July 21, 1994

Docket No. 50-397

Mr. J. V. Parrish (Mail Drop 1023)
Assistant Managing Director, Operations
Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352-0968

Dear Mr. Parrish:

SUBJECT: ISSUANCE OF AMENDMENT CORRECTION FOR THE WASHINGTON PUBLIC POWER SUPPLY SYSTEM NUCLEAR PROJECT NO. 2 (TAC NO. M88838)

The Commission issued Amendment No. 128 on July 8, 1994, to the Facility Operating License No. NPF-21 for WPPSS Nuclear Project No. 2. The amendment changed the Technical Specifications (TS) in response to your application dated December 6, 1993, and supplemental letter dated May 6, 1994.

Through a telephone conference on July 13, 1994, you identified an administrative error that had been made in issuing Amendment 128. The error and correction are as follows:

> Page 3/4 3-74, Item 9, entitled "Drywell Hydrogen Concentration," was issued with an "R" in the Channel Calibration column. The "R" should have remained a "Q" as it was originally.

The enclosed revised TS page is issued with the appropriate overleaf page 3/4 3-73.

We apologize for any inconvenience this might have caused you.

Sincerely,

ORIGINAL SIGNED BY:

James W. Clifford, Senior Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Enclosure:
Revised Page 3/4 3-74

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JClifford	MShuiabi	KPerkins, RIV/WCFO

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DATE	7/21/94	7/21/94	7/21/94

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Mr. J. V. Parrish
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

cc:
Mr. J. H. Swailes
WNP-2 Plant Manager
Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352-0968

G. E. C. Doupe, Esq. (Mail Drop 396)
Washington Public Power Supply System
3000 George Washington Way
Richland, Washington 99352-0968

Mr. Warren Bishop, Chairman
Energy Facility Site Evaluation Council
P. O. Box 43172
Olympia, Washington 98504-3172

Mr. H. K. Kook (Mail Drop PE20)
WNP-2 Licensing Manager
Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352-0968

Mr. Paul R. Bemis (Mail Drop PE20)
Regulatory Programs Manager
Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352-0968

Regional Administrator, Region IV
U. S. Nuclear Regulatory Commission
Harris Tower & Pavilion
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064

Chairman
Benton County Board of Commissioners
P. O. Box 69
Prosser, Washington 99350-0190

Mr. R. C. Barr
U. S. Nuclear Regulatory Commission
P. O. Box 69
Richland, Washington 99352-0968

M. H. Philips, Jr., Esq.
Winston & Strawn
1400 L Street, NW.
Washington, DC 20005-3502

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Revised Page 3/4 3-74

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See next page

Table 3.3.7.5-1 (Continued)

ACCIDENT MONITORING INSTRUMENTATION

ACTION STATEMENTS

ACTION 80 -

a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.

b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

ACTION 81 - With the number of OPERABLE accident monitoring instrumentation channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable channel(s) to OPERABLE status within 72 hours, or:

a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and

b. In lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

ACTION 82 - With the number of OPERABLE Safety/Relief Valve Position Indicator instrumentation channels less than the Minimum Channels OPERABLE requirement of Table 3.3.7.5-1,

a. Restore an inoperable channel to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours, and

b. Verify operability and perform daily surveillance of the Tailpipe Temperature Monitoring instrument for the affected SRV until the Minimum Channels OPERABLE requirement is satisfied. Absent an OPERABLE Tailpipe Temperature monitor for the affected SRV restore the inoperable Tailpipe Temperature Monitor to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

TABLE 4.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION	APPLICABLE OPERATIONAL CONDITIONS
1. Reactor Vessel Pressure	M	R	1,2
2. Reactor Vessel Water Level	M	R	1,2
3. Suppression Chamber Water Level	M	R	1,2
4. Suppression Chamber Water Temperature	M	R	1,2
5. Suppression Chamber Air Temperature	M	R	1,2
6. Primary Containment Pressure	M	R	1,2
7. Drywell Air Temperature	M	R	1,2
8. Drywell Oxygen Concentration	M	R	1,2
9. Drywell Hydrogen Concentration	M	Q	1,2
10. Safety/Relief Valve Position Indicators*	M	R#	1,2
11. Suppression Chamber Pressure	M	R	1,2
12. Condensate Storage Tank Level	M	R	1,2
13. Main Steam Line Isolation Valve Leakage Control System Pressure	M	R	1,2
14. Neutron Flux:			
APRM	M	R	1,2
IRM	M	R	1,2
SRM	M	R	1,2
15. RCIC Flow	M	R	1,2
16. HPCS Flow	M	R	1,2
17. LPCS Flow	M	R	1,2

*This includes acoustic monitor, valve stem position, and tailpipe temperature instrument channels.

#The provisions of Specification 4.0.4 are not applicable provided the surveillance is performed within 12 hours after reactor steam pressure and flow are adequate to perform the test.