

August 13, 2002

Mr. Jeffrey S. Forbes
Site Vice President
Monticello Nuclear Generating Plant
Nuclear Management Company, LLC
2807 West County Road 75
Monticello, MN 55362-9637

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT - REQUEST FOR ADDITIONAL
INFORMATION RELATED TO LICENSE AMENDMENT REQUEST
(TAC NO. MB4976)

Dear Mr. Forbes:

By application dated April 22, 2002, the Nuclear Management Company (the licensee), LLC, requested an amendment to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The proposed amendment would revise the reactor vessel pressure and temperature limit curves in the Monticello Technical Specifications in order to allow required hydrostatic and leak tests to be performed at a significantly lower temperature. This is expected to reduce challenges to plant operators associated with maintaining the reactor coolant system within a narrow temperature band during testing.

To complete its review of the proposed amendment, the Nuclear Regulatory Commission staff requests the licensee's response to the enclosed questions. After discussing this request with Mr. D. Neve of your staff, a mutually agreeable due date of September 16, 2002, for your response was established. If you need to revise the due date, please contact me at (301) 415-2303 at the earliest opportunity.

Sincerely,

/RA/

Samuel Miranda, Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosure: Request for Additional Information

cc w/encl: See next page

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Monticello Nuclear Generating Plant

cc:

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March 2002

REQUEST FOR ADDITIONAL INFORMATION
RELATED TO LICENSE AMENDMENT REQUEST DATED APRIL 22, 2002
MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

The Nuclear Regulatory Commission staff requires the following information to complete its review of the license amendment request for the Monticello pressure-temperature (P-T) limit curves, dated April 22, 2002, that was proposed based on the methods of Code Case N-640, "Alternative Reference Fracture Toughness for Development of P-T Limit Curves, Section XI, Division 1":

1. Submit all P-T limit point inputs used to generate the proposed unirradiated core beltline curves and reactor vessel (RV) non-beltline curves for pressure testing conditions, normal operating conditions with the core-not-critical (including times during heatups and cooldowns of the reactor and transient operating conditions), and normal operating conditions with the core-critical. Include in the submittal the following parameter inputs that contributed to the calculation of the P-T limit data points:
 - a. K_{IT} values and temperature gradient values (i.e., the ΔT values between the temperatures for RCS coolant and those at the $1/4T$ thickness location of the RV) used for calculation of the P-T limit data points.
 - b. Identification of the limiting unirradiated RT_{NDT} values and limiting materials for the beltline and non-beltline curves. With respect to submittal of this information, confirm that the Monticello RV beltline region is plate-limited and that Heat No. C2220 (used for fabrication of RV plates I-14 and I-15) is the limiting material for the beltline region of the RV.
2. Submit the P-T limit data points for the irradiated beltline curves that will be used by the Monticello control room operators during pressure testing conditions, normal operating conditions with the core-not-critical (including times during heatups and cooldowns of the reactor and transient operating conditions), and normal operating conditions with the core-critical.