June 10, 1993

Docket No. 50-397

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
3000 George Washington Way
P. O. Box 968
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT FOR THE WASHINGTON PUBLIC POWER SUPPLY SYSTEM NUCLEAR PROJECT NO. 2 (TAC NO. M82296)

The Commission has issued the enclosed Amendment No.116 to the Facility Operating License No. NPF-21 for WPPSS Nuclear Project No. 2. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated October 15, 1991, as supplemented by letters dated May 20, 1992 and July 28, 1992.

The amendment requested modification to the TS to change the surveillance test intervals and allowable outage times for the reactor core isolation cooling (RCIC) system actuation instrumentation.

A copy of the related Safety Evaluation is also enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
Original signed by:
James W. Clifford, Senior Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No.116 to NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

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NAME	DFoster	JClifford:lh		TQuay
DATE	5/11/93	5/14/93	5/19/93	6/10/93

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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cc w/enclosures:
See next page

Mr. G. C. Sorensen
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 116
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Washington Public Power Supply System (licensee) dated October 15, 1991 and supplemented by letters dated May 20, 1992 and July 28, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 116 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Theodore R. Quay

Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 3-48	3/4 3-48
3/4 3-49	3/4 3-49
3/4 3-51	3/4 3-51

3/4.3.5 REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.5 The reactor core isolation cooling (RCIC) system actuation instrumentation channels shown in Table 3.3.5-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.5-2.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2 and 3 with reactor steam dome pressure greater than 150 psig.

ACTION:

a. With a RCIC system actuation instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.5-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.

b. With one or more RCIC system actuation instrumentation channels inoperable, take the ACTION required by Table 3.3.5-1.

SURVEILLANCE REQUIREMENTS

4.3.5.1 Each RCIC system actuation instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.5.1-1.

4.3.5.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.

TABLE 3.3.5-1

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION

FUNCTIONAL UNITS	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM(a)	ACTION
a. Reactor Vessel Water Level - Low Low, Level 2	2	50
b. Reactor Vessel Water Level - High, Level 8	2(b)	51
c. Condensate Storage Tank Water Level - Low Low	2(c)	52
d. Manual Initiation	1(d)	53

(a) A channel may be placed in an inoperable status for up to 6 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.

(b) One trip system with two-out-of-two logic.

(c) One trip system with one-out-of-two logic.

(d) One trip system with one channel.

TABLE 3.3.5-1 (Continued)

REACTOR CORE ISOLATION COOLING SYSTEM

ACTUATION INSTRUMENTATION

ACTION STATEMENTS

ACTION 50 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement:

a. For one trip system, place the inoperable channel(s) and/or that trip system in the tripped condition within 24 hours or declare the RCIC system inoperable.

b. For both trip systems, declare the RCIC system inoperable.

ACTION 51 - With the number of OPERABLE channels less than required by the minimum OPERABLE channels per Trip System requirement, restore the number of operable channels to that required by the minimum OPERABLE channels per Trip System requirement within 24 hours or declare the RCIC system inoperable.

ACTION 52 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip System requirement, place at least one inoperable channel in the tripped condition within 24 hours or declare the RCIC system inoperable.

ACTION 53 - With the number of OPERABLE channels one less than required by the Minimum OPERABLE Channels per Trip System requirement, restore the inoperable channel to OPERABLE status within 24 hours or declare the RCIC system inoperable.

TABLE 3.3.5-2

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

FUNCTIONAL UNITS	TRIP SETPOINT	ALLOWABLE VALUE
a. Reactor Vessel Water Level - Low Low, Level 2	$\geq$ - 50 inches*	$\geq$ -57 inches
b. Reactor Vessel Water Level - High, Level 8	$\leq$ 54.5 inches*	$\leq$ 56 inches
c. Condensate Storage Tank Level - Low Low**	$\geq$ 448 ft 3 in. elevation (1 ft 9 in. tank level)	$\geq$ 448 ft 0 in. elevation (1 ft 6 in. tank level)
d. Manual Initiation	N.A.	N.A.

*See Bases Figure B 3/4 3-1.

**Provides automatic transfer from Condensate Storage Tank to the Suppression Pool.

TABLE 4.3.5.1-1

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNITS	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION
a. Reactor Vessel Water Level - (Low Low, Level 2)	S	Q	R
b. Reactor Vessel Water Level - High, Level (8)	S	Q	R
c. Condensate Storage Tank Level - Low	S	Q	R
d. Manual Initiation	N.A.	R	N.A.

3/4.3.6 CONTROL ROD BLOCK INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.6. The control rod block instrumentation channels shown in Table 3.3.6-1 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3.6-2.

APPLICABILITY: As shown in Table 3.3.6-1.

ACTION:

a. With a control rod block instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3.6-2, declare the channel inoperable until the channel is restored to OPERABLE status with its trip setpoint adjusted consistent with the Trip Setpoint value.

b. With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, take the ACTION required by Table 3.3.6-1.

SURVEILLANCE REQUIREMENTS

4.3.6 Each of the above required control rod block trip systems and instrumentation channels shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations for the OPERATIONAL CONDITIONS and at the frequencies shown in Table 4.3.6-1.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 116 TO FACILITY OPERATING LICENSE NO. NPF-21

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated October 15, 1991, and supplemented by letters dated May 20, 1992, and July 28, 1992, Washington Public Power Supply System submitted a request for changes to the Technical Specifications (TS) for Washington Nuclear Project No. 2 (WNP-2). The proposed change for the Emergency Core Cooling System (ECCS) and the reactor core isolation cooling (RCIC) system instrumentation increases the Allowable Outage Time (AOT) for the surveillance test from 2 hours to 6 hours and specified a time limit for the AOT for repair to 24 hours. Also Surveillance Test Intervals (STIs) have been increased from monthly to quarterly. The staff in its safety evaluation report (memo from S. Newberry to J. Dyer, dated July 15, 1991) had accepted the changes for the ECCS instrumentation and had denied the changes for the RCIC instrumentation. The reason for the staff's denial was that the GE Topical Report GENE-770-06-2 was under staff review at that time. Since then the NRC has accepted the GE Topical Report on a generic basis and each licensee was required to show the specific applicability of this report to their plant. Also each licensee was required to confirm that any increase in instrument drift due to the extended STIs is properly accounted for in the setpoint calculation methodology.

2.0 EVALUATION

By letter dated October 15, 1991, WPPSS resubmitted the request for the proposed changes to the TS for the RCIC instrumentation. In this request the licensee has stated that they reviewed the setpoint drift characteristics and confirmed that the setpoint will remain within existing allowances for the requested STI extension. The licensee has documented this analysis and it is available for future staff audit. This satisfies one of the conditions set in the staff's SER of the Topical Report.

The licensee submittal did not address the second condition which required the licensee to confirm the applicability of the generic analysis to WNP-2. By letter dated May 20, 1992, the licensee confirmed the applicability of the generic report to WNP-2 by an analysis performed by GE and documented in a GE Report RE-024, DRF A00-02558 dated March 1987. This report compares the RCIC

system design, support systems and instrumentation between the plant specific configuration versus configuration used in the generic analysis.

The report identified two plant specific differences from the generic configuration. However, it did not provide any basis for the acceptability of these differences. The differences dealt with RCIC instrumentation with respect to high drywell pressure and high suppression pool water level. In the generic plant analysis high drywell pressure concurrent with low steam line pressure is used for isolation of the RCIC turbine exhaust system to protect against operation at high pressures. In WNP-2, the high drywell pressure signal is not used for isolation purposes. Also in the generic plant analysis a high suppression pool water level signal automatically switches RCIC suction from the condensate storage tank to the suppression pool to ensure that suppression pool loads, as a result of excess water, are not exceeded. At WNP-2, this transfer is done manually.

The licensee by their letter of July 28, 1992, provided the basis for the acceptability of these differences. In both cases, the differences do not significantly affect the RCIC availability and overall water injection failure frequency and, therefore, the generic analysis is applicable to WNP-2.

Based on the review of the plant specific analysis for the RCIC system the staff concludes that the licensee has adequately addressed the conditions set forth by the staff in endorsing the GE Topical Report GENE-770-06-2 and has provided acceptable support for the proposed extensions for STIs for RCIC from monthly to quarterly and AOT from 2 hours to 6 hours for test and 1 hour to 24 hours for repair.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Washington State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 20520). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Garg, NRR

Date: June 10, 1993