

June 15, 1992

Mr. G. C. Sorensen, Manager
Regulatory Programs
Washington Public Power Supply System
3000 George Washington Way
P. O. Box 968
Richland, Washington 99352

Dear Mr. Sorensen:

SUBJECT: ISSUANCE OF AMENDMENT FOR THE WASHINGTON PUBLIC POWER SUPPLY SYSTEM
NUCLEAR PROJECT NO. 2 (TAC NO. M83194)

The Commission has issued the enclosed Amendment No.107 to the Facility Operating License No. NPF-21 for WPPSS Nuclear Project No. 2. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated April 10, 1992.

The amendment removes the schedule for withdrawal of reactor vessel material specimens (TS Table 4.4.6.1.3-1, "Reactor Vessel Material Surveillance Program-Withdrawal Schedule") and modifies TS Surveillance Requirement 4.4.6.1.3 to reflect removal of this schedule. This change is in response to guidance provided by the staff in Generic Letter 91-01, "Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from the Technical Specifications."

A copy of the related Safety Evaluation is also enclosed. A notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
Original signed by:
William M. Dean, Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 107 to NPF-21
- 2. Safety Evaluation

cc w/enclosures:
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Docket No. 50-397

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NAME	DFoster <i>Doster</i>	WDean: <i>W</i>	MZOB <i>MZ</i>	TQuay
DATE	5/28/92	5/28/92	5/29/92	/ /92

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

June 15, 1992

Docket No. 50-397

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Washington Public Power Supply System
3000 George Washington Way
P. O. Box 968
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Sincerely,

A handwritten signature in cursive script that reads "William M. Dean".

William M. Dean, Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

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2. Safety Evaluation

cc w/enclosures:
See next page

Mr. G. C. Sorensen
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

cc:

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WNP-2 Plant Manager
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Richland, Washington 99352

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

NUCLEAR PROJECT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107
License No. NPF-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Washington Public Power Supply System (licensee) dated April 10, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-21 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 107 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of the date of issuance and must be fully implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Theodore R. Quay, Director
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 15, 1992

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 107 TO FACILITY OPERATING LICENSE NO. NPF-21

DOCKET NO. 50-397

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE

xxiv
3/4 4-19
3/4 4-22

INSERT

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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.6.1.2 The reactor coolant system temperature and pressure shall be determined to be to the right of the criticality limit line of Figure 3.4.6.1 curve C within 15 minutes prior to the withdrawal of control rods to bring the reactor to criticality and at least once per 30 minutes during system heatup.

4.4.6.1.3 The reactor vessel material surveillance specimens shall be removed and examined, to determine changes in reactor pressure vessel material properties as required by 10 CFR Part 50, Appendix H in accordance with the NRC approved schedule. The results of these examinations shall be used to update the curves of Figure 3.4.6.1.

4.4.6.1.4 The reactor vessel flange and head flange temperature shall be verified to be greater than or equal to 80°F:

- a. In OPERATIONAL CONDITION 4 when reactor coolant system temperature is:
 1. $\leq 100^{\circ}\text{F}$, at least once per 12 hours.
 2. $\leq 90^{\circ}\text{F}$, at least once per 30 minutes.
- b. Within 30 minutes prior to and at least once per 30 minutes during tensioning of the reactor vessel head bolting studs.

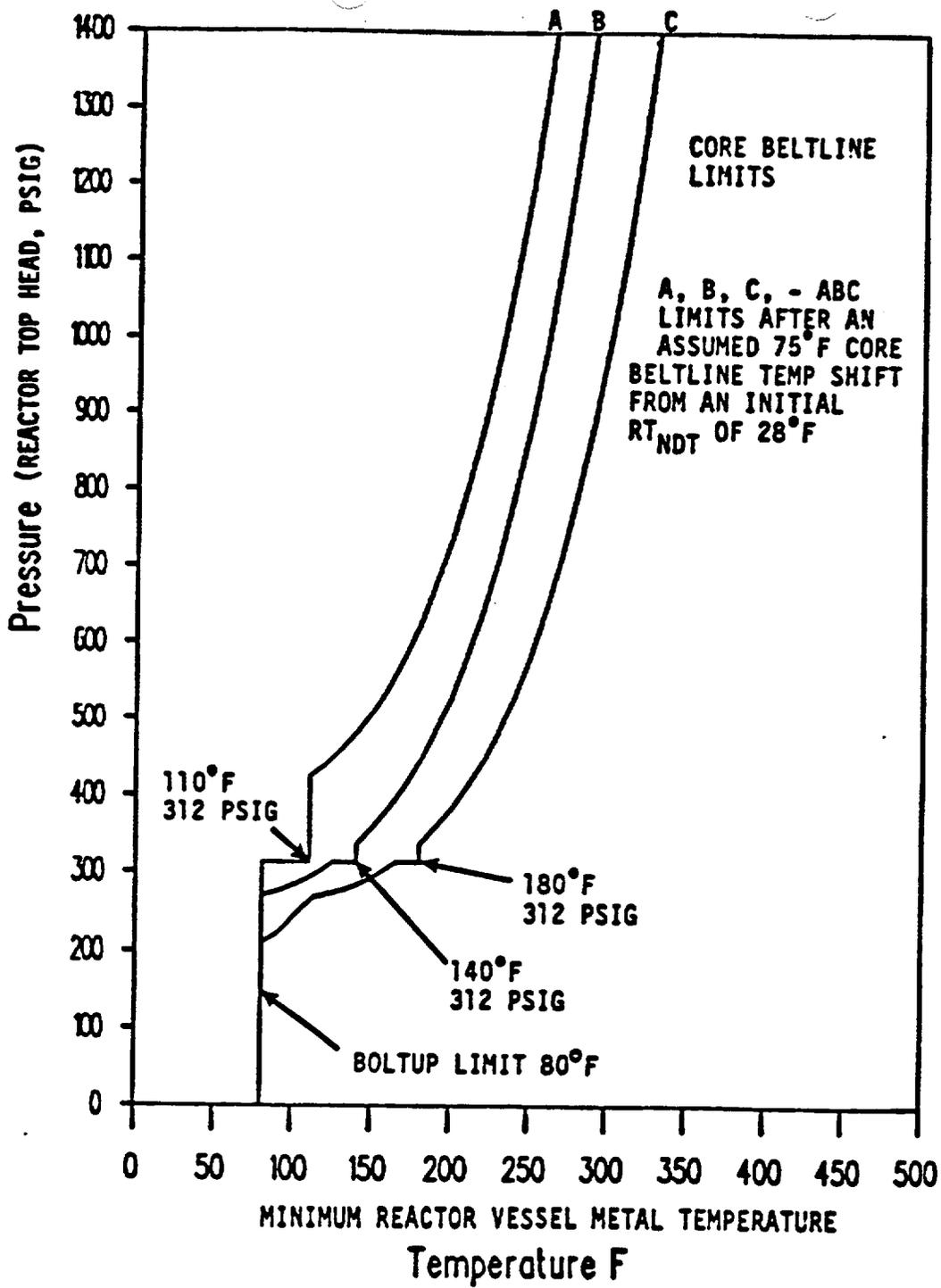


FIGURE 3.4.6.1

MINIMUM REACTOR VESSEL METAL TEMPERATURE VERSUS
 REACTOR VESSEL PRESSURE

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 107 TO FACILITY OPERATING LICENSE NO. NPF-21
WASHINGTON PUBLIC POWER SUPPLY SYSTEM
NUCLEAR PROJECT NO. 2
DOCKET NO. 50-397

1.0 INTRODUCTION

By letter dated April 10, 1992, Washington Public Power Supply System (Supply System) submitted a request for changes to the Technical Specifications (TS) for Nuclear Project No. 2. The proposed changes would remove TS Table 4.4.6.1.3-1, which provides the schedule for reactor vessel material specimen withdrawal. Guidance on the proposed TS change was provided by Generic Letter 91-01, of January 4, 1991, to all holders of operating licenses or construction permits for nuclear power reactors.

2.0 EVALUATION

Technical Specification 3/4.4.6, "Pressure/Temperature Limits," contains a limiting condition for operation for the reactor coolant system (RCS) that limits the rate of change in temperature and pressure to values consistent with fracture toughness requirements of the American Society of Mechanical Engineers (ASME) Code and Appendix G to Part 50 of Title 10 of the Code of Federal Regulations (10 CFR Part 50). Changes in the values of these limits are necessary because the fracture toughness properties of ferritic materials in the reactor vessel change as a function of the reactor operating time (neutron fluence).

For this reason, the TS include a surveillance requirement, TS 4.4.6.1.3, to require the removal and examination of the irradiated specimens of reactor vessel material. The licensee examines the specimens to determine the changes in material properties in accordance with the requirements of Appendix H to 10 CFR Part 50. Table 4.4.6.1.3-1 identifies the material specimens and specifies the schedule for removal of each specimen.

The removal of the schedule for withdrawing material specimens from the TS will eliminate the necessity of a license amendment to make changes to this schedule. However, Section II.B.3 of Appendix H to 10 CFR Part 50 requires the submittal of a proposed withdrawal schedule for material specimens to the NRC and approval by the NRC before implementation. Hence, adequate regulatory controls exist to control changes to this schedule without the necessity of subjecting it to the license amendment process.

The WNP-2 Final Safety Analysis Report (FSAR) Section 5.3.1.6.1 already describes the withdrawal schedule. The licensee has provided a commitment to update the FSAR to reflect the details of the removed schedule in the next revision of the FSAR. In addition, the licensee will include any subsequent NRC-approved revisions to this schedule in an update of the FSAR. The inclusion of the withdrawal schedule in the FSAR provides a source for this information that is readily available as a reference for NRC inspectors and other staff use. Finally, the surveillance requirements for removing material specimens remain unchanged except for removal of the reference to Table 4.4.6.1.3-1.

The licensee has proposed a change to TS 3/4.4.6 that is consistent with the guidance provided in Generic Letter 91-01 for the removal of Table 4.4.6.1.3-1 from TS. The NRC has reviewed this matter and finds the proposed changes to the TS for WNP-2 are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Washington State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment involves changes in recordkeeping, reporting, or administrative procedures or requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 20520). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Thomas G. Dunning, OTSB/DOEA
William M. Dean, PDV/DRPW

Date: June 15, 1992