## PROPOSED NRC-AUTHORIZED WALKTHROUGH JPMS

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## FOR THE LASALLE INITIAL EXAMINATION - APRIL 2002

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## LASALLE COUNTY NUCLEAR STATION

## JOB PERFORMANCE MEASURE

Secure RHR Service Water System

Examination Level: RO /SRO /SRO(I)

Date: April 8, 2002

Developed by: <u>Raymond Keith Walton</u> Date: <u>March 14, 2002</u>

Reviewed by: Date: \_\_\_\_\_ Approved by: \_\_\_\_\_ Date:\_\_\_\_

Facility: LaSalle Nuclear Station	Task No: <u>B.1.a</u>
Task Title: Secure RHR Service Water System	
K/A Reference: <u>400000A4.01</u>	K/A Importance: <u>3.1/3.0</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion: <u>15 min</u>	Time Critica Task: NO
<u>Method of testing:</u> Performance: Simulated X_ Actual	Location: <u>X</u> Simulator Plant
Task Standard:	
Dequired Meteriolo:	

Required Materials: LOP-RH-05, Operation of the RHR Service Water System

General References:

LOP-RH-05, Operation of the RHR Service Water System, Rev 22, 1/17/2001

#### READ TO THE EXAMINEE:

I will explain the initial conditions, which steps to simulate or discuss. and provide initiating cues. Tell me when you have successfully completed the task.

Initial Conditions:

Plant Conditions are as shown. 'B' RHRSW has been running for a pump vibration and flow data test. B RHR SW Process Radiation Monitor is INOP. Chemistry has completed the required sample within the last hour. 1E12-F448 and 1E12-F4541 have been cycled as part of the test. The test is complete. There are no Tech Spec time clocks in effect. An operator is standing by to assist you.

Initiating Cue:

The Unit Supervisor has directed you to secure 'B' RHRSW IAW LCP-RH-05. Inform the Unit Supervisor when the task is complete.

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Simulator Setup Instructions:

1. This JPM can be run from any IC.

It is OK to use a similar IC to the IC listed above, provided the IC actually used is verified to be compatible with this and other JPMs that are scheduled to be run concurrently.

- 2. Verify Div 1 and Div 2 Thermal O/L Bypass switches are in NORMAL.
- 3. Start 'B' RHR Service Water system
  - Start 'C RHR Service Water pump (1E12-C300C)
  - Open 'B' RHR Service Water Heat Exchanger Outlet Valve, 1E12-F068B
  - Start 'D' RHR service Water pump (1E12-C300D)
- 4. This completes the setup for this JPM.

Initial Conditions:

Plant Conditions are as shown.

- 'B' RHRSW has been running for a pump vibration and flow data test.
- B RHR SW Process Radiation Monitor is INOP.
- Chemistry has completed the required sample within the last hour.
- 1E12-F448 and 1E12-F4541 have been cycled as part of the test.
- The test is complete.

There are no Tech Spec time clocks in effect. An operator is standing by to assist you.

Initiating Cue:

The Unit Supervisor has directed you to secure 'B' RHRSW IAW LOP-RH-05. Inform the Unit Supervisor when the task is complete.

	JOB PERI	FORMANCE MEASURE			
(Deno	ote critical steps with an asterisk)	,			
NOTI	E: All steps of this JPM are to be o	completed at CR Panel 1H13-P601		UNSAT	Comment #
	Element	Expected Response	SAI		
1.	If RHR Service Water Process radiation Monitor is inoperable, VERIFY that Chemistry has collected the proper samples.	Candidate determines conditions met from 'Initial Conditions.'			
2.	Locally Backwash the operating strainer per LOP-RH-14.	Local operator directed to backwash B RHRSW strainer.			<u> </u>
CUE	As local operator, ack the B RHRSW strainer	nowledge direction. Wait a minute, th has been backflushed IAW LOP-RH-	nen <b>re</b> 14.	port	that
3.	PLACE the O/L Bypass for 1E12-F003B, 4B, 4C, 6B, 47B, 68B, 73B, 74B, 93, 94 Thermal Overload bypass switch to test.	O/L Bypass for 1E12-F003B, 4B, 4C, 6B, 47B, 68B, 73B, 74B, 93, 94 Thermal Overload bypass switch placed in TEST.			
4.	Log positioning of O/L Bypass switch in Unit Log.	Position of O/L Bypass switch o Logged in unit log o Requested to be logged by Unit NSO.			
CUE	: If requested, as Unit NSO for 1E12-F003B, 4B, 4C, 6 bypass switch to TEST ha	9, acknowledge request, report that th 6B, 47B, 68B, 73B, 74B, 93, 94 Therma as been logged.	e O/L I Ovei	Bypa rload	155
<b>5</b> .	If required, START 8 hour clock	Candidate determines that an 8 hour timeclock must be started.			
CUE	: If requested, as Unit Sup information, state that th Tech Spec Time clock.	ervisor, acknowledge Tech Spec Time e Unit NSO will perform the necessar	e clo <b>c</b> y actio	k ons f	or the

		RMANCE MEASURE		<u> </u>	
(Den	ote critical steps with an asterisk) Element	Expected Response	SAT	UNSAT	Comment #
	Liement				
6.	If only on RHRSW pump is in operation (Step E.5.4).	Candidate determines step is not applicable and continues with step E.5.5.			
*7.	Place control switch for 1E12- F068B, 'B' RHR Hx water outlet vlv to CLOSE.	Control switch for 1E12-F068B, 'B' RHR Hx water outlet vlv placed in close position.			
*8.	When flow indicated on 1E12- R602B decreased below 4000 gpm, STOP one RHRSW pump 1E12- 300C/D.	C or D RHRSW pump control switch taken to STOP when flow decreases to <4000 gpm.		<u> </u>	_
*9.	When 1E-F068B, 'B' RHR water outlet valve is closed, STOP the running RHRSW pump 1E12- 300D/C.	D or C RHRSW pump control switch taken to STOP when 1E- F068B, 'B' RHR water outlet valve is closed.			
10.	VERIFY flow decreases to 0 gpm on flow indicator 1E12-R602B	Flow decrease to 0 gpm on flow indicator 1E12-R602B.		<u></u>	
11.	Place the O/L Bypass for 1E12- F003B, 4B, 4C, 6B, 47B, 68B, 73B, 74B, 93, 94 Thermal Overload bypass switch to NORMAL.	O/L Bypass for 1E12-F003B, 4B, 4C, 6B, 47B, 68B, 73B, 74B, 93, 94 Thermal Overload bypass switch to NORMAL.			
12.	Log position O/L Bypass switch in unit log.	Position of O/L Bypass switch o Logged in unit log Requested to be logg Unit NSO.			 :

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(Denote critical steps with an asterisk)

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	Element	Expected Response	SAT	UNSAT	Comment #
CUE	: If requested, as Unit N that the O/L Bypass fo	SO, acknowledge request, report r 1E12-F003B, 4B, 4C, 6B, 47B, 68B, 7 ass switch to NORMAL has been logg	3B, 7 ed.	4B, 9	3, 94
13.	If required, STOP 8 hour clock	Candidate determines that an 8 hour timeclock must be stopped.			
CUE	: If requested, as Unit Supe information, state that the Tech Spec Time clock.	ervisor, acknowledge Tech Spec Time Init NSO will perform the necessary	clock actic	c ons fc	r the
14.	Shutdown the RHR SW process radiation monitor per LOP-PR-06.	Candidate determines that step to by not applicable.			
15.	Locally in the DG bulding penthouse, VERIFY the Div 2 Unit 1 1VY06C RHRSW Ventilation fan stops.	Step determined to be Not applicable.			
CUE	: As local operator, acknowle RHRSW Ventilation fan has	edge direction and then report that Div 2	Unit 1	1VY	06C
16.	Verify the appropriate ESF Status Panel Annunciator RHR B Cont Heat Rem Inop (#36) is CLEARED.	ESF Status Panel Annunciator window #36 is cleared.			
17.	Inform US that LOP-RH-05 is complete.	Unit Supervisor informed that LOP- RH-05 is complete.			
	: THIS JPM IS COMPLETE. er JPM Stop Time:				

## VERIFICATION OF COMPLETION

Job Performance Measure No	 -		
Examinee's Name:			
Examiner's Name:			
Date performed:			
Number of attempts:			
Time to complete:			
Question Documentation:			
Question:			
Response:	 · · · · · · · · · · · · · · · · · · ·		
Result: SAT or UNSAT			
Examiner's signature and date:		·.	

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# LASALLE COUNTY NUCLEAR STATION JOB PERFORMANCE MEASURE Transfer a Turbine Driven Reactor Feed Pump from Manual Backup Station to 3-Element Control Examination Level: RO /SRO /SRO(I) Date: March 5, 2002 Developed by: Raymond Keith Walton Date: \_\_\_\_\_ Reviewed by: \_\_\_\_\_ Date: \_\_\_\_\_ Approved by: \_\_\_\_\_ Date:\_\_\_\_

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Facility: LaSalle Nuclear Station	Task No: <u>B.1.b</u>
Task Title: Transfer a TDRFP from Manual Backup St	ation to 3-Element Control
Joh Petformance Measure No:	
K/A Reference:259002A4.02	K/A Importance: <u>3.9/3.7</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Time Critical Task: NO	
Method of testing: Performance: Simulated X_ Actual	Location: X Simulator Plant
Task Standard:	
Required Materials: LOP-FW-04, Startup cf a Turbine Driven Reactor Feed LOP-RL-01, Operation of the Reactor Level Control Sys	Pump, Rev 33, May 15, 2001 stem, Reve 14, 8/24/1999

#### General References:

LOP-FW-17, TDRFP Manual Backup Station Operation LOP-RL-01, Operation of the Reactor Level Control System

#### READ TO THE EXAMINEE:

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. Tell me when you have successfully complete the task.

#### Initial Conditions:

- Unit 1 experienced slight oscillations in the AUTO control signal for the 'B' TDRFP.
- The pump was transferred to the demand substitution mode on the manual backup station IAW LOP-FW-17, Step E.1.

#### Initiating Cue:

The Unit Supervisor has directed you to restore control of the B TDRFP to the M/A station IAW LOP-FW-17, Step E.2 and then place the difference of the automatic (three element control) on its M/A states HAW LOP-RL-01.

#### Simulator Setup Instructions:

1. Reset the simulator to IC 22 (rst22).

It is OK to use a similar IC to the IC listed above, provided the IC actually used is verified to be compatible with this and other JPMs that are scheduled to be run concurrently.

- 2. Conditions as follows:
  - Place the simulator in RUN
  - Place the B TDRFP in Manual on its M/A station
  - Shift it to the manual backup station in demand substitution mode
  - Lower the M/A station signal to minimum
- 3. Freeze the simulator until cued by examiner.
- 4. When the above steps are completed for this and other JPMs to be run concurrently, then validate the concurrently run JPMs using the noted steps on the Job Performance Measure Validation Checklist located on page 2.
- 5. This completes the setup for this JPM.

#### INITIAL CONDITIONS

(To be read by examinee)

Initial Conditions:

- Unit 1 experienced slight oscillations in the AUTO control signal for the 'B' TDRFP.
- The pump was transferred to the demand substitution mode on the manual backup station IAW LOP-FW-17, Step E.1.

Initiating Cue:

The Unit Supervisor has directed you to restore control of the B TDRFP to the M A station IAW LOP-FW-17, Step E.2 and then place the 'B' TDRFP into automatic (three element control) on its M/A station IAW LOP-RL-01.

	JOB PERFORMANCE MEASURE					
(Den	ote critical steps with an asterisk)					
	ss otherwise noted.	ed at control room panel H13-P603	SAT	UNSAT	Comment #	
	Element	Expected Response		<u>ر</u>		
1.	VERIFY Manual Backup Station MODE SELECTOR switch in AUTO.	Manual Backup Station MODE SELECTOR switch in AUTO.	<u> </u>		. <u> </u>	
*2.	RAISE/LOWER demand at the M/A Xfer Station to extinguish AUTO HI or AUTO LO LEDs and illuminate RESET Permissive LED.	Candidate RAISES/LOWERS demand at the M/A Xfer Station to extinguish AUTO HI or AUTO LO LEDs and illuminate RESET Permissive LED.				
*3.	RESTORE control of the 'B' TDRFP to the M/A station by Depressing RESET TO AUTO pushbutton.	Candidate depresses RESET TO AUTO pushbutton.				
4.	Observing the CHECK DEMAND SUBSTITUTION and DIRECT VALVE POSITIONING LEDs extinguish.	The candidate verifies the CHECK DEMAND SUBSTITUTION and DIRECT VALVE POSITIONING LEDs are extinguished.				
*5.	Match the output of the 'B' TDRFP to the 'A' TDRFP	Candidate RAISES/LOWERS demand at the M/A Xfer Station for the 'B' TDRFP as needed until it matches the output of the 'A' TDRFP M/A station.				

CUE: If feed pump flows are not balanced when M/A station outputs are equal, the candidate may request guidance from the Unit Supervisor. If so, inform the candidate to get M/A station outputs approximately equal and balance feed pump flows.

#### JOB PERFORMANCE MEASURE (Denote critical steps with an asterisk) Comment UNSAT SAT **Expected Response** Element Candidate verifies Reactor Water **VERIEY Reactor Water Level is** 6. Level is stable and deviation meter on stable and deviation meter on Single Element Controller is within Single Element Controller is Green Band. within Green Band. Candidate request IMD to check Check M/A station manual and 7. signal output voltages are automatic signals equal. approximately equal. CUE: as IMD, report that signal output voltages have been checked for TDRFP 1B and the signals are approximately equal. Place the 'B' TDRFP in AUTO Candidate depresses the AUTO \*8. pushbutton on the 'B' TDRFP M/A and VERIFY level control stable. station and observes RWL stable. Inform Unit Supervisor that B 9. TDRFP is now in 3-element Candidate states that TDRFP is now control. in 3-element control.

(CUE) THIS COMPLETES THIS JPM Record Stop Time: \_\_\_\_\_

## VERIFICATION OF COMPLETION

Job Performance Measure No.
Examinee's Name:
Examiner's Name:
Date performed:
Number of attempts:
Time to complete:
Question Documentation:
Question:
Response:
Result: SAT or UNSAT

Examiner's signature and date:

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## LASALLE COUNTY NUCLEAR STATION

## JOB PERFORMANCE MEASURE

Main Turbine Weekly Surveillance Test

Examination Level: RO /SRO /SRO(I)

Date: March 5, 2002

Developed by: Raymond Keith Walton Date: <u>3/14/2002</u>

Date: \_\_\_\_\_ Reviewed by:

Approved by: \_\_\_\_\_ Date:\_\_\_\_\_

Facility: LaSalle Nuclear Station	Task No: <u>B.1.c</u>
Task Title: <u>Main Turbine Weekly Surveillance Test</u>	
K/A Reference: 241000A4.06	K/A Importance: <u>3.9/3.9</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion: 20 min.	Time Critical Task: NO
Method of testing: Peformance: Simulated Actual	Location: <u>X</u> Simulator Plant
Task Standard:	
Required Materials: LOS-TG-W1, Turb ne Weekly Surveillances, Rev 34, 1	1/2/2001

General References: LOS-TG-W1, Turbine Weekly Surveillances

#### READ TO THE EXAMINEE:

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. Tell me when you have successfully complete the task.

Initial Conditions: Unit 1 is at <50% of full power.

Initiating Cue:

Unit supervisor has directed you to perform Turbine Weekly Surveillance testing per LOS-TG-W1, Attachment 1A, Section A.2 (BPV). All Prerequisites and Precautions have been met. The Load Dispatcher has given permission to perform this surveillance.

Simulator Setup Instructions:

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1. Setup for <50% power operations and main turbine generator on line.

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Initial Conditions: Unit 1 is at <50% of full power.

Initiating Cue:

Unit supervisor has directed you to perform Turbine Weekly Surveillance testing cer LOS-TG-W1, Attachment 1A, Section A.2 (BPV). All Prerequisites and Precautions have been met. The Load Dispatcher has given permission to perform this surveillance.

Inform your supervisor when this task has been completed.

(Denote critical steps with an asterisk)

		SAT	UNSAT	Comment #
<u>Element</u>	Expected Response			
<ul> <li>*1. TEST OPEN Turbine Bypass Valve 1B21-MSBPV-1:</li> <li>a. CHECK BPV Ready to Select light is energized.</li> <li>b. Place the BYPASS VALVE TEST selector SW in Postion 1</li> <li>c. DEPRESS the TEST BYPASS VALVE button.</li> </ul>	BPV Ready to Select light is energized Bypass Valve Test selector switch in Postion 1. Depresses the TEST BYPASS VALVE button 1B21-MS-BPV-1 goes to Full open exhibiting fast open characteristics the final 10% of travel.			
d. Release the TEST BYPASS VALVE button.	Test Bypass Valve button released 1B21MS-BPV-1 goes to Full closed.			
<ul> <li>*2. TEST OPEN Turbine Bypass Valve 1B21-MSBPV-2:</li> <li>a. CHECK BPV Ready to Select light is energized.</li> <li>b. Place the BYPASS VALVE TEST selector SW in Position 2.</li> <li>c. DEPRESS the TEST BYPASS VALVE button.</li> <li>d. Release the TEST BYPASS VALVE button.</li> </ul>	BPV Ready to Select light is energized Bypass Valve Test selector switch in Position 2. Depresses the TEST BYPASS VALVE button. 1B21-MSBPV-2 goes to Full open exhibiting fast open characteristics final 10% of travel. Test Bypass Valve button released 1B21MSBPV-2 goes to Full closed.		 	·
*3. Candidate REPORTS failure of MSBPV-2 to the period final 10% of travel.	CUE: Unit Supervisor acknowle gractailure. US will activate TADS.			

(Denote critical steps with an asterisk)

	<u>Element</u>	Expected Response	SAT	UNSAT	Comment #
b. c.	TEST OPEN Turbine Bypass Valve 1B21-MSBPV-3: CHECK BPV Ready to Select light is energized. Place the BYPASS VALVE TEST selector SW in Position 3. DEPRESS the TEST BYPASS VALVE button.	BPV Ready to Select light is energized Bypass Valve Test selector switch in Position 3. Depresses the TEST BYPASS VALVE button 1B21-MSBPV-3 goes to Full open exhibiting fast open the final 10% of travel. Test Bypass Valve button released 1B21MSBPV-3 goes to Full closed.			
b. c.	TEST OPEN Turbine Bypass Valve 1B21-MSBPV-4: CHECK BPV Ready to Select light is energized. Place the BYPASS VALVE TEST selector SW in Position 4 DEPRESS the TEST BYPASS VALVE button. Release the TEST BYPASS VALVE button.	BPV Ready to Select light is energized Bypass Valve Test selector switch in Position 4. Depresses the TEST BYPASS VALVE button. 1B21-MSBPV-4 goes to Full open exhibiting fast open the final 10% of travel. Test Bypass Valve button released 1B21MSBPV-4 goes to Full closed.			

(Denote critical steps with an asterisk)

	<u>Element</u>	Expected Response	SAT	UNSAT	Comment #
b. c.	TEST OPEN Turbine Bypass Valve 1B21-MSBPV-5: CHECK BPV Ready to Select light is energized. Place the BYPASS VALVE TEST selector SW in Position 5. DEPRESS the TEST BYPASS VALVE button.	BPV Ready to Select light is energized Bypass Valve Test selector switch in Position 5. Depresses the TEST BYPASS VALVE button 1B21-MSBPV-5 goes to Full open exhibiting fast open the final 10% of travel. Test Bypass Valve button released 1B21MSBPV-5 goes to Full closed.			
R	HECK that the Bypass Valve EADY TO SELECT Light is nergized. PLACE the BYPASS VALVE	Ready To Select Light is energized Bypass Valve Test Selector Switch to			
	TEST selector Sw to OFF.	OFF			

(CUE) THIS COMPLETES THIS JPM Record Stop Time: \_\_\_\_\_

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## VERIFICATION OF COMPLETION

Job Performance Measure No		
Examinee's Name:		
Examiner's Name:		
Date performed:		
Number of attempts:		
Time to complete:		
Question Documentation:		
Question:		
Response:		
Result: SAT or UNSAT		
Examiner's signature and date:		
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## LASALLE COUNTY NUCLEAR STATION

## JOB PERFORMANCE MEASURE

Stuck Control Rod Drive

Examination Level: RO /SRO /SRO(I)

Date: April 8, 2002

Developed by: Raymond Keith Walton Date: \_\_\_\_\_

Reviewed by: \_\_\_\_\_ Date: \_\_\_\_\_

18. K.L

approved by:

Facility: LaSalle Nuclear Station	Task No: <u>B.1.d</u>
Task Title: <u>Stuck Control Rod Drive</u>	
K/A Reference:201001A4.04	K/A Importance: <u>3.1/3.0</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion:	Time Critical Task: NO
Method of testing: Performance: Simulated Actual	Location: <u>X</u> Simulator Plant
Task Standard:	

Required Materials: Copy of LAP 100-13, Attachment C, for rod moves.

General References:

LOA-RD-101, Control Rod Drive Abnormal, Rev 4, 3/27/2001 LAP 100-13, Control Rod Sequence Package Preparation, Review and Implementation.

READ TO THE EXAMINEE:

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. Tell me when you have successfully complete the task.

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Initial Conditions: Unit 1 at full power operation.

Initiating Cue:

Unit Supervisor has instructed you to adjust control rods IAW LAP 100-13, Attachment C and qualified nuclear engineer instructions.

Simulator Setup Instructions:

- 1. Setup for full power operations
- 2. Have "in-step" CRD stick at present position. CRD will not respond to manual rod motion controls.

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#### INITIAL CONDITIONS

Initial Conditions: Unit 1 at full power operation.

Initiating Cue:

Unit Supervisor has instructed you to adjust control rods IAW LAP 100-13, Attachment C and qualified nuclear engineer instructions.

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#### JOB PERFORMANCE MEASURE (Denote critical steps with an asterisk) Comment UNSAT SAT **Expected Response** Element Candidate reviews LAP 100-13, Operator reviews LAP 100-13, 1. Attachment C. Notes that rods are to Attachment C for CRD moves. be moved in 4 steps. ATTEMPTS to INSERT CRD CRD (?) selected. Candidate 2. depresses CRD INSERT pushbutton. (?) by DEPRESSING notch in Pushbutton. NOTE: Candidate should refer to LOA-NOTIFIES Unit Supervisor that \*3. RD-101, CRD Abnormal Procedure, CRD will not move. Section B.5. CUE: Unit Supervisor acknowledges problems with CRD moving. Need to move rods! CRD Hydraulic System is Normal. VERIFIES CRD Hydraulic 4. System NORMAL. **Control Rod Position indication CHECK** Control Rod Position 5. Indication is NORMAL. system normal. Attempts to insert control rod but Attempts to INSERT CRD (?). 6. control rod does not move. **OBSERVE** Directional Control Valve Sequencing. Directional Control Valve Sequencing \_\_\_\_\_ 7. CHECK Directional Control normal. Valve Sequencing NORMAL. CHECK CRD (?) Failed to move CRD (?) Failed to move with drive 8. water product < 450 psid. with drive water produce <450 psid.

(Denote critical steps with an asterisk)

NOTE TO EXAMINER: INITIATE ALTERNATE PATH PORTION OF JPM. Goal is to insert CRD by increasing CRD pressure by throttling on CRD PCV.			F	UNSAT	Comment #
	Element	Expected Response	SAT	N	ů C
*9.	RAISE Drive Water pressure to 500 psid.	Candidate adjusts drive water pressure to 500 psid by throttling PCV 1C11-F003.			
10.	Attempt to INSERT CRD (?). Depresses CRM INSERT pushbutton	Depresses CRM INSERT pushbutton. Control Rod has not moved.			
*11.	RAISE Drive Water Pressure to 550 psid.	Candidate adjusts drive water pressure to 550 psid by throttling PCV 1C11-F003.			
*12.	Depresses CRM INSERT pushbutton	Control Rod moves. Candidate moves CRD (?) In 1 notch			
*13	RETURN Drive Water pressure to NORMAL	Candidate adjusts drive water pressure to <450 psid by throttling PCV 1C11-F003			
*14.	Attempts to more CRD (?) In 1 notch.	Candidate depresses CRM pushbutton. CRD (?) Moves in one notch.			
(CUE) THIS COMPLETES THIS JPM Record Stop Time:					

## VERIFICATION OF COMPLETION

Job Performance Measure No.		
Examinee's Name:		
Examiner's Name:		
Date performed:		
Number of attempts:		
Time to complete:		
Question Documentation:		
Question:		
Response:	 	
Result: SAT or UNSAT		
Examiner's signature and date:		··· · · · ·

## LaSalle Station

UNIT 1, 2 AND COMMON

OPERATING DEPARTMENT SURVEILLANCE

## TECHNICAL SPECIFICATIONS WEEKLY SURVEILLANCES

LOS-AA-W1 Revision 47 March 13, 2002



Procedure Responsibility/Revie	ew/Approval Requirements	
Responsible Department Head:	SOS	
Minimum Review Type:	TR	
Required Cross-Discipline Review(s):	QNE & ISTC	
Approval Position Required:	SOS	

1. Review/Approval requirements apply to non-editorial procedure revisions.

#### TECHNICAL SPECIFICATIONS WEEKLY SURVEILLANCES

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Level of Use Reference : -

#### TECHNICAL SPECIFICATION WEEKLY SURVEILLANCES

#### A. <u>PURPOSE</u>

#### A.1 <u>Objective</u>

To satisfy the following Technical Specification weekly surveillance requirements:

Control Rods:

- Tech Spec SR 3.1.3.2, SR 3.1.3.3, SR 3.1.3.5 and SR 3.9.5.1, Demonstrate withdrawn control rods are operable.
- Tech Spec SR 3.1.5.1 and SR 3.9.5.2, Determine each control rod scram accumulator operable.

Automatic Depressurization System

- Tech Spec SR 3.5.1.3 Verify ADS accumulator supply header pressure is ≥ 150 psig. (Tech Spec requirement is every 31 days)
- Tech Spec SR 3.5.1.4 Verify ADS accumulator backup compressed gas system bottle pressure is ≥ 5(9) psig. (Tech Spec requirement is every 31 days)

#### Containment:

- Tech Spec SR 3.6.1.6.1, Verify suppression chamber-dryvell vacuum breakers operable.
- TRM 3.6.b, Verify suppression chamber-drywell vacuum breaker closed, <u>if</u> vacuum breaker indication is inoperable.
- Tech Spec SR 3.6.3.2.1, Determine drywell and suppression chamber oxygen concentration within specification.

#### CO<sub>2</sub> System:

Technical Requirements Manual TSR 3.7.1.1 and TSR 3.7.1.2.
 Worify low pressure CO<sub>2</sub> system operable.

CR / AEER Air Conditioning (AC) Breaker Alignment

• Tech Spec SR 3.7.5.2, Determine correct breaker alignment and indicated power available to control room area / AEER ventilation AC subsystems.

Electrical Distribution:

- Tech Spec SR 3.8.1.1, Determine required A.C. Sources-Operating between the offsite transmission network and onsite Class 1E distribution system operable.
- Tech Spec SR 3.8.2.1, Determine required A.C. Sources-Shutdown between the offsite transmission network and onsite Class IE distribution system operable.
- Tech Spec SR 3.8.7.1. Determine required A.C. Distribution-Operating system electrical divisions operable.
- Tech Spec SR 3.8.8.1, Determine required A.C. Distribution-Shutdown system electrical divisions operable.

Spent Fuel Storage Pool

• Tech Spec SR 3.7.8.1, Determine spent Fuel Storage Pool water level above low level limit.

#### A.2 <u>Discussion</u>

Control Rods:

Operability of the control rods is verified by actual insertion of the control rods. On a weekly frequency, all control rods that are fully withdrawn when the surveillance is performed, are exercised. On a monthly frequency, all control rods that are at an intermediate position when the surveillance is performed, are exercised. An additional subsection provides methods for clearing high temperature conditions for any control rod that has a loss of cooling due to the operability check.

Operability of control rod accumulators is verified by checking the HCU accumulator pressures. If the HCU accumulator pressure is out of the specified limits, corrective actions are provided.

Verification of no overtravel is accomplished during the weekly control rod exercising.

- E.3.2 <u>If reactor power is NOT</u> above the Low Power Setpoint of Tech Spec 3.3.2.1, ENTER a note on Attachment 1A(2A) sign off sheet under "Comments" that Attachment 1B(2B)/1B-Int(2B-Int) is <u>NOT</u> applicable because power is at or below the Low Power Setpoint. Also, Tech Spec 3.9.5 is applicable for each CRD that is continuously withdrawn for 7 days in Mode 5.
- E.3.3 If reactor power is above the Low Power Setpoint:
- E.3.3.1 If core thermal power is at or above 25%, DETERMINE current MFLPD and FDLRX (if applicable) value as follows:
  - 0 DEMAND OD-20 Option 1.
  - <u>If</u> reactor power level has <u>NOT</u> changed since last CMSS Core Performance Log, CHECK that print out.
- E.3.3.2 If core thermal power is at or above 25%, CHECK MFLPD and FDLRX (if applicable) are less than or equal to 0.950.
  - 0 <u>If</u> either is greater than 0.950, a Control Rod Maneuver Request (CRMR) is required and must be obtained from a QNE.
  - o <u>If</u> both are less than or equal to 0.950, no power reduction is necessary and a CRMR <u>is</u> not required.
- E.3.3.3 VERIFY the APRM Gains are within 2% of RTP. If any of the Gains are out, NOTIFY IMD to perform LIS-NR-109 (209).
- E.3.3.4 OBTAIN and ATTACH an OD-7 Option 2 computer edit of the Control Rod Positions. <u>If</u> the Process Computer is unavailable, use of the Four-Rod Display is allowed.

Level of Use **Referenc**e

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- E.3.3.5 For each control rod <u>NOT</u> at position 00, on Attachment 1B(2B):
  - For Intermediate rods, RECORD the rod position in the column marked "Rod Position".
  - If Intermediate rods will NOT be exercised per Attachment 1B-Int(2B-Int), ENTER N/A for each Intermediate control rod in the column marked "Initials".
  - <u>If</u> Intermediate rods will be exercised per Attachment 1B-Int(2B-Int), ENTER each Intermediate control rod number and position on that Attachment and INITIAL the column marked "Initials" for each Intermediate control rod.
  - For rods full out (position 48), INDICATE rod position by leaving the "Rod Position" column blank.
- E.3.3.6 <u>Prior</u> to exercising Control Rods, SELECT the "Rod Exercise" mode via the RWM Touch Screen.

## NOTE

If required, Rod Drift Test should be performed concurrently with exercising one control rod.

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LOS-AA-W1 Revision 47 E.3.3.7 While exercising Control Rods, if rods are out in Mode 1 or 2, at 1(2)H13-P603, TEST Rod Drift Alarm as follows:

## NOTES

If any control rod multiple notches while exercising, the Shift Manager or his designee should be notified to determine if CRD Flushing should be performed. Control Rod Drive Problem Identification sheet of Attachment 1(2)B and a Condition Report shall be completed for any control rod that multiple notches.

CRD Flushing should <u>NOT</u> be performed on a drive if there is an associated work request written identifying the control rod as a multiple notching rod.

E.3.3.7.1	DEPRESS the Rod Drift Test push-button.	
-----------	---	--

- E.3.3.7.2 NOTCH a control rod as follows:
  - If above the LPSP, per rod exercising steps (an edge rod is preferred)
  - o If below the LPSP, per QNE direction.
- E.3.3.7.3 VERIFY 1(2)H13-P603 A504 CRD Drift alarm and red light on full core display is illuminated.
- E.3.3.7.4 DEPRESS Rod Drift Reset push-button and NOTE CRD Drift alarm and red light on full core display clears.
- E.3.3.7.5 RECORD testing of Rod Drift Alarm on Attachment 1B(2B).

## CAUTION

Elevated drive water pressures may result in multiple rod notch moves during rod exercising.

I INSERT each Future a Control Rod one (1) note: a additionally while monitoring the Rod position and OBSERVE that the Rod has moved.

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- E.3.3 RETURN the Rod to its original position and PERFORM a coupling check, verifying the rod does not go to the overtravel position. (Tech Spec SR 3.1.3.5)
- E.3.3:10 RECORD satisfactory Control Rod operation on Attachment 1B(2B) by initialing the "Initials" column for each rod.
- E.3.3 11 For those rods full-out (position 48) identified as multiple notching rods which do <u>NOT</u> have an associated work request, <u>if</u> conditions permit PERFORM the following to flush the Control Rod Drive.
  - Elevate drive pressure to between 350-400 psid.
  - Apply a continuous withdrawal signal and hold for 60-90 seconds.
  - Repeat application of continuous withdrawal signal at least 10 times.
- E.3.3 12 For those rods which were flushed, REPEAT Rod Exercising Steps.
  - <u>If</u> the control rod multiple notches a second time, NOTE all pertinent information on Control Rod Drive Problem Identification sheet of Attachment 1(2)B and ROUTE to System Engineer to evaluate for further corrective action.

## NOTE

For each control rod <u>NOT</u> at position 48 or 00, a second licensed operator or qualified member of the System Engineering Department shall verify the CRD movement and both the NSO performing the CRD movement and the Second Verifier shall initial on the appropriate check line.

- E.3.3.13 If Intermediate control rods are to be exercised, INSERT each Control Rod <u>NOT</u> at position 48 or 00, one (1) notch individually while monitoring the Rod position.
  - OBSERVE that the Rod has moved.
- E.3.3.13.1 RETURN the Rod to its original position.
- E.3.3.13.2 RECORD satisfactory Control Rod operation on Attachment 1B-Int(2B-Int) by initialing the "Performed" column for each rod.
- E.3.3.13.3 Independently VERIFY satisfactory Control Rod operation on Attachment 1B-Int(2B-Int) by initialing the "Verified" column for each rod.
- E.3.3.14 VERIFY all control rods required to be exercised have been checked. INDICATE any Control Rods that were <u>NOT</u> tested using the following notations on Attachment 1B(2B):
  - o Full In 00
  - o Isolated Isol
  - o Electrically Disarmed ED
- E.3.3.15 OBTAIN and ATTACH an OD-7 Option 2 computer edit of the Control Rod Positions.
- E.3.3.16 COMPARE the initial OD-7 obtained with the final OD-7 obtained to VERIFY there are no mispositioned rods.
- E.3.3.17 RETAIN Attachment 1B(2B)/1B-Int(2B-Int) for submittal with Attachment (1996).

Level of Use -Reference E.3.3.18 When control rod exercising is complete:

- EXIT the Rod Exercise mode.
- o RETURN to Power Reduction mode.
- o INCREASE core thermal power.
- o RESTORE reactor power per QNE.
- INITIATE a Condition Report regarding any control rod(s) that double notched.
- E.4 If CRDs become hot after exercising, ESTABLISH cooling water as follows:
- E.4.1 OPEN CRD FCV to increase CRD cooling water flow to approximately 70 gpm for intermediate position hot CRDs.
- E.4.1.1 If 70 gpm cannot be obtained with the CRD FCV 100% open, CRD drive water differential pressure should be temporarily lowered.

## NOTE

The increased flow should be maintained for up to fifteen minutes, in an attempt to clear as many of the hot CRDs as possible.

- E.4.1.2 <u>After</u> flushing, CHECK that:
  - CRD system parameters return to the normal ranges.
  - Any CRD filter dP alarms caused by this evolution are clear.

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#### ATTACHMENT IB

#### UNIT ONE

# ROD OPERABILITY CHECK SIGN-OFF SHEET FOR FULLY WITHDRAWN RODS

E.3.3.3 OBTAIN and ATTACH an OD-7 Option 2.

E.3.3.6.5 Rod Drift Alarm Tested Sat (Mode 1 or 2) /

E.3.3.9 RECORD satisfactory Control Rod operation.

ROD NO.		INITIALS			INITIALS		ROD* POSITION	INITIALS
02-19			06-43			14-07		
02-23			. 06-47			14-11		
02-27			[0-1]	<u>,,</u>		14-15		<u> </u>
02-31			10-15			14-19		
02-35			10-19			14-23		
02-39			10-23		<u></u>	14-27		
02-43	. <u></u>		10-27	<u></u>		14-31		
06-15			10-31			. 14-35		<u> </u>
06-19			10-35	<u>,,</u> ,	<del></del>	14-39		
06-23			- 10-39	<u> </u>		- 14-43		
06-27			10-43			14-47	<u></u>	
06-31			10-47		<u> </u>	- 14-51	<u> </u>	
06-35		······	10-51			- 14-55	· · · · · · · · · · · · · · · · ·	· -

\* <u>NOTATIONS for Rod Position Column</u> Intermediate Position -Rod Position Full In - 00 Isolanda Isol Electrically Disarmed - ED Full Out - Leave Blank

06-39

Level of L	lse
Reference	ce

# LASALLE COUNTY NUCLEAR STATION

## JOB PERFORMANCE MEASURE

Failure of SBGT System to Automatically Initiate

Examination Level: RO /SRO /SRO(I)

Date: April 8, 2002

Developed by: <u>Raymond Ke</u>	Date: <u>3/14/2002</u>		
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Reviewed by:	 	Date:	
	n an		
Approved,		Date:	

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Facility: LaSalle Nuclear Station	Task No: <u>B.1.e</u>
Task Title: Fuel Assembly Damage Results in Manu	al Actuation of ESF System
K/A Reference:	K/A Importance: <u>3.2/3.4</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion:	Time Critical Task: NO
<u>Method of testing:</u> Performance: Simulated X Actual	Location: <u>X</u> Simulator Plant
Task Standard:	
Required Materials: LOA-FH-001, Irradiated Fuel Assembly Damage, Rev ( LOP-VG-01, Preparation for Standby Operation of the	0, 7/31/1996 SBGT System, Rev. 9, 11/6/2000
General References: LOA-FH-001, Irradiated Fuel Assembly Damage, Rev LOP-VG-01, Preparation for Standby Operation of the	0, 7/31/1996 SBGT System, Rev. 9, 11/6/2000
READ TO THE EXAMINEE: I will explain the initial conditions, which steps to simul me when you have successfully complete the task.	ate or discuss, and provide initiating cues. Tell
Initial Conditions: Unit 1 is in Mode 5 with fuel shuffles in progress. Unit supervisor calls you in control room to inform you that fuel pool. The dropped fuel is emitting gas bubbles. T alarming. RP personnel have ordered an evacuation of	they have just dropped a fuel bundle in the spent The radiation monitor on the fuel handling bridge is
Initiating Cue: The Unit Supervisor has directed you to implement LC has been completed.	Addition of a inform your supervisor when this task.

Simulator Setup Instructions:

1. Reset Simulator to IC 26 (rst 26).

NOTE: It is OK to use a similar IC to the IC listed above, provided the IC actually used to verified to be compatible with this and other JPMs that are scheduled to be run concurrently.

- 2. Set Simulator up for Mode 5 operations.
- 3. Enter fault: SBGT system does not start with the control switch.
- 4. About 10 seconds after starting this JPM, insert annunciator H13-P601-E306, Fuel Pool Vent Rad Hi, in alarm condition.

#### INITIAL CONDITIONS

- Unit 1 is in Mode 5 with fuel shuffles in progress.
- Unit 2 is operating in Mode 1.
- Fuel handling supervisor calls you in the control room to inform you that they have just dropped a fuel bundle in the spent fuel pool.
- The dropped fuel is emitting gas bubbles. The radiation monitor on the fuel handling bridge is alarming.
- RP personnel have ordered an evacuation of the refuel floor.

Initiating Cue:

The Unit Supervisor has directed you to implement LOA-FH-001. Inform your supervisor when this task has been completed.

	JOB PEF	RFORMANCE MEASURE			
(Der	ote critical steps with an asterisk)	)			
NOT	E: All steps of the JPM are to be <u>Element</u>	completed at Control Room <u>Expected Response</u>	SAT	UNSAT	Comment #
1.	REFERS to LOA-FH-101, Irradiated Fuel Assembly Damage.	Locates and opens LOA-FH-101.			<u></u>
2.	CHECK Reactor Bldg Exhaust Duct Radiation <10 mr/hr.	RB exhaust radiation >10 mr/hr Annunciators H13-P601-E204, E205 & Annunciators H13-P601-F204, F205 illuminated.		<u></u>	
3.	VERIFY that Reactor Bldg Ventilation System isolates	Verifies RB Ventilation fans off and RB Ventilation inlet/outlet valves closed.			
*4.	IDENTIFY SBGT system failed to start.	SBGT fans <u>NOT</u> operating, SBGT valves did <u>NOT</u> open.			
5.	Informs Unit Supervisor that SBGT failed to start.	If the candidate asks CUE: Start SBGT using the Manual System Test Pushbutton on Panel PM07J.			
6.	START SBGT by DEPRESSING SBGT Test Push button on Panel PM07J.	Depresses SBGT PB on PM07J. SBGT fans <u>NOT</u> operating, SBGT valves did <u>NOT</u> open.	 *		
7.	Informs Unit Supervisor that SBGT failed to start using Manual System Test Pushbutton.	CUE: Need to have SBGT operating.	<u> </u>		

(Den	ote critical steps with an asterisk)				
NOTE TO EXAMINER: INITIATE ALTERNATE PATH PORTION OF JPM. Goal is to manually initiate SBGT system.				JNSAT	Comment #
	Element	Expected Response	SA	N 	Ö
8.	REFERs to LGA-VG-101, Secondary Containment Pressure Control.	Locates and Opens LGA-VG-101			
*9.	Manually lines up and initiates at least one SBGT system IAW LGA-VG-101, Secondary Containment Pressure Control.	OPEN: 1/2VG002, SBGT Inlet Isol Valve. 1/2VG003, SBGT Disch Isol Valve. ON: 1/2VG01C, SBGT Primary fan 1/2VG01A, SBGT Electric Htr Coil OFF: 1/2VG02C, SBGT Cooling Fan Verify flow 3600 - 4400 CFM on 1/2FR-VG009 & 1/2FI-VG003.			
10 11. 1(12.	SUSPEND all refueling operations. EVACUATE unnecessary personnel from Refuel Floor.	Picks up CR phone, calls refuel floor RP desk and informs supervisor to suspend all refueling operations and evacuate personnel from refuel floor.			

CUE: THIS COMPLETES THIS JPM Record Stop Time: \_\_\_\_\_

# **VERIFICATION OF COMPLETION**

Job Performance Measure No		
Examinee's Name:		
Examiner's Name:		
Date performed:		
Number of attempts:		
Time to complete:		
Question Documentation:		
Question:		
Response:		 
Result: SAT or UNSAT		
	:	
Examiner's signature and date:	<u>1</u> 1	 
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# LASALLE COUNTY NUCLEAR STATION JOB PERFORMANCE MEASURE Transfer Electrical Loads from SAT 142 to UAT 141 Examination Level: RO /SRO /SRO(I) Date: March 5, 2002

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Facility: LaSalle Nuclear Station	Task No: <u>B.1.f</u>
Task Title: <u>Transfer Loads from SAT 142 to UAT 141</u>	
K/A Reference: <u>262001K5.01</u>	K/A Importance: <u>3.1/3.4</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion:	Time Critical Task: NO
Method of testing: Performance: Simulated Actual	Location: <u>X</u> Simulator Plant

Task Standard:

Required Materials: LOP-AP-08, System Auxiliary Transformer SAT 142 Operation in Shutdown, Rev 25, 4/2/2002.

General References:

LOP-AP-08, System Auxiliary Transformer SAT 142 Operation in Shutdown LOP-AP-01, Unit Auxiliary Transformer Backfeeding Operations LOR PM01J-A317, SAT 142 Trouble Alarm LOA-TRAN-101, Unit 1 Transformer Trouble, B13, Combustible Gas Monitor

#### READ TO THE EXAMINEE:

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. Inform me when you successfully completed the task.

#### Initial Conditions:

Unit 1 is shutdown (Mode 4). All electrical loads are powered from SAT 142. U1 Main Generator Disconnect Links open and UAT 141 Transformer Disconnect Links to Switchyard are closed. Backfeeding operations had previously been completed IAW LOP-AP-01, Unit Auxiliary Transformer Backfeeding Operations. UAT 141 is available for backfeeding.

#### en lating Cue:

Monitor Panel PM01J for alarms. Inform your supervisor when this task has been completed.

Simulator Setup Instructions:

- 1. Reset Simulator
- 2. Set up Simulator for Mode 4 with Unit 2 at full power.
- 3. All Unit 1 loads powered from SAT 142. ACBs 1415 & 1425 are closed.
- 4. Backfeed operations for UAT141 have been completed, Generator Output disconnect open, UAT141 Transformer Output disconnect closed.
- 5. When cued by examiner, input annunciator SAT 142 Trouble alarm (PM01J-A317).
- 6. After a short period of time, call and inform candidate that SAT 142 has a high gas concentration. Fifth LED OFF (Confirms very high gas reading). Other LED status as follows: first LED: blinking yellow, second LED ON (green), third LED ON (green), fourth LED ON (green). If asked, sixth LED is OFF (high gas reading). Gas level is \_\_\_\_\_(?)

#### **INITIAL CONDITIONS**

(To be read by examinee)

Initial Conditions:

Unit 1 is shutdown (Mode 4). All electrical loads are powered from SAT 142. U1 Main Generator Disconnect Links open and UAT 141 Transformer Disconnect Links to Switchyard are closed. Backfeeding operations had previously been completed IAW LOP-AP-01,Unit Auxiliary Transformer Backfeeding Operations. UAT 141 is available for backfeeding.

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Initiating Cue: Monitor Panel PM01J for alarms.

(Denote critical steps with an asterisk) Comment JNSAT SAT **Expected Response** Element Candidate reviews annunciator Annunciator PM01J-A317 1. response procedure for alarm. alarms. Refers to annunciator response Candidate calls simulator operator and \*2. dispatches auxiliary operator to SAT procedure. Requires that an 142. operator be DISPATCHED to SAT 142. CUE: Receives a call from operator stating that the Combustible Gas Monitor is Alarming. On Panel PM01J: SAT 142 indicates **REFERS to LOA-TRAN-101:** 3. energized CHECK SAT 142 energized с Directs NLO to perform actions. **REMOVE** Outer Monitor Cover 4. and locate 6 small LEDs (NLO reports that 5<sup>th</sup> LED is ON) 5. CHECK fifth LED OFF 1<sup>st</sup> LED blinking yellow: 2<sup>nd</sup>, 3<sup>rd</sup> & 4<sup>th</sup> **VERIFY** the Combustible Gas 6. LEDs are ON (green) (Normal Monitor is operating properly: conditions). CHECK status of LEDs:1st LED blinking vellow: 2<sup>nd</sup>, 3<sup>rd</sup> & 4<sup>th</sup> LEDs are ON (green) CONTACT System Engineering CUE: Shift Supv to contact System 7. with Gas Level. Engineering Candidate RECOMMENDS removing \*8. IMMEDIATELY REMOVE SAT loads from SAT 142 142 from service

CUE: Inform candidate to transfer electrical loads from SAT 142 to UA1 141 by establishing a backfeed IAW LOP-AP-08, "SAT 142 Operation in Shutdown," step E.1.

(Denote critical steps with an asterisk)

#### <u>Element</u>

#### Expected Response

9. VERIFY UAT 141 ready to supply power for backfeeding per LOP-AP-01. Candidate may verify 345 KV bkrs CLOSED: OCB 1-9, OCB 9-10, OCB 10-11, OCB 11-13, OCB 1-13.

(Denote critical steps with an asterisk)

NOTE: Allow candidate to locate and review copy of LOP-AP-08.

#### Element

\*8. Transfer Bus 152 from SAT to UAT: VERIFY CLOSED ACB 1522. SYNCHRONIZE & CLOSE ACB 1521. OPEN ACB 1522.

Transfer Bus 151 from

VERIFY CLOSED ACB

SYNCHRONIZE & CLOSE

SAT to UAT:

ACB 1511.

OPEN ACB 1512.

1512.

#### Expected Response

SYNCHRONIZE ACB:

- Engage Synchroscope for oncoming ACB.
  - Verify synchroscope at 12:00 position, incoming & running voltages equal and sync lights NOT illuminated.
- Verify 6.9 KV Paralleled Bus Alarm annunciates.
- Open off going circuit bkr.
- Verify electrical bus parameters are normal
- Disengage the synchroscope.
- **Reset Parallelled Bus Annunciator**

ACB 1521 indicate closed. ACB 1522 indicates open.

SAT UNSAT

SYNCHRONIZE ACB:

- Engage Synchroscope for oncoming ACB.
  - Verify synchroscope at 12:00 position, incoming & running voltages equal and sync lights NOT illuminated.
- Verify 6.9 KV Paralleled Bus Alarm annunciates.
- Open off going circuit bkr. ٠
- Verify electrical bus parameters are normal •
- Disengage the synchroscope.
- **Reset Parallelled Bus Annunciator**

#### ACB 1511 indicate closed, ACB 1512 indicates open.

SAT\_\_\_\_\_ UNSAT \_\_\_\_\_

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10. TERFORM LOS-AP-01.

CUE Step E.1.4, LOS-AP-R5, Alternate Power Source Breaker Operability Surveillance is NOT required. 

\*9.

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(Denote critical steps with an asterisk)

#### <u>Element</u>

\*11. Transfer Bus 141Y & 141X from SAT to UAT: VERIFY CLOSED ACB 1415, SYNCHRONIZE & CLOSE ACB 1411, OPEN ACB 1412

#### Expected Response

SYNCHRONIZE ACB:

- Engage Synchroscope for oncoming ACB.
  - Verify synchroscope at 12:00 position, incoming & running voltages equal and sync lights NOT illuminated.
- Verify 6.9 KV Paralleled Bus Alarm annunciates.
- Open off going circuit bkr.
- Verify electrical bus parameters are normal
- Disengage the synchroscope.
- Reset Parallelled Bus Annunciator

ACBs 1415 & 1411 indicate closed, ACB 1412 indicates open.

SAT \_\_\_\_\_ UNSAT \_\_\_\_

\*12. Transfer Bus 142Y & 142X from SAT to UAT: VERIFY CLOSED ACB 1425, SYNCHRONIZE & CLOSE ACB 1421, OPEN ACB 1422. SYNCHRONIZE ACB:

- Engage Synchroscope for oncoming ACB.
  - Verify synchroscope at 12:00 position, incoming & running voltages equal and sync lights NOT illuminated.
- Verify 6.9 KV Paralleled Bus Alarm annunciates.
- Open off going circuit bkr.
- Verify electrical bus parameters are normal
- Disengage the synchroscope.
- Reset Parallelled Bus Annunciator

ACBs 1425 & 1421 indicate closed, ACB 1422 indicates open.

SAT UNSAT

CUE: This completes this JPM

Completion Time: \_\_\_\_\_

# VERIFICATION OF COMPLETION

Job Performance Measure No			
Examinee's Name:			
Examiner's Name:			
Date performed:			
Number of attempts:			
Time to complete:			
Question Documentation:			
Question:			 
Response:			 
Result: SAT or UNSAT	:: • •		
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Examiner's signature and date:	-		
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# LASALLE COUNTY NUCLEAR STATION

# **JOB PERFORMANCE MEASURE**

Reestablish Shutdown Cooling After Clearing Level 3 PCIS Signal

Examination Level: RO /SRO /SRO(I)

Date: April 8, 2002

Developed by: Raymond Keith Walton Date: \_\_\_\_\_

Reviewed	by:	Date:	
<b>Approve</b>	: : :	Date:	

.

Facility: LaSalle Nuclear Station	Task No: <u>B.1.g</u>
Task Title: Reestablish Shutdown Cooling After PCIS Le	evel 3 Signal Cleared
K/A Reference: <u>20500A2.06</u>	K/A Importance: <u>3.4/3.5</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion:	Time Critical Task: NO
Method of testing: Performance: Simulated X Actual	Location: <u>X</u> Simulator Plant
Task Standard:	
Required Materials: LOA-RH-101, RHR Abnormal, Rv. 4, 12/5/2000 LOP-RH-07, Shutdown Cooling System Startup, Operat General References: LOA-RH-101, RHR Abnormal, Rv. 4, 12/5/2000 LOP-RH-07, Shutdown Cooling System Startup, Operat	tion and Transfer, Rv. 46, 10/27/2000
I will explain the initial conditions, which steps to simula me when you have successfully complete the task.	te or discuss, and provide initiating cues. Tell
Initial Conditions: Plant is in Mode 4. A spurious PCIS Level signal 3 had tripping and a loss of shutdown cooling. The PCIS leve of service for maintenance. Both trains of CS are opera	I 3 signal has been reset. The 'B' RHRSW is out
Initiating Cue: The Shift Manager has requested you reestablish shute 07, Shutdown Cooling System Startup, Operation and has been completed.	Lown cooling on 'A' train of RHR IAW LOD-RH- manster. Inform your supervisor when this task

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Simulator Setup Instructions:

- 1. Setup Simulator for Mode 4 operations: Reactor temperature 120 degrees F, RPV pressure 100 psig, RPV water level at 50" and both recirculation pumps operating at 15 hz.
- 2. SDC valves associated with 'A train RHR have been closed by PCIS level 3 and remain closed.
- 3. Plant conditions as shown:
  - 'B' RHRSW in operation
  - 'A' RHRSW system OOS (OOS tag hung on 'B' RHRSW pump control sw)
- 3. Input a failure for 'A' RHR pump to start.

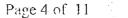
#### INITIAL CONDITIONS

Initial Conditions:

Plant is in Mode 4. A spurious PCIS Level 3 signal had occurred which resulted in 'A' RHR pump tripping and a loss of shutdown cooling. The PCIS level 3 signal has been reset. The 'B' RHRSW is out of service for maintenance. Both trains of CS are operable. The 'A' RHRSW system is in operation.

Initiating Cue:

The Shift Manager has requested you reestablish shutdown cooling or 'A' train of RHR IAW LOP-RH-07, Shutdown Cooling System Startup, Operation and Transfer. Inform your supervisor when this task has been completed.



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(Denote critical steps with an asterisk)

			SAT	UNSAT	Comment #
	Element	Expected Response			
<b>1.</b>	Refer to LOP-RH-07, Shutdown Cooling System Startup, Operation and Transfer.	Locate and Open LOP-RH-07. Start at step E.1. CUE: RHR & SDC systems have been filled & vented. Cycled Condensate is in operation and RBEDT is available.			
2.	DECLARE LPCI Mode of 'A' RHR inoperable.	CUE: Shift Manager notified.			
3.	CLOSE valves: 1E12-F004A, A RHR Pmp Suction 1E12-F064A, A RHR In Flow	1E12-F004A and 1E12-F064A valves indicate closed.			
*4.	OPEN valve 1E12-F006A, A RHR shutdown cooling suction valve.	1E12-F006A valve indicates open	—		
5.	LOCALLY CLOSE 1E12-F085A, A RHR/LPCS Water Leg Pump Discharge Valve.	CUE: Operator reports 1E12-F085A valve closed.	—		
<b>6.</b>	VERIFY OPEN 1E12-F048A, A RHR Hx Bypass VIv.	1E12-F048A valve indicates open.	. <u> </u>	—	 /Ω
*7.	VERIFY CLOSED: 1E12-F047A, A RHR Hx Inlet VIv 1E12-F003A, A RHR Hx Outlet VIv	1E12-F047A and 1E12-F003A valves indicate closed.		<u> </u>	<u></u>

Page 5 of 11

(Denote critical steps with an asterisk)

	Element	م <u>Expected Response</u>	SAT	UNSAT	Comment #
	NLOCK and CLOSE bkr for vlv E12-F008 at 135X-1.	CUE: 1E12-F008 chain operator unlocked and removed. Bkr for 1E12-F008 valve previously closed.			
Co 1E	VERIFY OPEN valves: E12-F008, RHR Shutdown ooling Suction Inboard and E12-F009, RHR Shutdown ooling Suction Outboard Isolation	Valves 1E12-F008 and 1E12-F009 indicate closed. Operator opens F008 and F009 valves.			
10.	Start RHRSW pump.	Should verify RHRSW 'A' pump operating.			
11.	If reactor water temperature exceeds 'A' RHR Hx outlet temperature by 100 degrees F, perform warmup procedure.	Cue: If necessary, warmup procedure waived by Shift Manager.			_
12.	CHECK both recirculation pumps in operation. Both recirc suction and discharge valves OPEN.	Verifies both recirulation pumps in operation with suction and discharge valve open.		 	 . ·
*13.	OPEN 1E12-F047A, 'A' RHR Hx Inlet Vlv.	Opens 1E12-F047A.			
14.	VERIFY OPEN 1E12-F048A, 'A' RHR Hx Bypass VIv.	1E12-F048A indicates open.			· · · · · · · · · · · · · · · · · · ·
				• •	· · · · · · · · · · · · · · · · · · ·
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		Page 6 of 11			• ::

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	Standard	Expected Response	SAT	UNSAT	Commont #	
Co	VERIFY CLOSED VIvs: E12-F053A, 'A' Shutdown poling Return Isolation VIv. E12-F003B, 'A' RHR Hx Outlet v.	1E12-F053A & 1E12-F003A indicate closed.				
16.	VERIFY Reactor Level >40"	Verifies Reactor Level >40"			_	
*17.	Attempts to Start 'A' RHR pump. Identifies that pump does <u>NOT</u> start	Candidate attempts to start pump. Verifies by looking at Pump amps, flow rate and pump breaker indication light that pump does <b>NOT</b> start.				
*18.	DETERMINES pump inoperable and trips 'A' RHR pump.	Pump switch in OFF position				
NOT	E TO EXAMINER: Initiate Alterr alternative method of removing	nate Path portion of JPM. Goal is to e g decay heat from core.	stabli	ish ar	)	
*19.	REFERS to LOA-RH-101, "U1 RHR Abnormal," Section B.1, step 4.5.1.	Recommends another heat removal method.				
NOT	recommend 'A' SDC (drain p recommends another metho perform LOP-RH-17, "Altern	y heat removal are prioritized. Applic bath) and LPCS (charging path). If can be of SDC, as Shift Supervisor, have c ate Shutdown Cooling?" using 'A' train is return. Hand candidate Attachment or steps E.16 and E.17).	ndida andic 1 shu	te late tdowi	ne in Ne trian	1
20.	Reviews Attachment A of LOP-RH-17.	Reviews Attachment A of LOP- RH-17.	• •		··· <i>i</i>	
		Page 7 of 11			· · · ·	

(Denote critical steps with an asterisk)

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			SAT	UNSAT	Comment #
	<u>Element</u>	Expected Response			
*21	OPEN E12-F047A, 'A' RHR Hx Inlet Valve.	E12-F047A indicates OPEN		- ,	
22.	VERIFY CLOSED: E12-F003A, 'A' RHR Hx Outlet valve	E12-F003A, indicates closed			<u></u>
	E12-F004A, 'A' RHR pump Suction Valve	E12-F004A, indicates closed.		·	<del></del>
	E12-F065A, 'A' RHR Hx level Controller	E12-R604A in Manual and full closed.			
23.	VERIFY: E12-F051A, 'A' RHR Hx Press. Controller E12-R605, RHR Hx Drain to RCIC Suct. Press Controller.	Switch E12-R606A in Manual and full closed. Switch E12-R605 in Manual and full closed			
24.	PLACE applicable air supply isolation switch (no EPN) in OPEN: E12-F065A/51A.	E12-F065A/51A air supply switch in OPEN			
*25	VERIFY the following valves are CLOSED:				
	E12-F074A, 'A' RHR Hx vent	E12-F074A indicates closed.			
	Upstream valve E12-F073A, 'A' RHR Hx vent Downstream valve	E12-F073A indicates closed.	-		· _ · ·
	E12-F052A, 'A' RHR Hx Steam inlet valve	E12-F052A indicates closed.	. 4 . 1 - 1 . 1 - 1 . 1 - 1	LE 45	
	1:12-F048A, 'A' RHR H Typase valve				
	E12-F <b>026A, 'A'</b> RHR H× Drain	E12-F026A indicates closed.		·	
	to RCIC.				

(Denote crtical steps with an asterisk)

			SAT	UNSAT	Comment #
	<u>Element</u>	Expected Response			
*26.	OPEN E12-F020, RHR SDC Suct. Header Manual Stop VIv.	Orders NLO to Open E12-F020.		. <u> </u>	
*27.	OPEN E12-F008 & E12-F009, RHR SDC Suct.Outboard and Inbcard Isol Valves.	E12-F008 and E12-F009 Valves indicate open.			
*28.	READS NOTE in LOP-RH-17. Must keep drain flow >450 gpm and must read Computer digital points to determine drain flow rate.	Accesses computer points A800.			
*29.	OPEN E12-F006A, "A" RHR SDC Suction valve.	Valve E12-F006A indicate open			
30.	START 'A' RHR Service Water.	Verifies 'A' RHRSW operating.		. <u> </u>	
31.	NOTIFY RP of changing dose rates.	CUE: SS will notify RP of changing dose rates.		. <u> </u>	
*32.	OPERATE LPCS pump per LOP-LP-05: Verify open DG035, LPCS	CUE: Time does not permit performing steps E.1 and E.2.	•		
Pump Cooler Isol. Valve. START LPCS pump E21-C0		DG035 indicates open. LPCS pump indicates ON			
	Test to SP valve Establish flow rate between	E21-F012 indicates open			
	6000 gpm to 6500 gpm.	Flow rate established between 6000 gpm and 6500 gpm.			. <u> </u>

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(Denote critical steps with an asterisk)

			SAT	UNSAT	Comment
	<u>Element</u>	Expected Response			
*32.	OPERATE LPCS Pump per LOP-LP-05, (Cont'd)				
	VERIFY E21-F011, LPCS Pump Min Flow valve closes.	E21-F011 indicates Closed.			
		0 EDG Cooling Water Pump running.		<u> </u>	<u> </u>
		Local Operator calls from local panel and states all equipment is running.		<u> </u>	<u> </u>
	Throttle Open E21-F005, LPCS Injection Valve.	E21F005 throttled open.			
*33.	IF using E12-F024A, 'A' RHR Test to SP VIv. as a drain path: CLOSE E12-F048A, 'A' RHR Hx	E12E048A indicates closed			
	Bypass Valve. OPEN E12-F003A, 'A' RHR Hx Outlet Valve.				
*34.	BEGIN DRAINING by throttling OPEN E12-F024A, 'A' RHR Test to SP Valve.	E12F024A indicates open. Candidate monitors flow rate on computer point A800.			<u> </u>
*35.	OBTAIN desired cooldown rate of less than 100 °F/hr by	Establishes cooldown rate of <100°F/hr			
	adjusting drain flow.		-		
CUE	: THIS COMPLETES THIS JPM.	jain (Interior Color Color Color Color Color) C			
Reco	ord Stop Time:				

# VERIFICATION OF COMPLETION

Job Performance Measure No.			
Examinee's Name:			
Examiner's Name:			
Date performed:			
Number of attempts:			
Time to complete:			
Question Documentation:			<u></u>
Response:			
Result: SAT or UNSAT			
Result: SAT of UNSAT	· ·		
Examiner's signature and date:			
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## LASALLE COUNTY NUCLEAR STATION

## **JOB PERFORMANCE MEASURE**

Startup RPS MG Set and Transfer Power from Alternate Source to Normal Source

Examination Level: RO /SRO /SRO(I)

Date: April 8, 2002

Developed by: <u>Raymond Keith Walton</u> Date: <u>3/14/2002</u>

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Reviewed by:	1771 p. 1774 p. 1944	Date:		
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Approved by:		Date:		··· · ·

CALLER THE STREET & GLASS MAL

Facility: LaSalle Nuclear Station	Task No: <u>B.2.a</u>
Task Title: Startup RPS MG Set and Transfer Power fr	rom Alternate Source to Normal Source
K/A Reference: <u>212000A2.02</u>	K/A Importance: <u>3.7/3.9</u>
Examinee:	
NRC Examiner:	Date:
Time Started:	Time Finished:
Estimated Time to Completion:	Time Critical Task: NO
Method of testing: Performance: X Simulated Actual	Location: Simulator _X Plant
Task Standard:	
Required Materials: LOP-RX-01, RPS MGS Set Startup and Operation LOP-RX-03, RPS Bus A Transfer	
General References: LOP-RX-01, RPS MGS Set Startup and Operation LOP-RX-03, RPS Bus A Transfer	
READ TO THE EXAMINEE: I will explain the initial conditions, which steps to simula me when you have successfully completed the task.	te or discuss, and provide initiating cues. Inform
Initial Conditions: Unit 1 is operating in Mode 1. Bus 'A' RPS power is on being performed which produce half RPS or half PCIS	
Initiating Cue: You have been instructed by the Shift Manager to Start from Alternate to Normal. Previously, an operator had	up the 'A' RPS MG set and transfer RPS power completed prerequisite steps for the LOP-OP-03

from Alternate to Normal. Previously, an operator had completed prerequisite steps for the LOP-OP-03 RPS MG Set Clartup Procedure. The same operator also completed prerequisite steps and steps E.1 through E.2 for RPS Bus 'A' Transfer Operating Procedure. Inform your supervisor when this task has been completed. Simulator Setup Instructions:

NONE: In-plant



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#### **INITIAL CONDITIONS**

(To be read by examinee)

Initial Conditions:

Unit 1 is operating in Mode 1. Bus 'A' RPS power is on Alternate supply. There are no surveillances being performed which produce half RPS or half PCIS acuations.

Initiating Cue:

You have been instructed by the Shift Manager to:

- Startup the 'A' RPS MG set IAW LOP-RP-01, "RPS MG Set Startup and Operation" and •
- Transfer RPS power from Alternate to Normal AW LOP-RP-03, "RPS Bus A Transfer." •

Previously, an operator had:

- completed prerequisite steps for LOP-RP-01, "RPS MG Set Startup Procedure" and
- completed prerequisite steps E.1 through E.7 for LOP-RP-03, "RPS Bus 'A' Transfer" Operating Procedure.

Inform your supervisor when this task has been completed.



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<b></b>	JOB PERI	FORMANCE MEASURE			
(De	enote critical steps with an asterisk)				
All plant operations are to be simulated.					nt #
CUE: Give candidate partially completed copies of LOP-RP-01 and LOP-RP-03			SAT	UNSAT	Comment
	Element	Expected Response			
1.	Refer to LOP-RP-01, RPS MG Set Startup and Operation and LOP-RP-03, RPS Bus A Transfer.	Locate and Open LOP-RP-01 Procedures. Candidate to perform steps E.1 and E.2 of LOP-RP-01.			
*2.	CLOSE feeder breaker 135X-2 Bkr A-5 to 'A' RPS MG set.	Candidate closes Feeder Breaker 135X-2, A-5.			<u>-</u>
*3.	START the 'A' RPS MG set.	Candidate starts the 'A' RPS MG set.			
4.	Check 'A' RPS MG set output breaker is OPEN.	'A' RPS MG set output breaker checked open.			
*5. •	PRESS and HOLD the 'A' RPS MG set Motor On pushbutton (1PB) at local 'A' RPS MG set control panel. Observe: Motor ON light illuminates Local volt meter indicates 120 VAC. RELEASE Motor ON pushbutton.	Candidate presses and holds 1PB to 'A' RPS MG set. CUE: Motor ON light illuminates Local Voltmeter indicates 120 VAC.			
*6.	CLOSE the 'A' RPS MG set output circuit breaker at the local MG set panel.	Candidate closes 'A' RPS MG set output breaker		. i	:. 
7.	Unock Electrical Protection Assembly (EPA) breaker 1C71-S003A Red Motor Gen light ON.	CUE: EPA bkr 1C71-S003A Red Motor Gen light ON.	· · ·		•
		Page 5 of 8			

# JOB PERFORMANCE MEASURE

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(Denote critical steps with an asterisk)

	<u>Sta</u>	andard	Expected Respo	<u>nse</u>	SAT	UNSAT	Comment
*8.		OSE EPA brea 71-S003A.	ker	CUE: EPA bkr 1C71-S003A breaker indicates closed	<u></u>		
9.	As	eck Electrical F sembly (EPA) b 71-S003A Red I.	oreaker	CUE: EPA bkr 1C71-S003A Red RPS Bus light ON.			
10.		Check Electric Assembly (EP 1C71-S003C I light ON.		CUE: EPA bkr 1C71-S003C Red Motor Gen light ON.			
*11	•	CLOSE EPA b 1C71-S003C.	preaker	CUE: EPA bkr 1C71-S003C breaker indicates closed			
12.		Check Electric Assembly (EP 1C71-S003C I light ON.	A) breaker	CUE: EPA bkr 1C71-S003C Red RPS Bus light ON.			
13.		Main Control F 1H13-P610 wh 'A' feed light il	nite Generator	CUE: 1H13-P610 white Generator 'A' feed light illuminated in MCR.			

Operator should transition from LOP-RP-04 to 1 OP-RP-03. Start at stop F 8 of NOTE: LOP-RP-03. Cancildate should N/A step L.o.

# JOB PERFORMANCE MEASURE

(Denote critical steps with an asterisk)

			SAT	UNSAT	Comment #
	<u>Element</u>	Expected Response			
14.	At 1H13-P610, VERIFY the GEN A Feed lamp is energized.	Feed Lamp Energized.			
15.	VERIFY plant conditions will allow a half scram and half isolation.	CUE: Plant conditions allow half RPS and half PCIS.			<del></del>
*16.	SLOWLY position the RPS BUS Power Supply Transfer Switch to NORM.	CUE: RPS BUS Power Supply Transfer Switch to NORM. Not necessary to RESET half scram or half PCIS, this will be done by another operator.			

(CUE) THIS COMPLETES THIS JPM Record Stop Time: \_\_\_\_\_

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# VERIFICATION OF COMPLETION

Job Performance Measure No		
Examinee's Name:		
Examiner's Name:		
Date performed:		
Number of attempts:		
Time to complete:		
Question Documentation:		
Response:		
Result: SAT or UNSAT		
Examiner's signature and date: _	Page 8 of 8	

# LASALLE COUNTY NUCLEAR STATION

# **JOB PERFORMANCE MEASURE**

Startup and Operation of RCIC from the Remote Shutdown Panel

Examination Level: RO /SRO /SRO(I)

Date: April 8, 2002

Developed by: <u>Raymond Keith Walton</u> Date: <u>3/14/2002</u>

Reviewed by: \_\_\_\_\_ Date: \_\_\_\_ 

Previoued by: \_\_\_\_\_\_ Date:\_\_\_\_\_

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# JOB PERFORMANCE MEASURE

Facility: LaSalle Nuclear Station	Task No: <u>B.2.b</u>						
Task Title: Startup and Operate RCIC from Remote Shutdown Panel							
K/A Reference: 295016 AA2.02	K/A Importance: <u>4.2/4.3</u>						
Examinee:							
NRC Examiner:	Date:						
Time Started:	Time Finished:						
Estimated Time to Completion:	Time Critical Task: NO						
Method of testing: Performance: X Simulated Actual	Location: <u>X</u> Simulator Plant						
Task Standard:							

#### **Required Materials:**

Copy of LOP-RX-04, Startup and Operation of RCIC from the Remote Shutdown Panel, Rev 12, 10/17/2000

General References:

Licensee JPM S-RX-05, Rev 6, 8/31/1999 LOP-RX-04, Startup and Operation of RCIC from the Remote Shutdown Panel, Rev 12, 10/17/2000

#### READ TO THE EXAMINEE:

I will explain the initial conditions, which steps to simulate or discuss, and provide initiating cues. Inform me when you have successfully complete the task.

#### Initial Conditions:

The Control Room has been evacuated. Control has been transferred to the Remote Shutdown Panel. Suppression Pool Cooling has been established from the Remote Shutdown Panel. U1 CY tank level has been locally verified to be above 15 feet. Plant operators are available.

#### Initiating Cue:

The Unit Supervisor has directed you to start RCIC from the Remote Shutdown Panel and inject and stabilize RDV level. Inform the Unit Supervisor when RCIC is injecting and level has stabilized. Inform your supervisor when this task has been completed.

Page 2 of 8

Simulator Setup Instructions:

- 1. Reset Simulator
  - a. Insert malfunction to prevent RCIC automatic initiation, (imf mnb080-150)
  - b. Place MDRFP control switch in PTL
  - c. Place HPCS in PTL.
  - d. Place the mode switch to Shutdown
  - e. Trip both TDRFPs and close the FW010A and FW010B valves
  - f. Place all Remote Shutdown Panel transfer switches to Emergency IAW LOA-RX-101.
  - g. Start Suppression Pool Coolin IAW LOP-RX-06
  - h. Stabilize reactor water level between -30 inches to -10 inches.
  - i. Verify the RCIC turbine trip light is NOT lit on RSP.
- 2. Freeze simulator until examinee has acknowledged the initiating cue(to prevent reactor water level from going too high or too low for this JPM to be performed)
- 3. Have a copy of LOP-RX-04 available to replace one used by examinee.

Initial Conditions:

- The Control Room has been evacuated.
- Control has been transferred to the Remote Shutdown Panel.
- Suppression Pool Cooling has been established from the Remote Shutdown Panel.
- U1 CY tank level has been locally verified to be above 15 feet.
- Plant operators are available.

#### Initiating Cue:

The Unit Supervisor has directed you to start RCIC from the Remote Shutdown Panel and inject and stabilize RPV level.

Inform the Unit Supervisor when RCIC is injecting and level has stabilized. Inform your supervisor when this task has been completed.

Page 4 of 8

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	JOB PER	FORMANCE MEASURE			
(De	note critical steps with an asterisk)				
NOTE: All steps of the JPM are to be completed at the <b>Unit 1 Remote Shutdown Panel</b> .				٨T	Comment #
Sim	ulator Operator: Place simulator in	RUN.	SAT	UNSAT	Comr
	<u>Element</u> <u>E</u>	xpected Response	·	<u> </u>	
1.	VERIFY RCIC flow controller is SET at 100%.	Examinee adjusts RCIC flow controller to 600 gpm.	<del></del>		
2.	VERIFY OPEN 1E51-F068, RCIC Turbine Exhaust Valve.	Examinee verifies valve is open.	—		
*3.	Open 1E51-F022, RCIC Full Flow Test Upstream Stop Viv.	Examinee places and holds 1E51- F022 control switch in OPEN until valve is fully open.	—	<u> </u>	
*4.	<ul> <li>CONCURRENTLY</li> <li>OPEN 1E51-F045, RCIC Turbine Steam Supply Stop Valve.</li> <li>OPEN 1E51-F059, RCIC Full Flow Test to CST Stop Valve.</li> </ul>	Examinee simultaneously turns both control switches to OPEN.	_		
5.	START RCIC Barometric Condenser Vacuum Tank Vacuum Pump	Examinee turns control switch to START.			
6.	CHECK that RCIC Pump flow is increasing as indicated on RCIC flow controller.	Examinee checks flow increasing on flow controller.			
*7.	OPEN 1E51-F046 RCIC Lube Oil Cooler Supply Stop Valve.	Examinee turns control switch to OPEN.			
*8.	OPEN 1E51-F022, RCIC Full Flow Test Upstream Stop Valve.	Exminee turns control switch to OPEN.		••	·
*9.	CLOSE 1E51-F059, RCIC Full Flow Test to CST Stop Valve.	Examinee places and holds 1E51- F022 control switch in CLOSE until valve is fully closed.			

# JOB PERFORMANCE MEASURE (Denote critical steps with an asterisk)

Si	andard Expected Response		SAT	UNSAT	Comment #
10.	ADJUST RCIC flow controller setpoint tape to increase or decrease flow to maintain desired level.	- Examinee leaves tape setpoint at 600 gpm (level is below designated band)			
11.	VERIFY that RCIC System flow indicates above 120 gpm.	Examinee verifies flow above 120 gpm.			
12.	CLOSE 1E51-F059, RCIC Full Flow Test to CST Stop Valve.	Examinee turns control switch to CLOSE.			
13.	ADJUST flow to the Reactor Vessel as necessary to maintain vessel level after closing 1E51- F022, RCIC Full Flow Test Upstream Stop Valve.	Examinee leaves tape setpoint at 600 gpm (Level is below designated band).			
14.	OPERATE the Condensate Vacuum Tank Condensate Pump 1E51-C004 as necessary to keep the vacuum tank high level and low level lights cleared.	Examinee STARTS/STOPs pump as necessary to keep the lights cleared.			
15.	NOTIFY Unit Supervisor RCIC is injecting and level is stable.	Examinee notifies Unit Supervisor.		<b>8</b>	
*16	Have candidate locate remote shutdown panel in plant	Candidate locates remote shutdown panel in plant.			
	: THIS COMPLETES THIS JPM.	loric cigns ture s			
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# VERIFICATION OF COMPLETION

Job Performance Measure No.			
Examinee's Name:			
Examiner's Name:			
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Time to complete:			
Question Documentation:			
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#### PREPARATION OF THE REMOTE SHUTDOWN SYSTEM FOR STANDBY OPERATION

#### A. <u>PURPOSE</u>

Outline the steps necessary to prepare the Remote Shutdown Panel for standby operation.

#### B. <u>REFERENCES</u>

- 1. P & ID's:
  - a. M-96 (M-142), Residual Heat Removal (RHR).
  - b. M-101 (M-147), Reactor Core Isolation Cooling (RCIC).
  - c. M-68 Service Water (WS).
  - d. M-93 (M-139), Nuclear Boiler and Reactor Recirculation.
  - e. M-87 (M-134), CSCS Equipment Cooling Water System.
  - f. M-90 (M-137), Reactor Building Closed Cooling Water (RBCCW).
  - g. M-55 (M-116), Main Steam.
- 2. Electrical Schematics.
  - a. 1E-1(2)-4214, Remote Shutdown RS (C61).
  - b. 1E-1(2)-4226, Reactor Core Isolation Cooling RI (E51).
  - c. 1E-1(2)-4220, Residual Heat Removal RH (E12).
  - d. 1E-1(2)-4201, Auto Depressurization System.
  - e. 1E-1(2)-4205, Reactor Recirc Pump Suction Valve.
  - f. 1E-0-4449, Service Water Jockey Pump.
  - g. 1E-1(2)-4097, Service Water 1WS113 Valve.
  - h. 1E-1(2)-4096, Reactor Building Closed Cooling Water.
  - i. 1E-1(2)-4033, Fire Detection.
- 3. TRM Appendix F

Level of Use Reference

1 of 2

#### LOP-RX-01 Revision 4 September 14, 2001

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#### C. <u>PREREQUISITES</u>

I. None.

#### D. <u>PRECAUTIONS</u>

- 1. Taking a Remote Shutdown Transfer switch to the "Emergency Position", provides system control at the Remote Shutdown Panel while disabling normal Control Room control.
- 2. In the "Emergency Position", Automatic system initiations, Primary Containment Isolation functions, and Automatic protection of the associated components is lost.
- 3. Taking a Remote Shutdown Transfer switch to the "Emergency Position" will cause a visual and audible alarm in the Control Room.

#### E. <u>LIMITATIONS</u>

1. The Remote Shutdown Monitoring Instrumentation shall be operable as specified in Tech Spec 3.3.2, TRM Appendix F.

#### F. <u>PROCEDURE</u>

- 1. VERIFY that the Remote Shutdown Panel is lined up in accordance with LOP-RX-01E(02E).
- 2. VERIFY that the RCIC Flow Controller at the Remote Shutdown Panel is set at:
  - a. 600 gpm.
  - b. AUTO.
- 3. VERIFY that valve and pump control switches located on the Remote Shutdown Panel are in accordance with LOP-RX-01T.

N E L

#### G. CHECKLISTS

1. None.

## H. <u>TECHNICAL SPECIFICATION REFERENCES</u>

1. Tech Spec 3.3.3.2, Remote Shutdowri Monitoring System: <u>Spec Spectrum and</u>

2 of 2

Level of Use Reference

LOP-RX-01 Revision 4 Soptember 14, 2001

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#### TRANSFER OF REACTOR SUPERVISORY INDICATION TO THE REMOTE SHUTDOWN PANEL

#### A. <u>PURPOSE</u>

A.1 <u>Objective</u>

Outline the steps necessary to transfer reactor supervisory indication to the Remote Shutdown Panel.

#### B. <u>PREREQUISITES</u>

B.1 The Remote Shutdown System is prepared for standby operation in accordance with LOP-RX-01, Preparation of the Remote Shutdown System for Standby Operation.

#### C. <u>PRECAUTIONS</u>

C.1 Transfer of reactor pressure, level, and RCIC alarms are from the same transfer switch.

#### D. <u>LIMITATIONS</u>

- D.1 Supervisory Indications referred to by this procedure are:
  - Wide Range Pressure, 0 to 1500 psig.

#### E. <u>PROCEDURE</u>

E.1 PLACE Remote Shutdown Transfer Switch S-18; Nuclear Boiler Instruments, to Emergency.

#### F. <u>REVIEW AND SIGNOFF</u>

F.1 None

#### G. <u>REFERENCES</u>

- G.1 M-93 (M-139); Nuclear Boiler and Reactor Recirculation.
- G.2 Electrical Schematic; 1E-1(2)-4212; Remote Shutdown System.
- G.3 TRM Appendix F
- G.4 <u>Technical Spec References</u>
- G.4.1 3.3.3.2, Remote Shutdown Monitoring System

Level of Use Reference

#### 1 of 1

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LOP-RX-02 Revision 3 September 14, 2001