Docket No. 50-397

June 17, 1994

Mr. J. V. Parrish (Mail Drop 1023)
Assistant Managing Director, Operations
Washington Public Power Supply System
P. O. Box 968
Richland, Washington 99352-0968

Dear Mr. Parrish:

SUBJECT: ISSUANCE OF AMENDMENT FOR THE WASHINGTON PUBLIC POWER SUPPLY SYSTEM (WPPSS) NUCLEAR PROJECT NO. 2 (TAC NO. M88756)

The Commission has issued the enclosed Amendment No. 125 to the Facility Operating License No. NPF-21 for WPPSS Nuclear Project No. 2. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated February 8, 1994, as supplemented March 25, 1994.

The amendment increases the stroke time, as specified in TS Table 3.6.3-1, for reactor core isolation cooling (RCIC) valve RCIC-V-8 from 13 seconds to 26 seconds and deletes the Note (j) reference from RCIC-V-8 and RCIC-V-63. Note (j) indicates that the stroke time specified in the table reflects the requirement for containment isolation only.

A copy of the related Safety Evaluation is also enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by William D. Reckley
James W. Clifford, Senior Project Manager
Project Directorate IV-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No.125to NPF-21
2. Safety Evaluation

cc w/enclosures:
See next page

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*See previous concurrence

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NAME	DFoster-Curseen	MPadovan	SPeterson*	EHOLLER	TQuay
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 17, 1994

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Assistant Managing Director, Operations
Washington Public Power Supply System
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William D Reckley for

James W. Clifford, Senior Project Manager
Project Directorate IV-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

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2. Safety Evaluation

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See next page

Mr. J. V. Parrish
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 125
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Washington Public Power Supply System (licensee) dated February 8, 1994, as supplemented March 25, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 125 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective on the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Wm. D Reckley for:

Theodore R. Quay, Director
Project Directorate IV-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 17, 1994

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 6-24	3/4 6-24

TABLE 3.6.3-1 (Continued)

PRIMARY CONTAINMENT ISOLATION VALVES

VALVE FUNCTION AND NUMBER	VALVE GROUP(a)	MAXIMUM ISOLATION TIME (Seconds)
a. Automatic Isolation Valves (Continued)		
Reactor Core Isolation Cooling		
RCIC-V-8	8	26
RCIC-V-63	8	16
RCIC-V-76	8	22
Low Pressure Core Spray		
LPCS-V-12	10	180
High Pressure Core Spray		
HPCS-V-23	11	180
b. Excess Flow Check Valves(e)		
Containment Atmosphere		N.A.
PI-EFC-X29b		
PI-EFC-X29f		
PI-EFC-X30a		
PI-EFC-X30f		
PI-EFC-X42c		
PI-EFC-X42f		
PI-EFC-X61c		
PI-EFC-X62b		
PI-EFC-X69f		
PI-EFC-X78a		

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 125 TO FACILITY OPERATING LICENSE NO. NPF-21

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

NUCLEAR PROJECT NO. 2

DOCKET NO. 50-397

1.0 INTRODUCTION

By application dated February 8, 1994, supplemented by a March 25, 1994, letter, the Washington Public Power Supply System (the licensee) applied for an amendment to the Nuclear Project No. 2 (WNP-2) operating license. The additional information contained in the March 25, 1994, letter was clarifying in nature, was within the scope of the initial notice, and did not affect the NRC staff's proposed no significance hazards consideration determination.

The proposed amendment would change Technical Specification (TS) Table 3.6.3-1, "Primary Containment Isolation Valves," by extending the maximum allowable isolation time interval for closure of reactor core isolation cooling (RCIC) turbine steam supply outboard isolation valve RCIC-V-8 from 13 to 26 seconds. In addition, Table 3.6.3-1 would also be revised to delete references to its Note (j) for RCIC-V-8 and RCIC turbine steam supply inboard isolation valve RCIC-V-63. Note (j) of Table 3.6.3-1 is used to identify those valves listed in the table for which radiological dose limitations are not the only consideration in establishing the minimum allowable closure times, but are the most limiting consideration (e.g., minimum closure time requirements based on high-energy line break (HELB) environmental or dynamic effects considerations exist but are not as restrictive).

The application was initiated as a result of a need to change the valve operator pinion and worm gearing to increase the available torque. This need was identified by Generic Letter 89-10 analyses which determined that additional torque margin is desirable to provide additional assurance of capability to close under high differential pressure.

2.0 DISCUSSION AND EVALUATION

The RCIC system provides a means of feeding water to the reactor vessel for decay heat removal when the secondary plant is isolated. The RCIC system is located outside the primary containment but has piping which penetrates the primary containment boundary. The RCIC turbine steam supply line is one such line. This line has two automatic, power-operated isolation valves inside containment (a large block valve, RCIC-V-8, and a smaller bypass line valve, RCIC-V-76, for warmup purposes) and one automatic power-operated isolation valve, RCIC-V-63, outside containment. The valves are normally open, which is the desired position in the event of a design-basis accident (DBA). The

automatic controls for these valves are provided for the purpose of isolating a break in the RCIC steam supply line outside containment. Automatic isolation upon a DBA-loss-of-coolant accident (LOCA) signal (i.e., containment high pressure/reactor vessel low level) is not provided for the RCIC steam line as it is desirable that the RCIC system be available during the accident. RCIC isolation requirements are based on radiological dose criteria for independent breaks in lines outside containment (see Standard Review Plan (SRP) Section 15.6.2) and the vulnerability of safety-related equipment due to HELB outside containment (see SRP Section 3.6.1). HELB criteria specify that a break in a line outside containment should not threaten safety-related equipment. A break in the RCIC steam line is not postulated to occur concurrently with a DBA-LOCA. The most limiting requirements are used as the basis for establishing TS operability requirements.

The current stroke time limitations for RCIC-V-8 and RCIC-V-63, as specified in the TS, are 13 seconds and 16 seconds, respectively. These values are the original licensing values. They were based on actual stroke time capability of the valves and were accepted based on their being within the 60-second generic acceptance criterion of SRP Section 6.2.4. In Amendment 26, Note (j) was added to the table when it was found that HELB equipment environmental qualification considerations indicated a need for a more restrictive 10-second isolation response time. The final safety analysis report (FSAR) was revised to reflect the 10-second requirement for the RCIC steam supply isolation valves.

The licensee has since reanalyzed the HELB effects and determined that a 26-second isolation response time limit for a RCIC steam line break would not result in unacceptable radiological consequences or HELB equipment environmental conditions. In a supplemental letter dated March 25, 1994, the licensee confirmed that (1) the main steam line break continues to be the limiting steam line break accident for radiological dose consequences and (2) current equipment environmental qualification requirements bound the new RCIC steam line break conditions. Based on these findings, the proposed amendment is acceptable.

The staff has reviewed the licensee's application and determined that the appropriate acceptance criteria are met. The staff, therefore, concludes that the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Washington State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no

significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 24754). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Long

Date: June 17, 1994