

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

July 22, 2002

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 02-124A
NL&OS/ETS R0
Docket No. 50-339
License No. NPF-7

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNIT 2
ONE-TIME EXTENSION OF UNIT 2 SURVEILLANCE INTERVAL
FOR QUENCH SPRAY AND RECIRCULATION SPRAY NOZZLES

In a letter dated February 26, 2002 (Serial No. 02-124), Virginia Electric and Power Company (Dominion) requested amendments, in the form of changes to the Technical Specifications to Facility Operating Licenses Numbers NPF-4 and NPF-7 for North Anna Power Station Units 1 and 2, respectively. The proposed changes will revise the surveillance frequency of the Quench Spray System and Recirculation Spray System spray header nozzles from a periodic surveillance to a performance-based surveillance. In a telephone conference call on July 17, 2002 the NRC staff informed Dominion that they could not complete their review of the proposed change in time to support the upcoming Unit 2 Fall refueling outage. However, the staff indicated that they should be able to support a one-time extension of the surveillance interval for Unit 2 until the next refueling cycle currently scheduled to begin in the Spring of 2004. Based on the NRC current review status of the proposed Technical Specification change, Dominion agrees to a Technical Specification change, which will permit a one-time extension of the Quench Spray System and Recirculation Spray System spray header nozzles surveillance frequency.

This one-time extension of the North Anna Unit 2 Quench Spray and Recirculation Spray Systems spray header nozzles surveillance interval will permit extending the surveillance interval until the Spring 2004 Unit 2 refueling outage. The attachment to this letter provides proposed Technical Specifications pages with footnotes to permit the one-time extension. The ITS Bases pages contained in the Attachment are "for information only," as the Bases are revised under the Bases Control Program. Due to the possible delay in the implementation of improved Technical Specifications until after the upcoming Fall Unit 2 refueling outage, the footnote is provided in both current Technical Specifications and improved Technical Specifications formats. It is our intention that this one-time extension apply only to Unit 2 and that the previously proposed Technical Specifications changes for Units 1 and 2 remain in review.

We have evaluated the proposed Unit 2 one-time extension of the Quench Spray System and Recirculation Spray System spray header nozzles surveillance frequency and have determined that it is more restrictive than the proposed change originally

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requested and is, therefore, bounded by the original significant hazards consideration determination. Specifically, it does not involve a significant hazards consideration as defined in 10 CFR 50.92. The basis for our determination, which was provided in the February 26, 2002 submittal remains valid. We have also determined that operation with the proposed one-time extension of the surveillance frequency will not result in any significant increase in the amount of effluents that may be released offsite and no significant increase in individual or cumulative occupational radiation exposure. Therefore, the proposed one-time extension remains eligible for categorical exclusion as set forth in 10 CFR 51.22(c)(9).

If you have any further questions or require additional information, please contact us.

Very truly yours,



Leslie N. Hartz
Vice President – Nuclear Engineering

Attachments

Commitments made in this letter: None

cc: U.S. Nuclear Regulatory Commission
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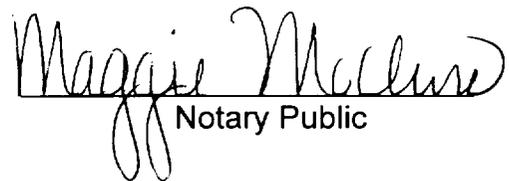
Subject: One-Time Extension – U2 Surv. Interval
Quench Spray & Recirc. Spray Nozzles

COMMONWEALTH OF VIRGINIA)
)
COUNTY OF HENRICO)

The foregoing document was acknowledged before me, in and for the County and Commonwealth aforesaid, today by Leslie N. Hartz, who is Vice President - Nuclear Engineering, of Virginia Electric and Power Company. She has affirmed before me that she is duly authorized to execute and file the foregoing document in behalf of that Company, and that the statements in the document are true to the best of her knowledge and belief.

Acknowledged before me this 22nd day of July, 2002.

My Commission Expires: March 31, 2004.


Notary Public

(SEAL)

Attachment

**Proposed Footnote for Extension of the Unit 2
Quench Spray and Recirculation Spray Systems
Spray Nozzle 10-year Surveillance Interval**

**North Anna Power Station
Unit 2
Virginia Electric and Power Company
(Dominion)**

**Current Technical Specifications Format
North Anna Unit 2**

CONTAINMENT SYSTEMS

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT QUENCH SPRAY SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.2.1 Two independent containment quench spray subsystems shall be OPERABLE.

APPLICABILITY: MODE 1, 2, 3 and 4.

ACTION:

With one containment quench spray subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.2.1 Each containment quench spray subsystem shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position.
 2. Verifying the temperature of the borated water in the refueling water storage tank is within the limits shown on Figure 3.6-1.
- b. Verifying that on recirculation flow, each pump develops a discharge pressure of greater than or equal to 123 psig when tested pursuant to Specification 4.0.5.
- c. At least once per 18 months during shutdown, by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment Pressure – high-high signal.
 2. Verifying that each spray pump starts automatically on a Containment Pressure – high-high signal.
- d. At least once per 10 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.*

* Performance of Surveillance 4.6.2.1.d is not required prior to the Spring 2004 refueling outage. |

CONTAINMENT SYSTEMS

CONTAINMENT RECIRCULATION SPRAY SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. Verify each RS and casing cooling pump's developed head at the flow test point is greater than or equal to the required developed head. The frequency shall be in accordance with the Inservice Testing Program.
 - c. At least once per 18 months by:
 - 1. Verifying that on a Containment Pressure-High-High signal, each casing cooling pump starts automatically without time delay, and each recirculation spray pump starts automatically with the following time delays: inside 195 ± 9.75 seconds, outside 210 ± 21 seconds.
 - 2. Verifying that each automatic valve in the flow path actuates to its correct position on a Containment Pressure – high-high test signal.
 - d. At least once per 10 years by performing an air or smoke flow test through each spray header and verifying each spray nozzle is unobstructed.*
- 4.6.2.2.2 The casing coolant tank shall be demonstrated OPERABLE:
- a. At least once per 7 days by:
 - 1. Verifying the contained borated water volume in the tank, and
 - 2. Verifying the boron concentration of the water.
 - b. At least once per 24 hours by verifying the casing cooling tank temperature.

* Performance of Surveillance 4.6.2.2.1.d is not required prior to the Spring 2004 refueling outage. |

**Improved Technical Specifications Format
North Anna Unit 2**

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.6.3	Verify each QS automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	18 months
SR 3.6.6.4	Verify each QS pump starts automatically on an actual or simulated actuation signal.	18 months
SR 3.6.6.5	<p>-----NOTE----- For Unit 2, not required to be performed prior to the Spring 2004 refueling outage. -----</p> <p>Verify each spray nozzle is unobstructed.</p>	10 years

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.7.4	Verify each RS and casing cooling manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	31 days
SR 3.6.7.5	Verify each RS and casing cooling pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Inservice Testing Program
SR 3.6.7.6	Verify on an actual or simulated actuation signal(s): a. Each RS automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position; b. Each RS pump starts automatically; and c. Each casing cooling pump starts automatically.	18 months
SR 3.6.7.7	-----NOTE----- For Unit 2, not required to be performed prior to the Spring 2004 refueling outage. ----- Verify each spray nozzle is unobstructed.	10 years

BASES

SURVEILLANCE
REQUIREMENTS

SR 3.6.6.5 (continued)

spray nozzle is unobstructed and that spray coverage of the containment during an accident is not degraded. Due to the passive nature of the design of the nozzle and the non-corrosive design of the system, a test at 10 year intervals is considered adequate to detect obstruction of the nozzles.

This SR is modified by a Note that provides an allowance to the Frequency for Unit 2. This Note states that the SR is not required to be performed prior to the Spring 2004 refueling outage.

REFERENCES

1. UFSAR, Section 6.2.
 2. 10 CFR 50.49.
 3. 10 CFR 50, Appendix K.
 4. ASME Code for Operation and Maintenance of Nuclear Power Plants.
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BASES

SURVEILLANCE
REQUIREMENTS

SR 3.6.7.7 (continued)

This Surveillance is modified by a Note that provides an allowance to the Frequency for Unit 2. This Note states that the SR is not required to be performed prior to the Spring 2004 refueling outage.

REFERENCES

1. UFSAR, Section 6.2.
 2. 10 CFR 50.49.
 3. 10 CFR 50, Appendix K.
 4. ASME Code for Operation and Maintenance of Nuclear Power Plants.
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