



Kewaunee Nuclear Power Plant
N490 Highway 42
Kewaunee, WI 54216-9511
920.388.2560

Point Beach Nuclear Plant
6610 Nuclear Road
Two Rivers, WI 54241
920.755.2321

Kewaunee / Point Beach Nuclear
Operated by Nuclear Management Company, LLC

NRC-02-068

July 24, 2002

10 CFR 50, App. E

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Ladies/Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
Radiological Emergency Response Plan Implementing Procedures

Pursuant to 10 CFR 50 Appendix E, attached is the latest revisions to the Kewaunee Nuclear Power Plant Radiological Emergency Response Plan Implementing Procedures (EPIPs). These revised procedures supersede the previously submitted procedures.

Pursuant to 10 CFR 50.4, two additional copies of this letter and attachment are hereby submitted to the Regional Administrator, U. S. Nuclear Regulatory Commission, Region III, Lisle, Illinois. As required, one copy of this letter and attachment is also submitted to the Kewaunee Nuclear Power Plant NRC Senior Resident Inspector.

Sincerely,

Small O. Kato for
Thomas J. Webb
Regulatory Affairs Manager

SLC

Attachment

cc - US NRC Senior Resident Inspector, w/attach.
US NRC, Region III (2 copies), w/attach.
Electric Division, PSCW, w/o attach.
QA Vault, wo/attach.

A045

DOCUMENT TRANSMITTAL

KEWAUNEE NUCLEAR POWER PLANT

FROM: DIANE FENCL - KNPP

TRANSMITTAL DATE 07-19-2002

EMERGENCY PLAN IMPLEMENTING PROCEDURES TRANSMITTAL FORM

OUTSIDE AGENCY COPIES (1-20)

T. Webb - NRC Document Control Desk (1)* Krista Kappelman - PBNP - EP (10)*
T. Webb - NRC Region III (2, 3)* Craig Weiss - Alliant Energy (11)*
T. Webb - NRC Resident Inspector (4) (receives Appx. A phone numbers)*
T. Webb - State of Wisconsin (5)* Jill Stern - Nuclear Management Company (12)*
T. Webb - KNPP QA Vault (NRC Letter & Memo Only) (15)*

PERSONAL COPIES (21-40) These copies are for the personal use of the listed individuals for reference or emergency response.

J. Bennett (33) D. Seebart (24) J. Ferris (13) T. Coutu (28)

REFERENCE COPIES - CUSTODIAN (41-100) These copies are for general reference by anyone. They are distributed throughout the plant and corporate offices. The named individual is the responsible custodian for the procedures and shall insure they are properly maintained.

NO Library - KNPP (59) Resource Center - Training (82)
C. Sternitzky - ATF-2 (44) D. Krall - CR/SS Office (51, 56)
M. Daron - Security Building (46) M. Lambert - TSC (50)
M. Lambert - EOF (81) W. Galarneau - RAF (53)
M. Lambert - OSF (52) W. Galarneau - SBF/EMT (54)
LOREB - STF (62, 66, 67, 68, 70, 72, 73, 74) W. Galarneau - RPO (55)
STF Library (43) STF (86, 87, 88)

WORKING COPIES (101-199) These copies of procedures are kept in the areas designated for use in response to an emergency.

W. Galarneau - RAF/RPO (106, 107) D. Krall - CR/Communicator (116)(Partial Distribution)
W. Galarneau - SBF/ENV (108, 109) Simulator/Communicator (117)
W. Galarneau - SBF/EM Team (110, 111, 111A) M. Fencl - Security (121)
W. Flint - Cold Chem/HR Sample Room (113) M. Kuether - Security Building (120)
M. Kuether - SBF/SEC (114) J. Stoeger (126)

Originals to KNPP QA Vault

Please follow the directions when updating your EPIP Manual. **WATCH FOR DELETIONS!!!** These are controlled procedures and random checks may be made to ensure the manuals are kept up-to-date.

***THIS IS NOT A CONTROLLED COPY. IT IS A COPY FOR INFORMATION ONLY.**

**KEWAUNEE NUCLEAR POWER PLANT
 REVISION OF EMERGENCY PLAN IMPLEMENTING PROCEDURES
 July 19, 2002**

Please follow the directions listed below. If you have any questions regarding changes made to the EIPs, please contact Dave Seebart at ext. 8719.

EPIP Index, dated 07-19-2002.

REMOVE		INSERT	
PROCEDURE	REV.	PROCEDURE	REV.
EPIP-AD-02	AC	EPIP-AD-02	AD
EPIP-SEC-03	AD	EPIP-SEC-03	AE

Return a signed and dated copy of this transmittal letter, within 10 days of transmittal date, to the sender. If you have any questions or comments, please contact Dave Seebart at ext. 8719.

I CERTIFY Copy No. _____ (WPSC No.) of the Kewaunee Nuclear Power Plant's EIPs has been updated.

 SIGNATURE DATE

Please return this sheet to *DIANE FENCL*.

Diane Fencl
 Diane Fencl

Enclosure

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EP-AD			
EPIP-AD-01	Personnel Response to the Plant Emergency Siren	J	01-08-2002
EPIP-AD-02	Emergency Class Determination	AD	07-19-2002
EPIP-AD-03	KNPP Response to an Unusual Event	AF	06-20-2002
EPIP-AD-04	KNPP Response to Alert or Higher	AJ	06-20-2002
EP-AD-5	Site Emergency	Deleted	04-27-87
EPIP-AD-05	Emergency Response Organization Shift Relief Guideline	D	05-09-2002
EP-AD-6	General Emergency	Deleted	04-24-87
EPIP-AD-07	Initial Emergency Notifications	AR	06-20-2002
EP-AD-8	Notification of Alert or Higher	Deleted	02-26-96
EP-AD-9	Notification of Site Emergency	Deleted	04-27-87
EP-AD-10	Notification of General Emergency	Deleted	04-27-87
EPIP-AD-11	Emergency Radiation Controls	R	04-11-2002
EP-AD-12	Personnel Assembly and Accountability	Deleted	03-26-94
EP-AD-13	Personnel Evacuation	Deleted	04-25-94
EP-AD-13A	Limited Area Evacuation	Deleted	03-01-83
EP-AD-13B	Emergency Assembly/Evacuation	Deleted	03-01-83
EP-AD-13C	Site Evacuation	Deleted	03-01-83
EP-AD-14	Search and Rescue	Deleted	05-25-94
EPIP-AD-15	Recovery Planning and Termination	O	10-30-2001
EP-AD-16	Occupational Injuries or Vehicle Accidents During Emergencies	Deleted	03-14-97
EP-AD-17	Communications	Deleted	03-05-84
EPIP-AD-18	Potassium Iodide Distribution	P	02-27-2002
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EPIP-AD-20	KNPP Response to a Security Threat	C	05-23-2002
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EPIP-ENV-01	Environmental Monitoring Group Organization and Responsibilities	V	10-02-2001
EPIP-ENV-02	Environmental Monitoring Team Activation	X	10-02-2001
EP-ENV-3A	Environmental Protection Director Actions and Directives	Deleted	09-26-84
EP-ENV-3B	EM Team Actions	Deleted	09-26-84
EPIP-ENV-03C	Dose Projection Using RASCAL Version 2.2 Software	V	10-09-2001
EP-ENV-3D	Revision and Control of ISODOSE II	Deleted	02-14-95
EP-ENV-3E	Manual Determination of X/Q	Deleted	04-24-87
EP-ENV-3F	Manual Determination of X/Q (Green Bay Meteorological Data)	Deleted	05-30-86
EP-ENV-3G	Manual Dose Projection Calculation	Deleted	06-02-89
EP-ENV-3H	Protective Action Recommendations	Deleted	04-13-90
EPIP-ENV-04A	Portable Survey Instrument Use	S	06-15-2000
EPIP-ENV-04B	Air Sampling and Analysis	W	10-09-2001
EP-ENV-4C	Environmental Monitoring Teams	Deleted	04-13-90
EPIP-ENV-04C	Ground Deposition Sampling and Analysis	W	10-09-2001
EPIP-ENV-04D	Plume Tracking for Environmental Monitoring Teams	N	10-02-2001
EP-ENV-5A	LCS-1 Operation	Deleted	04-14-86
EP-ENV-5B	MS-3 Operation	Deleted	04-14-86
EP-ENV-5C	SAM II Operation	Deleted	04-14-86
EP-ENV-5D	PAC-4G (Alpha Counter) Operation	Deleted	04-14-86
EP-ENV-5E	Reuter-Stokes Operation	Deleted	08-27-85

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EP-ENV-6	Data Analysis, Dose Projections and Protective Action Recommendations	Deleted	12-21-81
EP-ENV-6	Alternate Sample Analysis and Relocation of EM Team	Deleted	04-14-86
EP-ENV-6A	Relocation of Site Access Facility (Habitability)	Deleted	03-23-84
EP-ENV-6B	SAF Environmental Sample Analysis Relocation	Deleted	03-23-84
EP-ENV-7	Site Access Facility Communications	Deleted	09-26-84
EP-ENV-8	Total Population Dose Estimate Calculations	Deleted	04-14-86
EP-EOF			
EP-EOF-1	Corporate Emergency Response Organization	Deleted	03-11-94
EPIP-EOF-02	Emergency Operations Facility (EOF) Activation	Z	11-29-2001
EPIP-EOF-03	EOF Staff Action for Unusual Event	AC	02-06-2002
EPIP-EOF-04	EOF Staff Action for Alert or Higher	AI	02-06-2002
EP-EOF-5	Corporate Staff Action for Site Emergency	Deleted	04-24-87
EP-EOF-6	Corporate Staff Action for General Emergency	Deleted	04-24-87
EP-EOF-7	Notification of Unusual Event	Deleted	04-06-94
EP-EOF-8	Relocation of EOF	Deleted	03-01-83
EPIP-EOF-08	Continuing Emergency Notifications	X	06-20-2002
EP-EOF-9	Interface with Support Organizations	Deleted	03-05-84
EP-EOF-9	Notification of Site Emergency	Deleted	04-24-87
EP-EOF-10	Notification of General Emergency	Deleted	04-24-87
EPIP-EOF-11	Internal Communication and Documentation Flow	U	11-15-2001
EPIP-EOF-12	Media Center/Emergency Operation Facility/Joint Public Information Center Security	Q	06-20-2002

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EP-OP			
EP-OP-1	Control Room Emergency Organization	Deleted	04-24-87
EP-OP-2	Emergency Control Room Activation for Emergency Response	Deleted	04-24-87
EP-OP-3	Control Room Communications	Deleted	04-24-87
EP-OSF			
EP-OSF-1	Operation Support Facility Emergency Organization	Deleted	04-24-87
EPIP-OSF-02	Operational Support Facility Operations	U	02-06-2002
EPIP-OSF-03	Work Orders During an Emergency	P	05-09-2002
EP-OSF-4	Operational Support Facility Communications	Deleted	04-24-87
EPIP-OSF-04	Search and Rescue	E	05-23-2002
EP-RET			
EP-RET-1	Radiation Emergency Team Organization	Deleted	04-16-96
EPIP-RET-02	In-Plant Radiation Emergency Team	V	05-23-2002
EPIP-RET-02A	Radiation Protection Office/Radiological Analysis Facility (RPO/RAF) Activation	T	11-29-2001
EPIP-RET-02B	Gaseous Effluent Sample and Analysis	R	04-11-2002
EP-RET-2C	Containment Air Sampling and Analysis	Deleted	03-01-83
EPIP-RET-02D	Emergency Radiation Entry Controls and Implementation	M	06-12-2001
EP-RET-2E	Handling of Injured Personnel	Deleted	04-16-96
EP-RET-2F	Personnel Decontamination	Deleted	04-13-90
EPIP-RET-03	Chemistry Emergency Team	O	02-01-2000
EPIP-RET-03A	Liquid Effluent Release Paths	L	11-29-2001
EP-RET-3B	Post-Accident Reactor Coolant Alternate Sampling Procedure	Deleted	01-25-88

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EPIP-RET-03C	Post Accident Operation of the High Radiation Sample Room	P	01-15-2002
EPIP-RET-03D	Containment Air Sampling Analysis Using CASP	N	01-15-2002
EP-RET-3E	Post Accident Operation of High Rad Sample Room Inline Multiported Count Cave	Deleted	08-27-85
EPIP-RET-04	SBF Activation	S	06-20-2002
EP-RET-4A	EOF Radiological Monitoring	Deleted	03-10-83
EPIP-RET-04A	SBF Operation/Relocation	Deleted	10-02-2001
EP-RET-4B	Radiological Controls at Site Access Facility	Deleted	07-12-94
EP-RET-4C	Site Radiological Monitoring	Deleted	07-12-94
EP-RET-4D	SAM-II Operation	Deleted	07-12-94
EP-RET-5	Plume Projection	Deleted	09-26-84
EPIP-RET-05	Site Boundary Dose Rates During Controlled Plant Cooldown	H	10-09-2001
EP-RET-5A	Plume Projection	Deleted	04-27-87
EP-RET-6	Dose Projection	Deleted	04-24-87
EP-RET-7	Radiological Analysis Facility/Radiation Protection Office Communications	Deleted	04-24-87
EPIP-RET-08	Contamination Control of the Aurora Medical Center	Deleted	05-23-2002
EPIP-RET-09	Post-Accident Population Dose	L	04-16-2002
EP-SEC			
EP-SEC-1	Security Organization	Deleted	04-24-87
EPIP-SEC-02	Security Force Response to Emergencies	X	02-06-2002
EP-SEC-2A	Manual Activation of Emergency Sirens	Deleted	04-16-82
EPIP-SEC-03	Personnel Assembly and Accountability	AE	07-19-2002
EPIP-SEC-04	Security Force Actions for Dosimetry Issue	P	10-02-2001

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EP-SEC-5	Security Force Response to the EOF	Deleted	07-28-88
EPIP-SEC-05	Personnel Evacuãtion	G	06-20-2002
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EPIP-TSC-01	Technical Support Center Organization and Responsibilities	Q	02-06-2002
EPIP-TSC-02	Technical Support Center Activation	T	02-06-2002
EPIP-TSC-03	Plant Status Procedure	V	10-09-2001
EPIP-TSC-04	Emergency Physical Changes, Major Equipment Repair	N	05-09-2002
EP-TSC-5	Technical Support Center Communications Equipment	Deleted	04-24-87
EP-TSC-6	Assessment of Reactor Core Damage	Deleted	09-30-86
EPIP-TSC-07	RV Head Venting Time Calculation	J	06-20-2002
EPIP-TSC-08A	Calculations for Steam Release from Steam Generators	N	12-14-2001
EPIP-TSC-08B*	STMRLS Computer Program	G	06-20-2002
EP-TSC-8C*	See EP-TSC-8B	Deleted	04-16-92
* EP-TSC-8B was totally deleted; therefore, EP-TSC-8C was changed to EP-TSC-8B			
EP-TSC-9	Core Damage Assessment Using Released Radionuclides	Deleted	09-30-86
EPIP-TSC-09A*	Core Damage Assessment	J	05-16-2002
EPIP-TSC-09B*	CORE Computer Program	Deleted	05-16-2002
EP-TSC-9C*	See EP-TSC-9B	Deleted	04-16-92
* EP-TSC-9A, Rev. D was totally deleted; therefore, EP-TSC-9B became EP-TSC-9A. EP-TSC-9B was previously EP-TSC-9C.			
EPIP-TSC-10	Technical Support for IPEOPs	K	05-09-2002

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EPIP-APPX-A-06	EP-FIG-005	APPX-A-06-02	Site Boundary Facility - KNP Floor Plan	A	10-31-2000
EPIP-APPX-A-06	EP-FIG-008	APPX-A-06-01	Radiological Analysis Facility - KNP Floor Plan	A	10-31-2000
EPIP-EOF-12 Form EPIPF-EOF-02-01	EP-FIG-009	EOF-12-01	Division Office Building (2nd Floor) Floor Plan	B	10-24-2000
EPIP-APPX-A-06	EP-FIG-012	APPX-A-06-08	State/County Work Area - WPSC D2-1 Floor Plan	C	10-31-2000
EPIP-APPX-A-06	EP-FIG-013	APPX-A-06-09	NRC Work Area - WPSC D2-4 Floor Plan	A	10-31-2000
EPIP-AD-19	EP-FIG-014	AD-19-01	Population Distribution by Geographical Sub-Areas (with sectors)	A	10-31-2000
EPIP-APPX-A-06	EP-FIG-022	APPX-A-06-04	EOF - WPSC D2-3 Floor Plan	C	10-30-2001
EPIP-EOF-12	EP-FIG-024	EOF-12-02	Location of JPIC and Media Briefing Center Map	C	06-20-2002
EP-SEC-5	EP-FIG-026	SEC-05-01	KNP Site Map & Evacuation Routes	C	06-20-2002
APPX-A-6	EP-FIG-034	---	Floor Plan - Media Briefing Center	Deleted	08-04-98
EPIP-EOF-12 EPIP-APPX-A-06	EP-FIG-035	APPX-A-06-06	General Office Building - WPSC (1st Floor) Floor Plan	C	10-24-2000
APPX-A-6	EP-FIG-037	---	Floor Plan - Corporate Response Center	Deleted	08-04-98
APPX-A-6	EP-FIG-038	---	Floor Plan - JPIC	Deleted	08-04-98
EPIP-OSF-02	EP-FIG-039	OSF-02-01	High Priority Work	A	10-02-2001
EPIP-OSF-02	EP-FIG-039A	OSF-02-02	Lower Priority Work	A	10-02-2001
EPIP-APPX-A-06	EP-FIG-043	APPX-A-06-10	JPIC - Federal Work Area - WPSC D2-9	B	12-21-2001
EPIP-APPX-A-06	EP-FIG-044	APPX-A-06-07	JPIC - State and County Work Area - WPSC D2-8	C	12-21-2001
EPIP-APPX-A-06	EP-FIG-045	APPX-A-06-05	JPIC - Utility Work Area - WPSC D2-7	C	12-21-2001
RET-08	EP-FIG-046	RET-08-01	Aurora Medical Center Location	Deleted	05-23-2002
EPIP-APPX-A-02	---	APPX-A-02-01	ERO Call Tree	Deleted	12-04-2001

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	APPENDIX A		
APPX-A-1	Communication System Description	AF	08-04-98
EPIP-APPX-A-02	Response Personnel Call List	Deleted	02-06-2002
EPIP-APPX-A-03	Off-Site Telephone Numbers	Deleted	02-06-2002
EPIP-APPX-A-06	KNPP Emergency Response Facility Telephone Numbers	AA	12-21-2001

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AD-07-02	State Call-Back - Question Guideline	C	11-15-2001
AD-11-01	Emergency Radiation Work Permit	G	04-11-2002
AD-18-01	Airborne Radioiodine Dose Accountability and Potassium Iodide Distribution	A	02-27-2002
AD-18-02	Record of Known Allergy To or Voluntary Refusal to Take Potassium Iodide	A	02-27-2002
EP-ENV			
ENV-01-01	Environmental Dispatch Area Activation Checklist	D	10-31-2000
ENV-01-02	EMT Status	B	10-31-2000
ENV-01-03	Meteorological and Plant Status Data	C	12-14-2001
ENV-01-04	EMT Orders/Field Data	B	10-31-2000
ENV-02-01	EMT Activation Checklist	M	06-15-2000
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EOF-02-02	EOF Deactivation Checklist	L	10-30-2001
EOF-04-01	SRCL Initial Action Checklist	C	12-14-2001
EOF-04-02	Telephone Communications Log Sheet	A	12-14-2001
EOF-08-03	Fax for Emergency Declaration or Status Updates	G	11-27-2001
EOF-08-05	Plant Emergency Status Report	A	11-27-2001
EOF-08-06	Radiological Status Report	D	11-27-2001
EOF-11-02	Operating Status	F	11-15-2001
EOF-11-03	Environmental Status Board	F	11-15-2001
EOF-12-01	I.D. Badge Registration Form	G	10-24-2000

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OSF-03-01	Operational Support Facility Team Briefing	C	12-04-2001
EP-RET			
RET-02A-02	Emergency Sample Worksheet	E	06-05-2001
RET 2B.1	Containment Stack Release (Grab Sample)	C	04-16-96
RET 2B.2	Auxiliary Building Stack (Grab Sample)	C	04-16-96
RET 2B.3	Auxiliary Building Stack (Sping Reading)	C	04-16-96
RET 2B.4	Containment Stack (Sping Reading)	B	04-16-96
RET 2B.5	Steam Release	C	04-16-96
RET 2B.6	Field Reading (Grab Sample)	A	04-16-96
RET-04-01	SAM-2 Counting Equipment Worksheet	E	06-12-2001
RET 8.3	Hospital Survey 1	Deleted	06-05-2001
RET 8.4	Hospital Survey 2	Deleted	07-25-97
RET 8.5	Hospital Survey 3	Deleted	07-25-97
RET-08-06	Hospital Survey 4	Deleted	05-23-2002
RET-09-01	Post-Accident TLD Record Sheet	D	04-16-2002
EP-SEC			
SEC-03.01	Emergency Accountability Log	A	03-28-2000
SEC-04-01	Emergency Dosimeter Log	G	06-20-2002
EP-TSC			
TSC-01.01	Plant Status Summary for SAM Implementation	B	02-06-2002
TSC-01.02	Severe Accident Management Summary and Strategy Recommendation	B	02-06-2002
TSC-01.03	Severe Accident Management – Status	B	02-06-2002
TSC-02-01	TSC and OSF Actiyation Checklist	O	09-27-2001

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TSC-02-04	TSC Chart Recorder Operation Checklist	D	01-30-2001
TSC-02-05	TSC and OSF De-activation Checklist	A	10-09-2001
TSC-03-01	Plant System Status	L	06-12-2001
TSC-03-02	Plant Equipment Status	L	06-12-2001
TSC-03-03	Environmental Status Board	J	06-12-2001
TSC-03-04	Radiation Monitors	I	01-08-2002
TSC-04-01	Emergency Physical Change Request	G	05-09-2002
TSC-04-02	Emergency Physical Change Safety Review	Deleted	05-09-2002
TSC-04-03	Emergency Physical Change Index	F	08-29-2000
TSC-07-01	Head Venting Calculation	G	06-20-2002
TSC-08A-01	Steam Release Data Sheet (Energy Balance)	H	12-14-2001
TSC-08A-02	Steam Release Calculation Sheet (Energy Balance)	G	12-14-2001
TSC-08A-03	Steam Release Data/Calculation Sheet (Open Valve)	E	12-14-2001
TSC-08A-04	Steam Release Data/Calculation Sheet (STMRLS Program)	D	12-14-2001
TSC-09A-01	Core Exit Thermocouple Data	D	05-16-2002
TSC-09A-02	Fuel Rod Clad Damage Estimate	D	05-16-2002
TSC-09A-03	Fuel Rod Overtemperature Damage Estimate	E	05-16-2002
TSC 9A.4	Core Damage Based on Activity Ratios	Deleted	05-16-2002
TSC-09A-05	Core Damage Assessment (Monitoring Data)	E	05-16-2002
TSC 9A.6	Core Damage Summary	Deleted	05-16-2002

WISCONSIN PUBLIC SERVICE CORP. Kewaunee Nuclear Power Plant <i>Emergency Plan Implementing Procedure</i>		No. EPIP-AD-02	Rev. AD		
		Title Emergency Class Determination			
		Date JUL 19 2002	Page 1 of 21		
Reviewed By Dave Lohman		Approved By William Yarosz			
Nuclear Safety Related	<input checked="" type="checkbox"/> Yes <input type="checkbox"/> No	PORC Review Required	<input checked="" type="checkbox"/> Yes <input type="checkbox"/> No	SRO Approval Of Temporary Changes Required	<input checked="" type="checkbox"/> Yes <input type="checkbox"/> No

1.0 Purpose

- 1.1 This procedure provides instruction for determining proper emergency classification listed in order to activate the appropriate level of response from the Kewaunee Nuclear Power Plant (KNPP) emergency response organization and off-site response organization.

2.0 General Notes

- 2.1 None

3.0 Precautions and Limitations

- 3.1 Plant monitors used to determine whether emergency classification levels are being exceeded should be checked for accuracy prior to declaring an emergency class (e.g., compare against redundant channels, determine if consistent with system status, or verification by sample analysis when required by Chart A(1).
- 3.2 This procedure is not written to facilitate de-escalation. Therefore, any decision to de-escalate must be based on a thorough review of procedures and plant conditions. If appropriate, it is preferable to terminate or enter recovery. However, there may be occasions where it is appropriate to de-escalate.
- 3.3 An emergency classification should be made within 15 minutes of recognizing that conditions exist requiring classification in accordance with the EALs. This 15 minute goal is in addition to the 15 minute notification requirement once an emergency declaration has been made on "Event Notice," Form EPIP-AD-07-01. There are times when it may be appropriate to delay classification while significant changes in plant parameters are evaluated for their impact on classification. Examples of such events are an unanticipated:
- Plant Trip
 - SI Initiation
 - Entry into an orange or red path
 - Loss of a safety system

If such an event should occur during classification, it may be appropriate to exceed the 15 minute goal to ensure an accurate classification.

WISCONSIN PUBLIC SERVICE CORP. Kewaunee Nuclear Power Plant <i>Emergency Plan Implementing Procedure</i>	No.	EPIP-AD-02	Rev.	AD
	Title	Emergency Class Determination		
	Date	JUL 19 2002	Page 2 of 21	

4.0 Initial Conditions

- 4.1 This procedure applies during any plant evolution that may result in an emergency declaration.

5.0 Procedure

- 5.1 Determine if a plant emergency exists during abnormal plant conditions by referring to Table 2-1, Emergency Action Level Charts.
- 5.2 IF a plant emergency exists, THEN perform the required actions of the appropriate emergency procedure listed below:
- 5.2.1 EPIP-AD-03, "KNPP Response to an Unusual Event"
- 5.2.2 EPIP-AD-04, "KNPP Response to Alert or Higher"
- 5.3 As plant conditions change, continue referring to the Emergency Action Level Charts.
- 5.4 Determine if the emergency should be reclassified.
- 5.5 IF the event is reclassified, THEN return to Step 5.2.
- 5.6 IF Final Conditions (Section 6.0) are not met, THEN return to Step 5.3.
- 5.7 IF Final Conditions (Section 6.0) are met, THEN use of this procedure may be suspended.

6.0 Final Conditions

- 6.1 Plant Emergency has been Terminated or Recovery actions have begun and the Responsible Director has suspended the use of EIPs.

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7.0 References

- 7.1 Kewaunee Nuclear Power Plant Emergency Plan
- 7.2 EPIP-AD-01, Personnel Response to the Plant Emergency Siren
- 7.3 EPIP-AD-03, KNPP Response to an Unusual Event
- 7.4 EPIP-AD-04, KNPP Response to Alert or Higher
- 7.5 COMTRAK 89-001, NRC Inspection Report 88-11, Improve Guidance for Fires Chart G
- 7.6 OEA 87-246, Report OE 2265, Improve Description of Unusual Aircraft Activity Chart P
- 7.7 NRC Letter 07-11-94, Branch Position on Acceptable Deviations to NUREG-0654

8.0 Records

8.1 The following QA records and non-QA records are identified in this directive/procedure and are listed on the KNPP Records Retention Schedule. These records shall be maintained according to the KNPP Records Management Program.

8.1.1 QA Records

None

8.1.2 Non-QA Records

None

EMERGENCY ACTION LEVEL CHARTS

The following charts are separated into different abnormal operating conditions which may, depending upon their severity, be classified as an Unusual Event, Alert, Site Emergency, or General Emergency.

	CHART	PAGE
Abnormal Radiological Effluent	A (1)	5
Gaseous Effluent Action Levels	A (2)	6 - 8
Fuel Damage Indication	B	9
Primary Leak to LOCA	C	10
Primary to Secondary Leak	D	11
Loss of Power	E	12
Engineered Safety Feature Anomaly	F	13
Loss of Indication	G	14
DELETED	H	14
Secondary Side Anomaly	I	15
Miscellaneous Abnormal Plant Conditions	J	16
Fire and Fire Protection	K	17
DELETED	L	17
Earthquake	M	18
High Winds or Tornado	N	18
Flood, Low Water, or Seiche	O	19
External Events and Chemical Spills	P	20
Security Contingency	Q	21

**CHART A(1)
ABNORMAL RADIOLOGICAL EFFLUENT**

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
SEE CHART A(2)	Effluent monitors detect levels corresponding to greater than 1 rem/hr whole body or 5 rem/hr thyroid at the site boundary under " <u>actual meteorological</u> " conditions.	GENERAL EMERGENCY
Projected or measured dose rates to be provided by the Radiological Protection Director or Environmental Monitoring Teams.	Projected or measured in the environs dose rates greater than 1 rem/hr whole body or 5 rem/hr thyroid at the site boundary.	GENERAL EMERGENCY
SEE CHART A(2)	Effluent monitors detect levels corresponding to greater than 50 mr/hr for ½ hour <u>OR</u> greater than 500 mr/hr for two minutes (or five times these levels to the thyroid) <u>OR</u> for "adverse meteorology."	SITE EMERGENCY
Projected or measured dose rates to be provided by the Radiological Protection Director or Environmental Monitoring Teams.	At the site boundary, projected or measured dose rates greater than 50 mr/hr for ½ hours <u>OR</u> greater than 500 mr/hr for two minutes (or five times these levels to the thyroid) or EPA PAGs are projected to be exceeded outside the site boundary.	SITE EMERGENCY
SEE CHART A(2)	Radiological effluents greater than 10 times ODCM instantaneous limits.	ALERT
a. Containment R-2 OR R-7 $\geq 1.0E+4$ mr/hr, <u>OR</u> b. Charging Area R-4 $\geq 1.0E+4$ mr/hr, <u>OR</u> c. SFP Area R-5 $\geq 1.0E+4$ mr/hr, <u>OR</u> d. Plant area air sample indicates airborne contamination > 1,000 times the occupational DAC values.	Radiation levels or airborne contamination which indicate a severe degradation in the control of radioactive materials (e.g., radiation levels suddenly increase by a factor of 1,000).	ALERT
(1) <u>Gaseous Releases</u> : See Chart A(2) (2) <u>Liquid Releases</u> : Notification by the Rad-Chem Group of violating ODCM 3.3.1 limits.	Offsite Dose Calculation Manual limits exceeded.	UNUSUAL EVENT

CHART A(2) GASEOUS EFFLUENT ACTION LEVELS

1. AUX BUILDING VENT RELEASES - WITH SIGNIFICANT CORE DAMAGE

Instrument readings assuming a post accident gas release AND Containment High Range Radiation Monitors 42599 (R-40) and 42600 (R-41) reads 1000 R/hr within one-half hour of the accident.

NOTE: Use adverse meteorology conditions (ADV MET) only when, 10m and 60m wind speed < 5mph AND Delta-T > +2.4 degrees F or Sigma Theta is < 3.01 degrees. All other cases are average meteorology (AVG MET).

NOTE: R-13 and R-14 are expected to be off scale high during all events on this page.

SV & SFP FANS	AUX BLDG SPING MONITORS				AUX BLDG STACK MONITORS				EMERG. CLASS.
	MID RANGE CPM (01-07) PPCS PT G9086G		HIGH RANGE CPM (01-09) PPCS PT G9088G		R-35 MR/HR		R-36 R/HR		
TOTAL NUMBER RUNNING	AVG MET	ADV MET	AVG MET	ADV MET	AVG MET	ADV MET	AVG MET	ADV MET	
1	**	1.1E+4	6.5E+1	*	**	7.9E+2	1.27E+2	7.9E-1	GENERAL EMERG.
2	8.8E+5	5.5E+3	3.25E+1	*	**	3.9E+2	6.35E+1	4.0E-1	
3	5.9E+5	3.7E+3	2.16E+1	*	**	2.6E+2	4.2E+1	2.6E-1	
4	4.4E+5	2.7E+3	1.62E+1	*	**	2.0E+2	3.175E+1	2.0E-1	

1	8.8E+4	5.5E+2	3.0E+0	*	6.3E+3	3.9E+1	6.3E+0	*	SITE EMERG.
2	4.4E+4	2.7E+2	1.5E+0	*	3.1E+3	1.9E+1	3.1E+0	*	
3	2.9E+4	1.8E+2	1.0E+0	*	2.1E+3	1.3E+1	2.1E+0	*	
4	2.2E+4	1.3E+2	*	*	1.5E+3	9.5E+0	1.5E+0	*	

1	1.0E+3	6.2E+0	*	*	7.0E+1	*	*	*	ALERT
2	5.0E+2	3.1E+0	*	*	3.5E+1	*	*	*	
3	3.3E+2	2.0E+0	*	*	2.3E+1	*	*	*	
4	2.5E+2	1.5E+0	*	*	1.75E+1	*	*	*	

1	1.0E+2	6.2E-1	*	*	7.0E+0	*	*	*	UNUSUAL EVENT
2	5.0E+1	3.1E-1	*	*	3.5E+0	*	*	*	
3	3.3E+1	2.0E-1	*	*	2.3E+0	*	*	*	
4	2.5E+1	1.5E-1	*	*	1.7E+0	*	*	*	

* Offscale Low

** Offscale High (Confirmation Only)

CHART A(2) GASEOUS EFFLUENT ACTION LEVELS continued

2. AUX BUILDING VENT RELEASES WITHOUT CORE DAMAGE

NOTE: Use adverse meteorology conditions (ADV MET) only when, 10m and 60m wind speed < 5mph AND Delta-T > +2.4 degrees F or Sigma Theta is < 3.01 degrees. All other cases are average meteorology (AVG MET).

NOTE: R-13 and R-14 are expected to be off scale high during all events on this page.

SV & SFP FANS	AUX BLDG SPING MONITORS				EMERG. CLASS.
	MID RANGE CPM (01-07) PPCS PT G9086G		HIGH RANGE CPM (01-09) PPCS PT G9088G		
	AVG MET	ADV MET	AVG MET	ADV MET	
1	**	9.4E+4	1.6E+4	1.0E+2	GENERAL EMERG.
2	**	4.7E+4	8.0E+3	5.0E+1	
3	**	3.1E+4	5.3E+3	3.3E+1	
4	**	2.3E+4	4.0E+3	2.5+1	

1	7.5E+5	4.6E+3	8.0E+2	5.0E+0	SITE EMERG.
2	3.7E+5	2.3E+3	4.0E+2	2.5E+0	
3	2.5E+5	1.5+3	2.6E+2	1.6E+0	
4	1.8E+5	1.1E+3	2.0E+2	1.2E+0	

SV & SFP FANS TOTAL NUMBER RUNNING	AUX BLDG SPING MONITORS		EMERG. CLASS.
	LOW RANGE Ci/cc (01-05) PPCS PT G9084G	MID RANGE CPM (01-07) PPCS PT 9086G	
1	**	8.6E+3	ALERT
2	**	4.3E+3	
3	**	2.8E+3	
4	**	2.1E+3	

1	6.3E-2	8.6E+2	UNUSUAL EVENT
2	3.1E-2	4.3E+2	
3	2.1E-2	2.8E+2	
4	1.5E-2	2.1E+2	

** Offscale High (Confirmation Only)

CHART A(2) GASEOUS EFFLUENT ACTION LEVELS continued

3. STEAM LINE RELEASE WITH SIGNIFICANT CORE DAMAGE

Instrument readings assuming radioactive steam is releasing at a total of 1.4E+5 pounds per hour to the atmosphere AND Containment High Range Radiation Monitor 42599 (R-40) or 42600 (R-41) reads 1000 R/hr within one-half hour of the accident.

R-15 (cpm)	"A" Steam Line Monitors		"B" Steam Line Monitors		Emergency Classification
	R-31 (mR/hr)	R-32 (R/hr)	R-33 (mR/hr)	R-34 (R/hr)	
**	1.3E+3	1.3E+0	1.3E+03	1.3E+0	General Emergency
**	6.0E+1	--	6.0E+1	--	Site Emergency
**	1.5E-1	--	1.5E-1	--	Alert
2.0E+05	--	--	--	--	Unusual Event

** Offscale High (Confirmation Only)

4. SHIELD BUILDING STACK RELEASE

Instrument readings assuming SBV System is operating in the recirculation mode.

Reactor Bldg. Discharge Vent SPING		Emergency Classification
PPCS PT G9077G (02-07) Mid Range (cpm)	PPCS PT G9079G (02-09) High Range (cpm)	
1.3E+05	1.5E+2	General Emergency
6.7E+03	7.0E+0	Site Emergency
1.5E+1	--	Alert
--	--	Unusual Event

CHART B FUEL DAMAGE INDICATION

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) CET > 1,200 Degrees for greater than 15 minutes, <u>OR</u> (2) R40 or R41 > 1,000 r/hr, <u>OR</u> (3) SACRG-1, Severe Accident Control Room Guideline Initial Response has been implemented.	Plant conditions exist that make the release of large amounts of radioactivity in a short time period possible.	GENERAL EMERGENCY
(Major damage is more than one spent fuel element damaged.) (1) <u>Fuel Handling accident in Containment</u> a. Alarm on R-11 <u>OR</u> R-12, <u>AND</u> b. Dropped spent fuel assembly, <u>OR</u> c. Report of a large object dropped in Rx core, <u>OR</u> (2) <u>Fuel Handling Accident in Auxiliary Bldg.</u> a. Alarm on R-13 or R-14, <u>AND</u> b. A large object dropped in spent fuel pool, <u>OR</u> c. A dropped spent fuel assembly, <u>OR</u> d. A loss of water level below spent fuel.	Major damage to spent fuel in containment or auxiliary building.	SITE EMERGENCY
(1) R-9 indication is offscale high, <u>AND</u> (2) Laboratory analysis confirms RCS activity levels comparable to USAR Appendix D, Table D.4-1.	<u>Severe loss of fuel cladding</u> a. Very high coolant activity sample b. Failed fuel monitor indicates greater than 1% fuel failures within 30 minutes or 5% total fuel failures.	ALERT
(1) <u>Fuel Handling Accident in Containment</u> a. A confirming report, <u>AND</u> b. Alarm on R-11 <u>OR</u> R-12, <u>OR</u> (2) <u>Fuel Handling Accident in Auxiliary Bldg.</u> a. A confirming report, <u>AND</u> b. Alarm on R-13 <u>OR</u> R-14.	Fuel damage accident with release of radioactivity to containment or auxiliary building.	ALERT
(1) With RCS Temperature > 500°F, a. > 0.2 μCi/gram DOSE Equivalent I-131 for 48 hours, <u>OR</u> b. Exceeding T.S. Figure 3.1-3 for Dose Equivalent I-131, <u>OR</u> c. > 91/Ē μCi/cc As determined by SP-37-065 (from T.S. 3.1.c)	High reactor coolant activity sample.	UNUSUAL EVENT
(1) R-9 is greater than 5.0 R/hr, <u>AND</u> (2) Verified by RCS chemistry sample analysis.	Failed fuel monitor indicates greater than 0.1% equivalent fuel failures within 30 minutes.	UNUSUAL EVENT

CHART C PRIMARY LEAK TO LOCA

NOTE: This chart does not apply when leakage from the Reactor Coolant System is caused by a Steam Generator tube rupture.

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) <u>LOCA</u> is verified per IPEOP E-1, "Loss of Reactor or Secondary Coolant," <u>AND</u> (2) ECCS failure is indicated by: a. SI and RHR pumps not running, <u>OR</u> b. Verification of no flow to the reactor vessel, <u>OR</u> c. Core exit thermocouples indicate greater than 1,200°F, <u>AND</u> (3) Failure or potential failure of containment is indicated by: a. Physical evidence of containment structure damage, <u>OR</u> b. Containment Pressure is > 23 PSIG and loss of all containment fan coil units and both trains of ICS, <u>OR</u> c. Containment hydrogen monitor indicates ≥ 10% hydrogen concentration, <u>OR</u> d. Containment pressure exceeds 46 psig.	(1) Loss of coolant accident, <u>AND</u> (2) Initial or subsequent failure of ECCS, <u>AND</u> (3) Containment failure or potential failure exists (loss of 2 of 3 fission product barriers with a potential loss of 3rd barrier).	GENERAL EMERGENCY
(1) SI System is activated and RCS leakage exceeds charging system capacity as verified by Control Room indications or IPEOPs.	Reactor Coolant System leakage greater than make-up pump capacity.	SITE EMERGENCY
(1) Charging flow verses letdown flow indicates an unisolable RCS leak > 50 gpm.	Reactor Coolant System leak rate greater than 50 GPM.	ALERT
(1) Initiation of reactor shutdown <u>required</u> by Technical Specification, Section T.S. 3.1.d. Indicated leakage may be determined using Reactor Coolant System mass balance calculations performed by SP-36-082.	Exceeding Reactor Coolant System leak rate, Technical Specifications, requiring reactor shutdown.	UNUSUAL EVENT

CHART D PRIMARY TO SECONDARY LEAK

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) Entry into IPEOP E-3, "Steam Generator Tube Rupture," is expected or has occurred, <u>AND</u> (2) Primary-to-secondary flow > 800 GPM OR RCS pressure decreasing uncontrollably, <u>AND</u> (3) All three transformers Main Aux., Reserve Aux., and Tertiary Aux., are de-energized.	Rapid failure of steam generator tubes with loss of off-site power.	SITE EMERGENCY
(1) Entry into IPEOP E-3, "Steam Generator Tube Rupture," is expected or has occurred, <u>AND</u> (2) Primary-to-secondary leak rate > 400 GPM, <u>AND</u> (3) All three transformers: Main Aux., Reserve Aux., and Tertiary Aux., are de-energized.	Rapid gross failure of one steam generator tube with loss of off-site power.	ALERT
(1) Entry into IPEOP E-3, "Steam Generator Tube Rupture," is expected or has occurred, <u>AND</u> (2) Primary-to-secondary leak rate greater than 800 GPM indicated by SI flow <u>OR</u> RWST level change.	Rapid failure of multiple steam generator tubes.	ALERT
(1) Primary-to-secondary leakage > 150 gallons per day for more than 4 hours (TS 3.1.d.2). (Do not delay declaration if leakage suddenly increases above 150 gallons per day <u>AND</u> plant shutdown actions are initiated.)	Exceeding Primary-to-Secondary leak rate Technical Specification.	UNUSUAL EVENT

CHART E LOSS OF POWER

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) RCS is $\geq 350^{\circ}\text{F}$, <u>AND</u> (2) Buses 1 through 6 are de-energized including the D/G supplies to buses 5 and 6, <u>AND</u> (3) Loss of the turbine driven AFW pump, <u>AND</u> (4) Conditions exist for greater than 2 hours.	Failure of off-site and on-site AC power, <u>AND</u> Total loss of auxiliary feedwater makeup capability for greater than 2 hours. (Loss of power plus loss of all AFW would lead to clad failure and potential containment failure.)	GENERAL EMERGENCY
(1) Buses 1 through 6 are de-energized including the D/G supplies to buses 5 and 6 for longer than 15 minutes. (Does not apply when core is unloaded or cavity is flooded with internals removed.)	Loss of off-site power, <u>AND</u> Loss of on-site AC power (for more than 15 minutes).	SITE EMERGENCY
(1) Low voltage lockout <u>OR</u> de-energized condition on all safeguards DC distribution cabinets for greater than 15 minutes. a. BRA 102 and BRB 102, <u>OR</u> b. BRA 104 and BRB 104, <u>OR</u> c. BRA 102 and BRB 104, <u>OR</u> d. BRB 102 and BRA 104 (Does not apply when core is unloaded or cavity is flooded with internals removed.)	Loss of all vital on-site DC power (for more than 15 minutes).	SITE EMERGENCY
(1) Low voltage lockout <u>OR</u> de-energized condition on all safeguards DC distribution cabinets for less than 15 minutes. a. BRA 102 and BRB 102, <u>OR</u> b. BRA 104 and BRB 104, <u>OR</u> c. BRA 102 and BRB 104, <u>OR</u> d. BRB 102 and BRA 104 (Does not apply when core is unloaded or cavity is flooded with internals removed.)	Loss of all vital on-site DC power (for less than 15 minutes).	ALERT
(1) Buses 1 through 6 are de-energized, <u>AND</u> (2) The D/G supplies to buses 5 and 6 do not respond as designed. AC power is restored to bus 5 or 6 within 15 minutes. (Does not apply when core is unloaded or cavity is flooded with internals removed.)	Loss of off-site power, <u>AND</u> Loss of on-site AC power (for less than 15 minutes.)	ALERT
(1) With the Reactor Coolant System above cold shutdown condition: a. All three transformers: Main Aux., Reserve Aux., and Tertiary are de-energized, <u>OR</u> b. Both D/Gs unavailable (unable to supply bus 5 or 6 by any means).	Loss of off-site power, <u>OR</u> Loss of on-site power capability.	UNUSUAL EVENT
(1) Core is unloaded or reactor cavity is flooded with internals removed, <u>AND</u> (2) Buses 1 through 6 are de-energized including the D/G supplies to buses 5 and 6 for longer than 15 minutes.	Loss of off-site power, <u>AND</u> Loss of on-site AC power (for more than 15 minutes).	UNUSUAL EVENT

CHART F ENGINEERED SAFETY FEATURE ANOMALY

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
<p>(1) RCS > 200°F with a loss of cooling capability or inventory control:</p> <p>a. Loss of negative reactivity control, <u>OR</u></p> <p>b. Steam dump, S/G safeties, and power operating reliefs not operable (> 350°F), <u>OR</u></p> <p>c. Inability to feed S/Gs (No AFW or Main Feedwater/Condensate Flow), <u>OR</u></p> <p>d. Loss of RCS inventory control, <u>OR</u></p> <p>e. Loss of both trains of RHR, <u>AND</u></p> <p>the inability to sustain either natural <u>OR</u> forced circulation with the steam generators ($\leq 350^\circ\text{F}$).</p> <p>(A Site Emergency should be declared upon the initiation of bleed and feed per FR H.1, "Response to Loss of Secondary Heat Sink.")</p>	<p>Complete loss of any function needed when RCS > 200°F.</p>	<p>SITE EMERGENCY</p>
<p>(Apply this criteria when the RCS is $\leq 200^\circ\text{F}$.)</p> <p>(1) Loss of both trains of RHR</p> <p>(Does not apply when core is unloaded <u>OR</u> cavity is flooded with internals removed.)</p>	<p>Complete loss of any function needed when RCS $\leq 200^\circ\text{F}$.</p>	<p>ALERT</p>
<p>(1) Failure of both Rx trip breakers to open upon receipt of a valid signal. Applies even if IPEOP FR S.1 is not entered.</p>	<p>Failure of the Reactor Protection System to initiate and complete a reactor trip which brings the reactor subcritical.</p>	<p>ALERT</p>
<p>(1) Loss of ESF function, required support function or required Tech Spec instruments <u>OR</u> Exceeding Tech Spec Safety Limits, <u>AND</u></p> <p>(2) upon discovery, inability or failure to take required shutdown or mode change actions within the required time.</p> <p>(Total loss of AFW system when required (FR-H.1 implemented) should be declared a UE regardless of Tech Spec action compliance.)</p>	<p>Inability to reach required shutdown within Tech Spec limits</p>	<p>UNUSUAL EVENT</p>

**CHART G
LOSS OF INDICATION**

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) Total loss of Annunciator System computer alarms, and sequence of events recorder for greater than 15 minutes, <u>AND</u> (2) Uncontrolled plant transient in progress or initiated during the loss.	Most or all alarms (annunciators) lost and a plant transient initiated or in progress.	SITE EMERGENCY
(1) Total loss of Annunciator System, computer alarms, and sequence of events recorder. (Not applicable when plant is at or below cold shutdown.)	Most or all alarms (annunciators) lost.	ALERT
(1) Significant loss of ESF or Rx Protection instrumentation. An Unusual Event should <u>NOT</u> be declared for a non-emergency Tech Spec backdown, when the affected parameter remains monitorable. (Not applicable when plant is at or below cold shutdown.)	Indications or alarms on process or effluent parameters not functional in control room to an extent requiring plant shutdown or other significant loss of assessment capability.	UNUSUAL EVENT

**CHART H
(DELETED)**

CHART I SECONDARY SIDE ANOMALY

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) Main steam line break that results in a SI actuation, <u>AND</u> (2) a. R-15 or R-19 reads offscale high with confirmation by chemistry analysis, <u>OR</u> b. Primary-to-secondary leakage > 50 gpm, <u>AND</u> (3) a. R-9 or CNTMT high range rad monitors (42599, 42600) indicate > 10 R/hr, <u>OR</u> b. CNTMT hydrogen monitor indicates > 1% hydrogen concentration.	Steam line break, <u>AND</u> Primary-to-secondary leak > 50 GPM, <u>AND</u> Indication of Fuel Damage.	SITE EMERGENCY
(1) Main steam line break that results in a SI actuation, <u>AND</u> a. R-15 <u>OR</u> R-19 reads a factor of 1000 above normal, <u>OR</u> b. Primary-to-secondary leakage > 10 gpm.	Steam line break with significant (greater than 10 GPM) primary-to-secondary leakage. (Applies even if events occur in opposite steam generators.)	ALERT
(1) Turbine trip and observation of penetration of casing.	Turbine rotating component failure causing rapid plant shutdown.	UNUSUAL EVENT
(1) The uncontrolled depressurization of the secondary system that results in an SI actuation.	Rapid depressurization of the secondary side.	UNUSUAL EVENT

CHART J MISCELLANEOUS ABNORMAL PLANT CONDITIONS

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
<p>(1) Containment boundary failure or potential failure:</p> <ul style="list-style-type: none"> a. Containment pressure > 46 psig, <u>OR</u> b. Loss of all containment fan coil units and both trains of ICS, <u>OR</u> c. Containment hydrogen monitor \geq 10% hydrogen concentration, <u>AND</u> <p>(2) Loss of core cooling capability:</p> <ul style="list-style-type: none"> a. Loss of SI and RHR flow, <u>AND</u> <p>(3) Failure of shutdown system when required:</p> <ul style="list-style-type: none"> a. Entry into IPEOP FR-S.1, "Response to Nuclear Power Generation/ATWS," <u>OR</u> b. Loss of AFW for greater than 30 minutes with loss of main FW and condensate. 	<p>Other plant conditions that make a release of large amounts of radioactivity in a short time period possible; e.g., any core melt situation.</p> <p>Examples:</p> <ul style="list-style-type: none"> - Failure of main FW and AFW systems for greater than 30 minutes without Safety Injection and Residual Heat Removal flow. Plus a containment failure is imminent. - Transient requiring the operation of shutdown systems with a failure of these shutdown systems. In addition, failure of SI and RHR and containment failure is imminent. 	GENERAL EMERGENCY
<p>(1) Evacuation of Control Room (E-O-06 event).</p>	<p>Evacuation of control room and control of shutdown systems required from local stations.</p>	SITE EMERGENCY
<p>(1) Conditions that warrant increased awareness on part of the plant staff will be evaluated by the Plant Manager or his designate. This is to determine if conditions are applicable for activating the E.P.</p> <p><u>Example:</u> Loss of AFW system when required, validated upon implementation of FR H.1 "Response to Loss of Secondary Heat Sink."</p>	<p>Other plant conditions that warrant increased awareness on the part of plant staff or state and/or local authorities.</p>	UNUSUAL EVENT

CHART K FIRE AND FIRE PROTECTION

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) A fire within the Auxiliary Building, Technical Support Center, safeguards alley, D/G rooms, Battery Rooms, or screenhouse that defeats redundant safety trains of ESF equipment causing the required ESF system to be inoperable.	A fire compromising the functions of safety systems.	SITE EMERGENCY
(1) A fire within the Auxiliary Building, Technical Support Center, safeguards alley, D/G rooms, Battery Rooms, or screenhouse that lasts more than 10 minutes <u>OR</u> causes a single train of required ESF equipment to be inoperable.	A fire potentially affecting safety systems.	ALERT
(1) Any fire within the protected area lasting more than 10 minutes.	A fire within the plant lasting more than 10 minutes.	UNUSUAL EVENT

CHART L (DELETED)

CHART M EARTHQUAKE

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) Activation of seismic recorder with TRIGGER, OBE, and DBE lights lit in relay room on RR159, <u>AND</u> (2) Verification of a seismic event by physical experience or from U. of W. - Milwaukee Seismic Center.	An earthquake greater than Design Basis Earthquake (DBE).	SITE EMERGENCY
(1) Activation of seismic recorder with TRIGGER, and OBE lights lit in relay room on RR159, <u>AND</u> (2) Verification of a seismic event by physical experience or from U. of W. - Milwaukee Seismic Center.	An earthquake greater than Operational Basis Earthquake (OBE).	ALERT
(1) Activation of seismic recorder with TRIGGER light lit in relay room on RR159, <u>OR</u> (2) An earthquake felt in the Plant*. (*Should be confirmed by evidence of physical damage or verification from University of Wisconsin Seismic Center.)	An earthquake felt in plant or detected on station seismic instrumentation.	UNUSUAL EVENT

NOTE: Telephone numbers for U of W - Milwaukee Seismic Center are in the KPB Emergency Telephone Directory, ETD 02.

CHART N HIGH WINDS OR TORNADO

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) Winds in excess of 100 mph for greater than 1 hour, <u>AND</u> (2) Plant above cold shutdown condition.	Sustained winds in excess of design levels with plant not in cold shutdown.	SITE EMERGENCY
(1) A tornado which strikes the facility, <u>AND</u> (2) Causes damage to render a single train of required ESF equipment to be inoperable.	Any tornado striking facility.	ALERT
(1) A tornado observed on-site causing significant damage to the facility.	Any tornado on-site.	UNUSUAL EVENT

CHART O FLOOD, LOW WATER, OR SEICHE

KNPP INDICATION				EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
FOREBAY LEVEL Indicated for > 15 minutes				Flood, low water, or seiche near design levels.	ALERT
0 PUMPS	1 PUMP	2 PUMPS	CORRESPOND TO LAKE LEVEL		
NOTE 3	NOTE 1	≥ 94% *	≥ 588 ft.		
< 50% *	NOTE 5	NOTE 5	< 568.5 ft.		
<u>OR</u> Deep water Wave ≥ 22.5 ft.					
FOREBAY LEVEL Indicated for > 15 minutes				50-year flood, low water level or seiche	UNUSUAL EVENT
0 PUMPS	1 PUMP	2 PUMPS	CORRESPOND TO LAKE LEVEL		
NOTE 2	≥ 98% *	≥ 88% *	≥ 586 ft.		
< 53.1% *	< 46.9% * NOTE 4	NOTE 5	< 569.5 ft.		
<u>OR</u> Deep water wave ≥ 18 ft. (as confirmed by the U.S. Coast Guard, Two Rivers)					

NOTE 1: Above the bottom of bar No. 1 painted on the south wall of the forebay.

NOTE 2: Above the bottom of bar No. 2 painted on the south wall of the forebay.

NOTE 3: Above the bottom of bar No. 3 painted on the south wall of the forebay.

NOTE 4: Applies to an uncontrollable decrease (cannot be restored by operator action. If the water box inlet valves are throttled, use other means to determine lake level per E-CW-04, "Loss of Circulating Water.")

NOTE 5: The corresponding forebay level for the associated lake level is below the circulating water pump trip setpoint of 42%. Therefore, this criterion will not be reached.

* Computer point for forebay level is L09075A and should be used because of its greater accuracy. Plant elevations and lake elevations are referenced to International Great Lakes Datum (IGLD), 1955

(IGLD 1955 = IGLD 1985 - 0.7 FEET)

CHART P EXTERNAL EVENTS AND CHEMICAL SPILLS

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) An aircraft crash into plant buildings which causes a complete loss of an ESF function.	Aircraft crash affecting vital structures by impact <u>OR</u> fire.	SITE EMERGENCY
(1) A missile strikes plant buildings, <u>OR</u> (2) An explosion occurs within a plant building, which causes a complete loss of an ESF function.	Severe damage to safe shutdown equipment from missiles or explosion.	SITE EMERGENCY
(1) Release of flammable or toxic gas from a ruptured container, which causes or is likely to cause evacuation of stations necessary to control shutdown systems, <u>AND</u> (2) Portable monitors indicate toxic or explosive concentrations of the gas at life threatening levels in those vital areas.	Uncontrolled release of toxic or flammable gas is confirmed within vital area.	SITE EMERGENCY
(1) An aircraft crashes into plant buildings <u>AND</u> causes a single train of required ESF equipment to be inoperable.	Aircraft crash on facility.	ALERT
(1) A missile strikes the facility <u>AND</u> causes a single train of required ESF equipment to be inoperable.	Missile impact from whatever source on facility.	ALERT
(1) Release of toxic or flammable gas at life threatening levels from a ruptured container enter the protected area <u>AND</u> impacts safe operation of the plant.	Uncontrolled release of toxic or flammable gas is confirmed within the protected area.	ALERT
(1) Self-explanatory.	Known explosion damage to facility affecting plant operation.	ALERT
(1) An aircraft crash within the site boundary, <u>OR</u> (2) Unusual aircraft activity such as erratic flying, dropped unidentified object, or other hostile acts, which threaten the plant or plant personnel. (Any other persistent aircraft activity for which identification attempts through the FAA or other agencies have been unsuccessful.)	Aircraft crash on-site or unusual aircraft activity over facility.	UNUSUAL EVENT
(1) Release of toxic or flammable gas from a ruptured tank/truck on site, <u>AND</u> (2) Portable monitors indicate toxic or explosive concentrations at life threatening levels of the gas near the spill area.	Uncontrolled release of toxic or flammable gas is confirmed on site.	UNUSUAL EVENT

CHART Q SECURITY CONTINGENCY

KNPP INDICATION	EMERGENCY CLASSIFICATION CRITERIA	CLASSIFICATION
(1) Physical attack on the plant that has resulted in unauthorized personnel occupying the control room or any other vital areas as described in the Security Plan.	Loss of physical control of the plant.	GENERAL EMERGENCY
(1) Physical attack on the plant involving imminent occupancy of the control room, auxiliary shutdown panels, or other vital areas as defined by the Security Plan.	Imminent loss of physical control of the plant.	SITE EMERGENCY
(1) Security safeguards contingency event that results in a hostile force entering the protected area of the plant, but not gaining control over shutdown capability or of any vital areas as defined in the Security Plan, <u>OR</u> (2) Security safeguards contingency event that results in a site specific HI level CREDIBLE threat as defined in the Security Plan.	Ongoing security compromise.	ALERT
(1) Security safeguards contingency event that results in a site specific LO level CREDIBLE threat as defined in the Security Plan, <u>OR</u> (2) Security safeguards contingency event that results in a Bomb threat accompanied by interception of bomb materials, <u>OR</u> (3) Security safeguards contingency event that results in an attempted entry into the protected area of the plant by a hostile force, <u>OR</u> (4) Security safeguards contingency event that results in undetonated bomb found within the protected area.	Security threat or attempted entry or attempted sabotage.	UNUSUAL EVENT

NOTE: Security staff will NOT act as notifier during **security events**. Utilize Control Room staff for notifications.

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Reviewed By Dave Seebart		Approved By William Yarosz	
Nuclear Safety Related	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	PORC Review Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No
		SRO Approval Of Temporary Changes Required	<input type="checkbox"/> Yes <input checked="" type="checkbox"/> No

1.0 Purpose

- 1.1 This procedure provides instruction for establishing and maintaining personnel accountability within the Protected Area (PA) of the plant.

2.0 General Notes

- 2.1 When initial assembly and accountability are in progress, certain individuals (as designated in EPIP-AD-01, "Personnel Response to the Plant Emergency Siren") will be allowed to move in, out, and about the PA before initial accountability is complete.
- 2.2 Emergency Response Organization (ERO) staff shall report to their duty locations.
- 2.3 Visitors, Contractors, and non-ERO personnel on-site should assemble in the nearest designated Assembly Area.
- 2.4 Severe Weather
- 2.4.1 Several of the Assembly Areas are NOT appropriate for severe weather safety.
- Warehouse Annex Lunchroom - relocate to the locker room.
 - Administrative Training Facility (ATF) Lunchroom - relocate to the ATF Basement.
 - Simulator Training Facility (STF) Lobby - relocate to the Simulator Control Room away from the glass partition along the west wall close to the floor.
 - Classroom C Security Building - relocate to the Security Building locker room.

Note

IF unable to relocate as stated above, personnel should relocate on lower levels of buildings in interior rooms away from all windows.

- 2.4.2 The following locations are adequate during severe weather:
- Control Room (CR)
 - Radiation Protection Office (RPO)
 - Technical Support Center (TSC)

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3.0 Precautions and Limitations

- 3.1 Initial **accountability starts** from the site announcement (siren sounding/public address message) and needs to be **completed within 30 minutes**.
- a. The SMS Emergency Accountability **Program will begin 2 minutes** after site announcement. This allows for personnel to exit areas that are not their normal emergency work area (i.e., Control Room).
 - b. The first report should be **generated after 5 minutes**.
 - c. **After 10 minutes**, print a report or review the computer screen every minute until the number of unaccounted for personnel plateaus.
 - d. When the number of unaccounted for personnel plateaus, **accountability is complete**.
- 3.2 Allow prompt movement of ERO personnel into and out of the Protected Area when the Site Protection Director has determined there is no personnel hazard restricting such movement.
- 3.3 Adhere to the radiation control policies and requirements outlined in EPIP-AD-11, "Emergency Radiation Controls," and EPIP-RET-02D, "Emergency Radiation Entry Controls and Implementation."
- 3.4 Personnel who are in the Control Room when the Emergency Accountability Program is started are accounted for by the SMS.
- 3.5 Security Force Members, Fire Brigade Members, NAOs, and dispatched Emergency Teams are exempt from the SMS Emergency Accountability Program. They will be accounted for as defined in EPIP-AD-01, Step 5.1.1.c.

4.0 Initial Conditions

- 4.1 Personnel **assembly** is required whenever the plant emergency siren is sounded. **Accountability** will be required upon assembly unless otherwise directed by the Shift Manager/Emergency Director.

5.0 Procedure

- 5.1 **Site Protection Director (SPD) or Designee shall:**
- 5.1.1 **IF** a designated Master Accountability Coordinator (MAC) is not available, **THEN** direct the Security Shift Captain to perform accountability functions.
 - 5.1.2 Contact the Emergency Director (ED) or Radiological Protection Director (RPD) to determine the location of any personnel hazards on-site or off-site and continue to monitor conditions.

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- 5.1.3 IF conditions warrant, THEN direct the Security Force to hold exiting personnel in the Security Building until the RPD determines a safe evacuation route.
- 5.1.4 Advise the MAC of all hazardous areas and/or severe weather.
- 5.1.5 Advise on-site directors of hazardous areas or severe weather conditions.
- 5.1.6 Direct Security Force personnel to make a tour through the Owner Controlled Area (OCA), sewage plant, exterior warehouses, exterior substation, and Met Tower to inform personnel to assemble in the STF or Security Building, or to leave the site, as appropriate.
- 5.1.7 Ensure attempts are made by any available means to contact all personnel who are unaccounted for.
- 5.1.8 IF attempts fail to locate personnel who are unaccounted for, THEN direct the Central Alarm Station (CAS) or Secondary Alarm Station (SAS) operator to run a computer report for that person(s).
- 5.1.9 Provide the Support Activities Director (SAD) and the RPD with information regarding all personnel who are unaccounted for based upon last known locations obtained from the computer report.
- 5.1.10 Keep the ED informed of the status of personnel accountability.
- 5.1.11 When initial accountability is complete, ensure the following message is announced over the Gai-tronics:

“Attention all personnel. Initial accountability is complete. Personnel may relocate to other areas but shall maintain accountability.”

- 5.1.12 Maintain cognizance of all personnel outside the PA who remain on-site.
- 5.1.13 Continue to provide the MAC with updates on any location determined to be a personnel hazard including areas outside the PA.

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5.2 **Master Accountability Coordinator (MAC) shall:**

5.2.1 **WHEN** accountability is required:

- a. Report to the Security Building.
- b. **IF** a MAC has been designated and **UNTIL** they are released,
 1. If appropriate, plan a shift relief per EPIP-AD-05, "Emergency Response Organization Shift Relief Guideline."
 2. Help the designated MAC.
- c. **IF** a MAC has **NOT** been designated, notify the Site Protection Director (SPD) of your intent to assume the responsibilities of the MAC and continue implementation of this procedure.
- d. Assign an Accountability Coordinator to the South Security Building Visitor's Desk.
 1. Inform this Accountability Coordinator to complete Form EPIP-SEC-03.01, "Emergency Accountability Log," for ERO members entering the Protected Area.
 2. Implement Step 5.3.8 of this procedure.
- e. **IF** there is no Accountability Coordinator available for the Security Visitor's Desk, **THEN** assign a plant staff or security staff member to perform Step 5.2.1.d.

5.2.2 After the sound of the plant emergency siren:

- a. Contact CAS to ensure SAS/CAS has started the "Emergency Accountability Program" on the SMS.
- b. Make a public address announcement that the program has been activated and personnel in accountability areas should now swipe.

5.2.3 Contact Security to verify there is **NOT** a hard copy visitor list. If this list is available, pick up the KNPP Protected Area Visitor and Non-Designated Vehicle log (SIP 20.02-1) at the Registration Desk.

5.2.4 Contact each Visitor's Escort via Gai-tronics to ensure accountability of their Visitor, place a check mark next to the Visitor's name as reported by the Escort.

5.2.5 Four minutes after siren activation for a declared emergency, make a public address announcement that personnel who have not swiped a card reader must do so immediately.

5.2.6 Five minutes after the SMS Emergency Program has been started, obtain an Emergency Report from CAS or SAS.

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- 5.2.7 Should attempts fail to locate personnel who are unaccounted for, direct the CAS or SAS to run a computer report on that person.
- 5.2.8 Review the Emergency Report
- a. Contact the Control Room and strike any names from the list that have been positively accounted for (individual is in the Control Room or is accounted for on their "Emergency Accountability Log").
 - b. Contact the Technical Support Center (TSC) and strike any names from the list that have been positively accounted for (individual is in the TSC or is accounted for on their "Emergency Accountability Log").
 - c. Contact all names remaining on the list by any possible means.
- 5.2.9 WHEN all personnel have been contacted or reasonable attempts to contact have failed, record the names of individuals not accounted for and their last known location and provide this information to the SPD.
- 5.2.10 Contact the Accountability Areas outside the PA (per Table 1) to determine the number and location of personnel on-site but not within the Protected Area.
- 5.2.11 Report to the SPD the number and location of personnel on-site but outside the PA and obtain an update regarding personnel hazards and/or severe weather.
- 5.2.12 IF the plant emergency siren sounds after initial accountability, THEN return to Step 5.2.2.
- 5.2.13 Provide Accountability Coordinators (AC) with information regarding personnel hazards and/or severe weather provided by the SPD and continue to do so on a periodic basis.
- 5.2.14 Brief the SPD on any change in accountability status or new problems (i.e., groups or individuals overdue in assembly areas).
- 5.2.15 If appropriate, plan a shift relief for the MAC per EPIP-AD-05.
- 5.2.16 IF Final Conditions (Section 6.0) have NOT been met, THEN return to Step 5.2.12.

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5.2.17 WHEN Final Conditions (Section 6.0) are met,

- a. Return your work area to normal status and return emergency procedures, forms, etc. to their proper place.
- b. Record any discrepancies to emergency supplies that need correction and report them to the SPD.
- c. Collect all completed forms, notes, and other documentation and give them to the SPD.

5.3 **Accountability Coordinator (AC) shall:**

5.3.1 WHEN accountability is required:

- a. Report to your assigned assembly area.
- b. IF an AC has NOT been designated, THEN notify a Facility Director or MAC of your intent to assume the responsibilities of the AC in the assembly area and continue implementation of this procedure.
- c. IF an AC has been designated and UNTIL they are released,
 1. Help in facility activation.
 2. If appropriate, plan a shift relief per EPIP-AD-05.
 3. Help the designated AC.

5.3.2 Obtain Accountability packet located in the assembly areas in the following locations:

- a. Control Room (CR) - On the wall behind the Control Room Supervisor's desk.

Note

Make a list for the MAC of Fire Brigade Members, NAOs, and dispatched Emergency Teams that are accountability exempt.

- b. Technical Support Center (TSC) - On the desk near the entrance door (No. 410).
- c. Radiation Protection Office (RPO) - Next to the entrance door (No. 55).
- d. Administrative Training Facility (ATF) - Lunchroom southwest corner, above the phone and Gai-tronics.
- e. Warehouse Annex - Lunchroom west wall, above the phone and Gai-tronics.
- f. Security Building - Room "C" on the wall by the phone and Gai-tronics.
- g. Site Training Facility (STF) - Lobby on the wall by the Gai-tronics.

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5.3.3 Locate yourself near the SMS accountability card reader.

Note

*Visitors have non-magnetic badges, so they should **NOT** swipe their key card in an accountability card reader. Log visitors and report their information to the MAC.*

5.3.4 **WHEN** the Gai-tronics announcement is made that emergency accountability has been initiated, ensure that all badged personnel located in the area swipe their Security key card in the accountability reader and receive a green light.

5.3.5 **IF** a Director or Supervisor notifies you of individuals who will **NOT** assemble, **THEN** record them as if they were leaving the facility per Step 5.3.8.

5.3.6 During accountability, demand quiet and cooperation. Maintain accountability of personnel assembled there.

5.3.7 **WHEN** provided by the MAC, inform personnel in your assembly area of information on hazardous areas.

Note

Only address the teams or individuals you logged out.

5.3.8 **WHEN** an individual elects to move to another area, log their departure on "Emergency Accountability Log," Form EPIPF-SEC-03-01, as follows:

Note

This includes visitors also.

- a. In the "Name" column: Enter the name of the individual leaving the area.
- b. In the "Card" column: Enter the individual's security card number.

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- c. In the "Destination"-column: Enter the location to which the individual is heading.

Acronym	Area
<i>For Another Accountability Area</i>	
CR	Control Room
RPO	Radiation Protection Office
TSC	Technical Support Center
WA	Warehouse Annex Lunchroom
ATF	Administrative Training Facility Lunchroom
<i>For Repair Team Activity</i>	
LPA	Leaving the Protected Area
PA	Outside the Buildings but inside the Protected Area
ADMN	Administrative Buildings
TURB	Turbine Building
AUX	Auxiliary Building
CNTM	Containment Building
WRHS	Warehouse or Shop Area

- d. In the "Departure Time" column: Enter the time the individual departed the area.
- e. In the "Return/Arrive Time" column: Enter the time the individual returns to the area or the time you are informed they arrive at another Assembly Area by the Accountability Coordinator in that area.

5.3.9 IF an individual arrives in your area who did not log out of your area and is not immediately returning to the area he checked out of, THEN contact the area he departed and notify them of the individual's arrival time and intent to stay.

5.3.10 IF moving the entire group of assembled personnel to another area, THEN:

- a. Contact the MAC to inform them of the intended move.
- b. Record badge numbers of people moving.
- c. Instruct personnel in your area to proceed directly to the new destination, staying in a group until head count can be verified.

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- d. Verify the badge numbers upon arrival at the new location.
- e. Contact the MAC to verify arrival and to provide a phone number at the new location.

5.3.11 **IF** the plant siren is sounded following the initial accountability, **THEN** initiate contacts to all individuals listed as **NOT** "Returned or Arrived" on your Form EPIPF-SEC-03-01:

- a. **WHEN** contacted, give them any information you know about the siren sounding.
- b. **IF** instructed by a Facility Director, instruct the individual to come back to an assembly area.
- c. Note the individuals that were contacted and notify the MAC of individuals logged out of your area and whether you were able to contact them.
- d. Return to Step 5.3.4.

5.3.12 If appropriate, plan a shift relief for the area AC per EPIP-AD-05.

5.3.13 **IF** Final Conditions (Section 6.0) have **NOT** been met, **THEN** return to Step 5.3.7.

5.3.14 **IF** Final Conditions (Section 6.0) are met, **THEN**

Note

Flag any discrepancies to emergency supplies that need correction.

- a. Return your work area to normal status and return emergency procedures, forms, etc. to their proper place.
- b. Collect all completed forms, notes, and other documentation and give them to the SPD.

5.4 **Security Force shall:**

5.4.1 **WHEN** accountability is required and the SMS system is offline, manually maintain a log of personnel entering/exiting the PA to enable the MAC to update the accountability roster as necessary.

5.4.2 Guide PA ingress/exit in accordance with EPIP-SEC-02, "Security Force Response to Emergencies."

5.4.3 **Patrolling Security Officers shall:**

- a. Verify their locations by portable radio to the Shift Captain for accountability.
- b. Obtain personal dosimetry to take with them on patrol in accordance with EPIP-SEC-04, "Security Force Actions for Dosimetry Issue."

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6.0 Final Conditions

- 6.1 Plant Emergency has been Terminated or Recovery actions have begun and the Emergency Director has suspended the use of EPIPs or it has been determined emergency accountability is no longer required.

7.0 References

- 7.1 Kewaunee Nuclear Power Plant Emergency Plan
- 7.2 EPIP-AD-01, Personnel Response to the Plant Emergency Siren
- 7.3 EPIP-AD-05, Emergency Response Organization Shift Relief Guideline
- 7.4 EPIP-AD-11, Emergency Radiation Controls
- 7.5 EPIP-RET-02D, Emergency Radiation Entry Controls and Implementation
- 7.6 EPIP-SEC-02, Security Force Response to Emergencies
- 7.7 EPIP-SEC-04, Security Force Actions for Dosimetry Issue

8.0 Records

- 8.1 The following QA records and non-QA records are identified in this directive/procedure and are listed on the KNPP Records Retention Schedule. These records shall be maintained according to the KNPP Records Management Program.

8.1.1 QA Records

None

8.1.2 Non-QA Records

- Emergency Accountability Log, Form EPIPF-SEC-03-01

**ACCOUNTABILITY AREAS
(Inside the Protected Area)**

LOCATION	PHONE NUMBER
Control Room	8207
Radiation Protection Office	8451
Technical Support Center (Included OSF and RAF)	8353
Administrative Training Facility (Lunchroom)	6480
Warehouse Annex (Lunchroom)	6452

ALTERNATE LOCATION	PHONE NUMBER
Administrative Training Facility (Basement)	6548
Warehouse Annex (Locker Room)	6454

**ACCOUNTABILITY AREAS
(Outside the Protected Area)**

LOCATION	PHONE NUMBER
Security Building Classroom C	6548
Simulator Training Facility (Lobby)	8427

ALTERNATE LOCATION	PHONE NUMBER
Security Locker Room (Security Building)	8292
Simulator Training Facility (Simulator Control Room)	8607

ACCOUNTABILITY LEADERS

LEADER	PHONE NUMBER
Master Accountability Coordinator (Security Building)	8509
Accountability Coordinator (Sec. Bldg. Visitor's Desk)	8289
Site Protection Director (Sec. Bldg.)	8418
(TSC)	PBX 8591 Kew. Ex. 388-0459

Friday, July 19, 2002

Attention:
NRC Document Control Desk

The purpose of this letter is to inform you that we have not yet received the Document Transmittal Sheet for revisions of procedures made in your copy of the Emergency Plan Implementing Procedures (EPIP) manual.

We are currently performing an audit for all manuals that are distributed from Document Control at Kewaunee Nuclear Power Plant. Since we have not received receipts from you in some time, we are enclosing an index of the Emergency Plan Implementing Procedures (EPIP) Manual. Please use this index to ensure that your manual is up to date. If you find any discrepancies in this manual, please contact me, Laura Ihlenfeldt at (920)-388-8881, or Yvonne Elsen at (920)-388-8136 and we will ensure that you receive the correct updates.

Please complete this audit and sign, date, and return this to me within 30 days, or your manuals will be cancelled.

Thank you in advance for your help!

Yvonne Elsen
Document Control Supervisor
388-8136

Laura Ihlenfeldt
388-8881

Signature: _____ Date: _____

EMERGENCY PLAN IMPLEMENTING PROCEDURES

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EPIP-AD-03	KNPP Response to an Unusual Event	AF	06-20-2002
EPIP-AD-04	KNPP Response to Alert or Higher	AJ	06-20-2002
EP-AD-5	Site Emergency	Deleted	04-27-87
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EP-AD-6	General Emergency	Deleted	04-24-87
EPIP-AD-07	Initial Emergency Notifications	AR	06-20-2002
EP-AD-8	Notification of Alert or Higher	Deleted	02-26-96
EP-AD-9	Notification of Site Emergency	Deleted	04-27-87
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EP-AD-12	Personnel Assembly and Accountability	Deleted	03-26-94
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EP-ENV-3A	Environmental Protection Director Actions and Directives	Deleted	09-26-84
EP-ENV-3B	EM Team Actions	Deleted	09-26-84
EPIP-ENV-03C	Dose Projection Using RASCAL Version 2.2 Software	V	10-09-2001
EP-ENV-3D	Revision and Control of ISODOSE II	Deleted	02-14-95
EP-ENV-3E	Manual Determination of X/Q	Deleted	04-24-87
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EP-ENV-3G	Manual Dose Projection Calculation	Deleted	06-02-89
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EPIP-ENV-04B	Air Sampling and Analysis	W	10-09-2001
EP-ENV-4C	Environmental Monitoring Teams	Deleted	04-13-90
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EPIP-ENV-04D	Plume Tracking for Environmental Monitoring Teams	N	10-02-2001
EP-ENV-5A	LCS-1 Operation	Deleted	04-14-86
EP-ENV-5B	MS-3 Operation	Deleted	04-14-86
EP-ENV-5C	SAM II Operation	Deleted	04-14-86
EP-ENV-5D	PAC-4G (Alpha Counter) Operation	Deleted	04-14-86
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EP-ENV-6B	SAF Environmental Sample Analysis Relocation	Deleted	03-23-84
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EP-EOF-5	Corporate Staff Action for Site Emergency	Deleted	04-24-87
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EP-EOF-9	Interface with Support Organizations	Deleted	03-05-84
EP-EOF-9	Notification of Site Emergency	Deleted	04-24-87
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EPIP-EOF-11	Internal Communication and Documentation Flow	U	11-15-2001
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EPIP-RET-03A	Liquid Effluent Release Paths	L	11-29-2001
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EPIP-TSC-04	Emergency Physical Changes, Major Equipment Repair	N	05-09-2002
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EP-TSC-6	Assessment of Reactor Core Damage	Deleted	09-30-86
EPIP-TSC-07	RV Head Venting Time Calculation	J	06-20-2002
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* EP-TSC-8B was totally deleted; therefore, EP-TSC-8C was changed to EP-TSC-8B			
EP-TSC-9	Core Damage Assessment Using Released Radionuclides	Deleted	09-30-86
EPIP-TSC-09A*	Core Damage Assessment	J	05-16-2002
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* EP-TSC-9A, Rev. D was totally deleted; therefore, EP-TSC-9B became EP-TSC-9A. EP-TSC-9B was previously EP-TSC-9C.			
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