL SAFETY REVIEW

Applicability

Applies to items directly related to safety limits and limiting conditions for operation.

Objective

To specify the minimum frequency and type of surveillance to be applied to plant equipment and conditions.

Specification

- A. Calibration, testing, and checking of instrumentation channels and testing of logic channels shall be performed as specified in Table TS.4.1-1.
- B. Equipment tests shall be conducted as specified in Table TS.4.1-2A.
- C. Sampling tests shall be conducted as specified in Table TS.4.1-2B.
- D. Whenever containment integrity is not required, only the asterisked items in Tables 4.1-1, 4.1-2A, and 4.1-2B must be continued beyond the first test that follows such a plant condition. An initial test for all discontinued items shall be made prior to resumption of containment integrity. The time interval between such initial tests and initiation of containment integrity shall be less than the test interval for the items.

Basis

Channel Check

Failures such as blown instrument fuses, defective indicators, faulted amplifiers which result in "upscale" or "downscale" indication can be easily recognized by simple observation of the functioning of an instrument or system. Furthermore, such failures are, in many cases, revealed by alarm or annunciator action, and a check supplements this type of built-in surveillance.

Based on experience in operation of both conventional and nuclear plant systems, when the plant is in operation, the minimum checking frequencies set forth are deemed adequate for reactor and steam system instrumentation.

(1) The interval between tests scheduled for refueling shutdowns may be 25 months for the first operating cycle of Unit 2.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-282 AND 50-306

NORTHERN STATES POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 15 and 9 to Facility Operating License Nos. DPR-42 and DPR-60, issued to the Northern States Power Company (the licensee), which revised Technical Specifications for operation of Units 1 and 2 of the Prairie Island Nuclear Generating Plant (the facilities) located in Goodhue County, Minnesota. The amendments are effective as of their date of issuance.

The amendments revised the Appendix A portion of the Technical Specifications for the facilities to allow a one-month extension of the maximum interval for refueling outage surveillance tests for the first operating cycle of Unit No. 2 of the Prairie Island Nuclear Generating Plant. This action also incorporated a temporary exemption from the requirements of Section III.D of Appendix J of 10 CFR Part 50 that allows containment leak rate tests for the first operating cycle of Unit No. 2 to be postponed for one month.

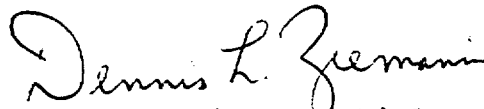
The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR § 51.5(d)(4) an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendment dated May 27, 1976, (2) Amendment Nos. 15 and 9 to License Nos. DPR-42 and DPR-60, respectively, (3) the Commission's concurrently issued related Safety Evaluation, and (4) the Commission's letter to the Northern States Power Company being issued concurrently with this Notice. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at The Environmental Conservation Library of the Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 11th day of August, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors